

On the International Radiation Characterization Benchmarks Experiment Project (IRCBEP):

User's Point of View

O. Cabellos
oscar.cabellos@upm.es

INDUSTRIALES
ETSII | UPM

1. The necessity of Nuclear Benchmarks
2. Handbooks for ND B&V
3. Purpose of these Handbooks
4. Content of Experimental Handbooks
5. The need of IRCBEP: Identifying quantities
 - 5.1 Integro-Differential Verification and Validation
 - 5.2 Maxwellian-Avg. n-Induced Cross-Sect, KADoNIS
 - 5.3 s_0 and Q_0 values, Kayzero library
 - 5.4 Resonance Integrals
 - 5.5 Decay Heat and Inventory predictions
 - 5.6 Other Experimental Data
6. Conclusion

The necessity of nuclear benchmarks

- Nuclear Safety

- Safety Assessment
- Safer Operation
- ...


- Computational methods

- Code verification
- Cross-section data


ND evaluation projects utilize benchmarks on:

- Criticality -> **ICSBEP**
- Shielding/transmission -> **SINBAD**
- Reactor Physics -> IRPHEP
- Spent nuclear fuel - > SFCOMPO
- ...
- Others

□ B&V of ND trough databases ... JEFF Project



Nuclear Energy Agency



B&V of neutron XSs through NEA Databases

B&V/WG

- Introduction
 - Past and recent B&V
 - Participants
- **B&V neutron cross-sections**
- **B&V other nuclear data**
- **B&V covariance data**
- **Agenda**
 - 1st B&V JEFF WG
- **Conclusion**

- Spectral average cross sections (Thermal,..., MXW, ²⁵²Cf/²³⁵U fission, ..., integral resonances, ... MACS,...)
 - EXFOR
- Criticality
 - ICSBEP/DICE
 - IDAT/IRPhEP
- Kinetics (β_{eff} ,...), spectral indexes
 - ICSBEP/DICE
 - IDAT/IRPhEP
- Transmission and Shielding
 - ICSBEP
 - SINBAD
- Burnup
 - Depletion: SFCOMPO
 - Reactivity: IRPhEP (EPRI program)

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Source: JEFF/DOC-1729. "Introduction to JEFF/WG on B&V". O. Cabellos (2015)



Project	International Project	Official Activity at ...
ICSBEP => DICE tool	International Criticality Safety Benchmark Evaluation Project https://www.oecd-nea.org/science/wpncs/icsbep/	WPNCNS The Working Party on Nuclear Criticality Safety (WPNCNS) WPEC ... Working Party on International Nuclear Data Evaluation Co-operation (WPEC) <ul style="list-style-type: none"> ○ SG45 "Validation of Nuclear Data Libraries (VaNDaL) Project" ○ SG40/CIELO, SG39/SG46,...
IRPhEP => IDAT tool	International Reactor Physics Experiment Evaluation Project https://www.oecd-nea.org/science/wprs/irphe/	WPRS ... Working Party on Scientific Issues of Reactor Systems (WPRS) <ul style="list-style-type: none"> ○ WPEC ... SG33/SG39/SG46
SINBAD => NEA tool ?	Shielding Integral Benchmark Archive and Database https://www.oecd-nea.org/science/wprs/shielding/	WPRS ... Working Party on Scientific Issues of Reactor Systems (WPRS) WPEC ... WP on International ND Evaluation Cooperation (WPEC)/SG47 "Use of Shielding Integral Benchmark Archive and Database for ND Validation" <ul style="list-style-type: none"> ○ SG40/CIELO, SG39/SG46,...
SFCOMPO 2.0 => SFCOMPO	Database of measured isotopic concentrations of spent nuclear fuel, with operational histories and design data https://www.oecd-nea.org/sfcompo/	WPNCNS... Expert Group on Assay Data of Spent Nuclear Fuel (ADSNF)
EXFOR => IAEA tools => NEA tools	An extensive compilation of experimental nuclear reaction data https://www-nds.iaea.org/exfor/exfor.htm	IAEA/NDS ... The International Network of Nuclear Reaction Data Centres (NRDC) NEA/DB JANIS Software

