#### URR measurements, evaluation, and testing for tantalum

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## Motivation

- Tantalum was chosen:
  - On NCSP list of nuclear data needs
  - Single isotope and thus easier analysis
  - Has some cross section fluctuations in the URR
    - Similar to some actinides
- Develop methodology to test the URR self shielding accuracy
- The validation method can be used for actinides such as U-238 and U-235.









#### **Motivation: LSDS capture rate measurements**

- Discrepant evaluated libraries
- Lead Slowing Down Spectrometer (LSDS) study: Discrepancies between libraries in simulated capture rate



## **URR** options

- LSSF=0 (what we used for Ta)
  - File 2 URR parameters
    - Used to generate probability tables (or self-shielding factors)
  - File 3 background cross section (zero here)
  - Cross section = (File 2 generated cross section) + (File 3)
- LSSF=1
  - File 2 URR parameters
    - Used to calculated self-shielding factor SF= <F2 shielded>/<F2 not shielded>
  - File 3 Gross structure infinite dilution cross section
  - Cross section = SF x (File 3) (fixes the energy grid)
- Probability tables and self-shielding factors are generated in application codes
  - Used NJOY 21 (LSSF=0) + MCNP 6.1









## The big picture



### **Detectors and Measurements**

#### MELINDA (100m)



<sup>6</sup>Li glass (35m)



#### $C_6 D_6$ Detector (45m)



Highest energy

Low neutron

sensitivity

at RPI

resolution for capture

Designed with digital

acquisition system

- High energy resolution
- Fast timing
- Large active detector area
- Data-processing well understood

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- Relatively good energy resolution
- Fast timing
- Shorter flight-path enables greater count rate
- Better count rate allowsfreedom of neutrontargets







#### **Data to Evaluate**

- <sup>181</sup>Ta Evaluation Datasets:
- Capture Yield: 1 and 2 mm
- Transmission: 1, 3, and 6 mm



## **SAMMY Evaluation**





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#### **Data to Evaluate: RRR (one example)** End of ENDF/B-VIII.0 RRR:

- 304 eV resonance updated
- Transmission and capture yield are well resolved



#### Validation transmission data

- <sup>181</sup>Ta Validation Dataset:
  - Transmission: 12 mm
  - <sup>238</sup>U verification dataset



## **Transmission for validation**

### <sup>6</sup>Li doped scintillating glass detector

• 2 PMT's viewing a light tight aluminum case











#### **URR Transmission Enhancement Math**

Neutron transmission through a sample:  $T(E) = e^{-n\sigma_t(E)}$ The "true" average transmission from energy  $E_1$  to  $E_2$ 

$$\langle T \rangle = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n\sigma_t(E)} dE = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n[\sigma_t(E) + \langle \sigma_t \rangle - \langle \sigma_t \rangle]} dE$$

Enhancement due to 
$$\sigma_t(E)$$
 fluctuations  
 $\langle T \rangle = e^{-n\langle \sigma_t \rangle} \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n[\sigma_t(E) - \langle \sigma_t \rangle]} dE$  Note: positive and negative contributions  
 $\operatorname{sft}(E) = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-n[\sigma_t(E) - \langle \sigma_t \rangle]} dE$   $\overline{T} = e^{-n\langle \sigma_t \rangle}$  Self-shielded  
 $\langle T \rangle = \overline{T} * sft(E)$  where  $\operatorname{sft}(E) > 1, \rightarrow \langle T \rangle > \overline{T} \rightarrow \langle \sigma_t \rangle < \overline{\sigma_t}$ 

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12

Evaluation procedure must preserve the fluctuations of  $\sigma_t(E)$ 

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## **URR Transmission Enhancement Example**

- Example calculating neutron transmission through a 6 mm Ta sample
- If the cross section was known in high energy resolution, the "true" transmission:

$$\langle T \rangle = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-N \cdot \sigma_t(E)} dE = 0.59$$
  
• If we use only the cross section average  
$$\overline{T} = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-N \cdot \langle \sigma_t \rangle} dE = e^{-N \cdot \langle \sigma_t \rangle} = 0.51$$
  
• If we use only the cross section average  
$$\overline{T} = \frac{1}{E_2 - E_1} \int_{E_1}^{E_2} e^{-N \cdot \langle \sigma_t \rangle} dE = e^{-N \cdot \langle \sigma_t \rangle} = 0.51$$

- Fluctuations enhance transmission and thus reduce the effective cross section (relative to the average) hence the term self shielding
- When measuring the total cross section with a thick sample a correction for the self shielding is needed.
  - Can use two sample thicknesses  $\rightarrow$
  - Can use a model based approach  $\rightarrow$  SESH



Froehner, et al, "Cross-section fluctuations and self-shielding effects in the unresolved resonance region ", International Evaluation Co-operation volume 15 (NEA-WPEC--15), Nuclear Energy Agency of the OECD, NEA, (1995).





### **Resonance Self-Shielding effect in Ta**

- The effect of self shielding is shown by turning off the URR treatment in MCNP
- Near 400 eV self-shielding reduces the transmission by a factor of about 4



#### Validation Transmission Measurement



## Validation Transmission and evaluations

- Transmission for 12 mm sample
  - Grouped to have about 50 resonances per bin
- Observe the limitations of the URR treatment using JEFF-3.3, JENDL-4.0, and ENDF/B-VIII.0



### **Multi-Region URR**



### **RPI Evaluation: Updated JEFF-3.3**

- Updated RRR and URR parameters
- Very sensitive to  $a_c$ , D and other  $\langle Pars \rangle$
- Using the RPI evaluation we can improve agreement with measured data



## **Multi-Region URR**

- Performs reasonably well compared to thick-sample transmission
- *(Pars)* for each region are less constrained
- Separated parameters do not significantly improve overall shape



## **Extended RRR fit**

RRR parameter fit to 4 keV

- Mughabghab publishes parameters up to 4 keV
- Fit  $E_{\lambda}$ ,  $\Gamma_{\gamma}$ ,  $\Gamma_n$
- No significant improvement



#### **Summary of all evaluations**

- Look very similar
  - extended resonance evaluation seems to under predict shelf shielding



#### **Example for U-238**

- MCNP simulation
  - Used 17 mm thick sample (can use thicker)
  - The self shielding effect is visible in the URR (E>20 keV)



# Conclusions

#### <sup>181</sup>Ta

- RRR representation is more accurate for calculating transmission (up to 2.4 keV)
- ENDF/B-VIII.0 needs to be updated
  - RRR treatment for ENDF/B evaluation should be extended (JEFF and JENDL  $\rightarrow$  2.4 keV)
  - URR treatment for ENDF/B evaluation should be extended beyond 5 keV (JEFF and JENDL  $\rightarrow$  100 keV)
- New RPI data provide best resolution to date and can be used to create a better <sup>181</sup>Ta evaluation

#### **Transmission** Validation

- Thick sample transmission measurement capable of validating the URR and RRR/URR boundary
  - Better targeted for validating the URR than previous high energy benchmark experiments
- Validation transmission is very sensitive to resonance self-shielding, *a<sub>c</sub>*, *R'* and other URR parameters
- A novel method that can help improve cross section evaluations that affect criticality