The purpose of these Handbooks can be summarized:

- **Compile benchmark-experiment data into standardized format** that allows analysts to easily use the data to validate calculational techniques and cross-section data
- **Evaluate the data and quantify overall uncertainties** through sensitivity analysis
- **Streamline the necessary step of validating** computer codes and nuclear data with experimental data.
e.g. develop models (reference/detail) with verified magnitudes
- **Preserve highly valuable experimental data**

Content & Format of evaluations:

1. Detailed description of the experiment:
 - Geometry, materials,... relevant information
2. Evaluation of experimental data
 - Missing data, effects of uncertainties, ...
3. Benchmark specifications
 - To provide the data necessary to construct calculational Benchmark-model: Simplifies, Detailed,...
4. Provide results of sample calculations
 - Calculated results,... discrepancies between Benchmark values and calculated values
5. References
6. Appendix A. Typical input listings
7. Appendix B. Supporting information

Example: ICSBEP

Quality & Assurance

- thorough internal review
- independent peer review

The need of a database on **Radiation Characterization Benchmark Experiments**:

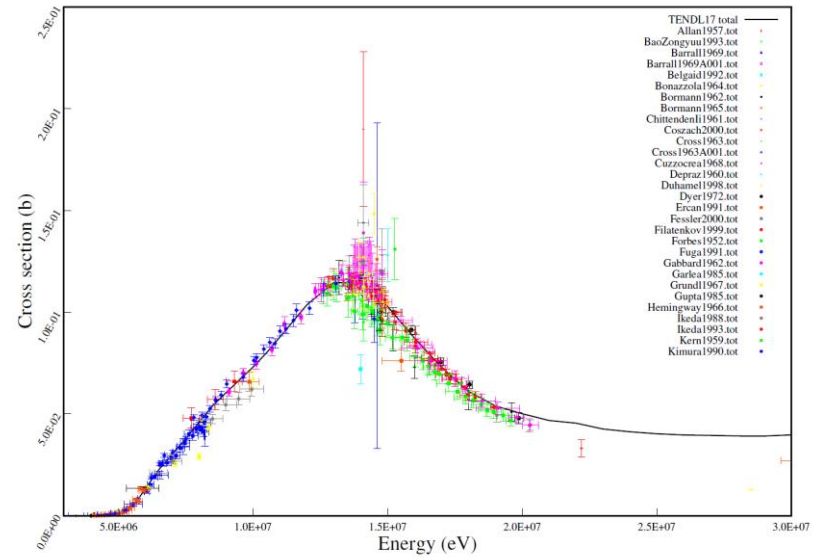
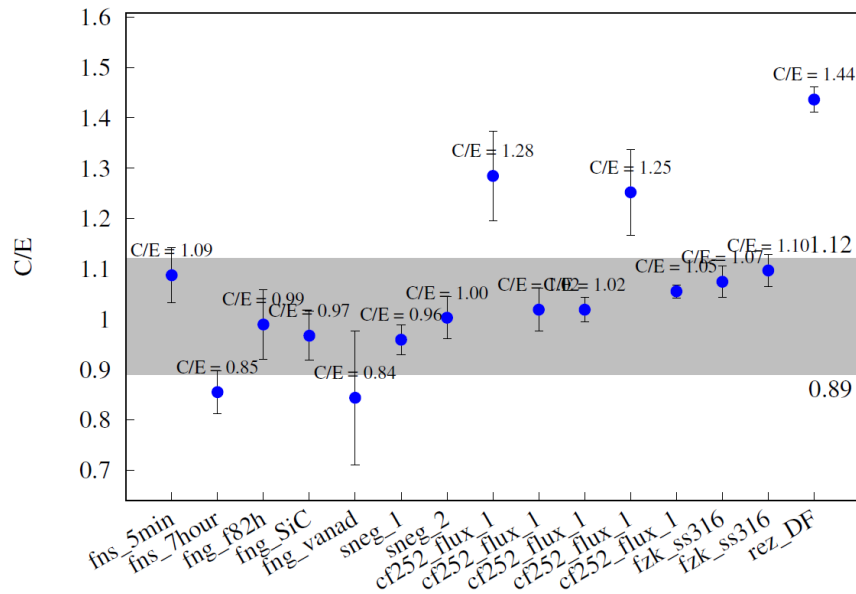
- Identifying quantities (not yet included in B&V) for “**Radiation Characterization**”
 - σ_0 thermal cross sections (2200 m/s)
 - RI
 - K0 - standardisation method of Neutron Activation Analysis (NAA)
 - $Q0 = I0 / \sigma0$

 - σ_{th} - spectrum-average thermal cross sections
 - Westcott g-factor

 - σ_{14} MeV
 - MACS - Maxwellian Averaged Cross Sections
 - ...

The need of a database on **Radiation Characterization Benchmark Experiments:**

□ Integro-Differential Verification and Validation



Source: UKAEA-R(18)004. February 2018

Californium-252 spontaneous fission data ← **already in EXFOR**

The need of a database on **Radiation Characterization Benchmark Experiments:**

□ Integro-Differential Verification and Validation

Content & Format of evaluations:

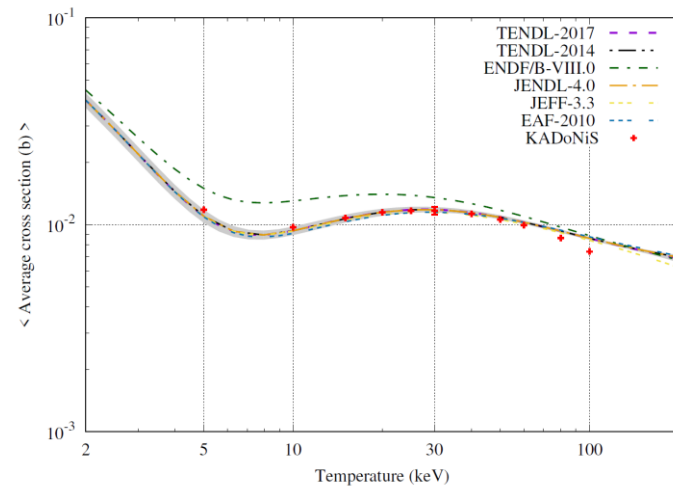
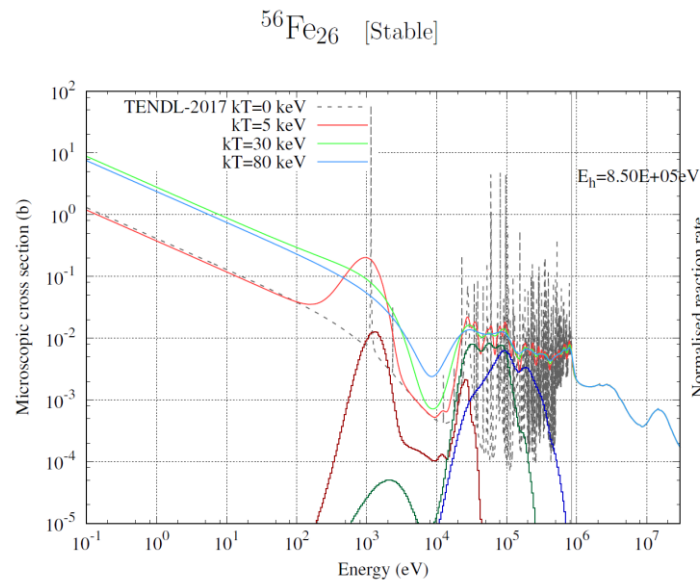
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Procedure:

- Exp. Data:
 - EXFOR data
 - Others Exp: JAEA, TUD, ...
- PREPRO/NJOY codes
- Database: Neutron Spectra
- Calculate: $C+\Delta C$

The need of a database on **Radiation Characterization Benchmark Experiments:**

- ❑ Maxwellian-Averaged Neutron-Induced Cross Sections for $kT=1$ keV to 100keV, KADoNiS



Source: UKAEA-R(18)005. February 2018

The need of a database on Radiation Characterization Benchmark Experiments:

❑ Maxwellian-Averaged Neutron-Induced Cross Sections for $kT=1$ keV to 100keV, KADoNiS

Content & Format of evaluations:

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 - Geometry, materials,... relevant information
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Procedure:

- Exp. Data:
 - EXFOR data?
 - Others Exp: KADoNiS
- PREPRO/NJOY codes
- MAXWAV code ?
- Calculate: $C+\Delta C$

The need of a database on Radiation Characterization Benchmark Experiments:

- ❑ **Comprehensive list of more than 100 reaction cross sections suitable for the activation technique: thermal cross sections (σ_0) and Q_0 values**

2.5.20. $^{63}\text{Cu}(n,\gamma)^{64}\text{Cu}$

The thermal cross sections and the Q_0 values are as follows:

Mughabghab [1]			Kayzero [2] / Nudat [3]		JENDL-4.0 [4]		ENDF/B-VII.1 [5]	
σ_0	$\Delta\sigma_0$	Diff.	σ_0	$\Delta\sigma_0$	σ_0	Diff.	σ_0	Diff.
[b]	[%]	[%]	[b]	[%]	[b]	[%]	[b]	[%]
4.50	0.4	-2.7	4.624	1.1	4.508	-2.5	4.471	-3.3

Mughabghab [1]			Kayzero [2]		JENDL-4.0 [4]		ENDF/B-VII.1 [5]	
Q_0	ΔQ_0	Diff.	Q_0	ΔQ_0	Q_0	Diff.	Q_0	Diff.
	[%]	[%]		[%]		[%]		[%]
1.104	1.7	-3.1	1.14	-	1.087	-4.6	1.086	-4.7

The measured values in the Kayzero library
<http://www.kayzero.com/>

Source: INDC (NDS) -0693

The need of a database on **Radiation Characterization Benchmark Experiments:**

- ❑ **Comprehensive list of more than 100 reaction cross sections suitable for the activation technique: thermal cross sections (σ_0) and Q_0 values**

Content & Format of evaluations:

1. Detailed description of the experiment:
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Procedure:

- Exp. Data:
 - EXFOR data
 - Others Exp: Kayzero library (A.Trkov)
- PREPRO/NJOY codes
- Calculate: C+ Δ C

The need of a database on **Radiation Characterization Benchmark Experiments:**

❑ Resonance Integrals.... data in EXFOR

Source: JRC report



JRC TECHNICAL REPORTS

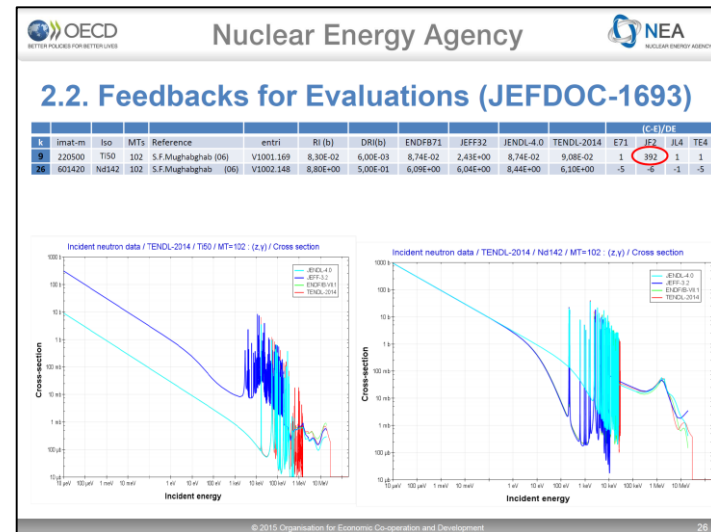
Comparison of resonance integrals of cross sections from JEFF-3.2 library for some problematic reactions

Gasper Zerovnik
Stefan Kopecky
Arjan Plompen
Peter Schillebeeckx

Author	Year	EXFOR Entry	E_{min} [eV]	RI [b]	Accepted
Heft	1978	12866.025	0.5	14.0 ± 0.9	Yes
Gleason	1975	10644.020	0.5	15.6 ± 0.5	Yes
Van Der Linden	1972	20643.015	0.55	13.8 ± 0.8	Yes
Breitenhuber	1970	20029.003	0.55	$15.4 (1 \pm 0.05)$	Yes
Nanjyo	1970	20314.002	0.63	14.1 ± 0.6	Yes
Schuman	1969	11687.002	0.5	11.3 ± 0.7	Yes
Schuman	1969	11687.004	0.5	14	No

Report on Integral Resonances: A comparison between EXFOR and ENDF/B-VII.1, JEFF-3.2, JENDL-4.0 and TENDL-2014.

O. Cabellos, OECD NEA/DB. NRDC Meeting 2015.



The need of a database on **Radiation Characterization Benchmark Experiments:**

□ Resonance Integrals.... data in EXFOR

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Procedure:

- Exp. Data: EXFOR data
- PREPRO/NJOY codes
- URR/CALENDF code?
- INTER code
- Input data for INTER:
Lower-Upper energy limits
- Calculate: $C+\Delta C$

The need of a database on **Radiation Characterization Benchmark Experiments:**

❑ Decay Heat and Inventory Predictions

➤ Fission Pulse Decay Heat

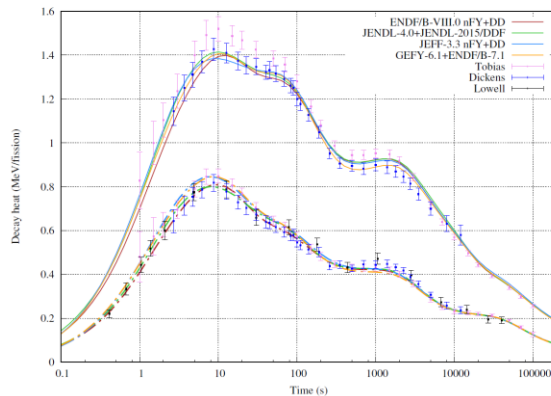
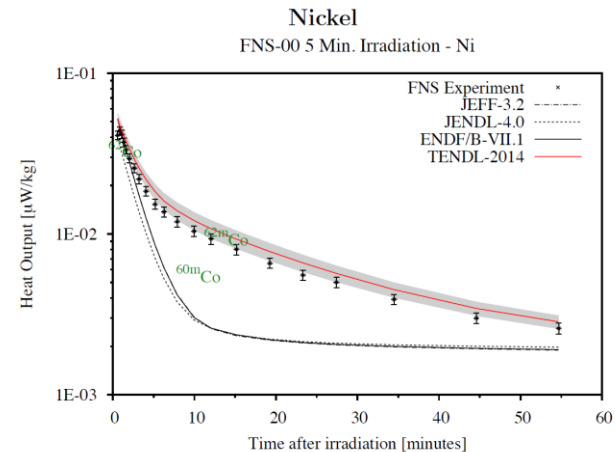


Figure 10: Total (solid) and beta (dash) decay heat from thermal pulse on ^{235}U .

“Meta-analysis performed by Tobias”

Source: UKAEA-R(18)003. February 2018

➤ Decay Heat and Inventory Predictions



“The lack of real chemical analysis of the irradiated samples, ... (page 15)”

Source: CCFE-R(15)25. January 2015



Integral experiments: Steel activation

PART I. Validation: integral activation experiments (FZKA-6764)

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Table. Calculation-to-experiment average ratios for the activity inventories induced in the SS316 intermediate energy activation experiment FZKA6764

This steel has been irradiated during 7,525 s with a neutron flux of $4.10 \cdot 10^{11}$ n/cm² s having a neutron spectrum very close to IFMIF

Conclusions:

- For the most important radionuclides to the total activity and contact dose rate (56Mn, 54Mn, 57Ni, 58Co and 60Co): reasonable agreement
- For the rest of isotopes, larger deviations of C/E from unity were found.

These discrepancies can arise from:

- the activation cross section library
- neglecting the sequential charged-particle reactions (C/E: V58~1.01, Mn52 ~1.08, Co55 ~0.95, Co56 ~1.07)
- uncertainties of initial composition

- Our results are in good agreement with those obtained by ALARA, ANITA and FISPACT codes

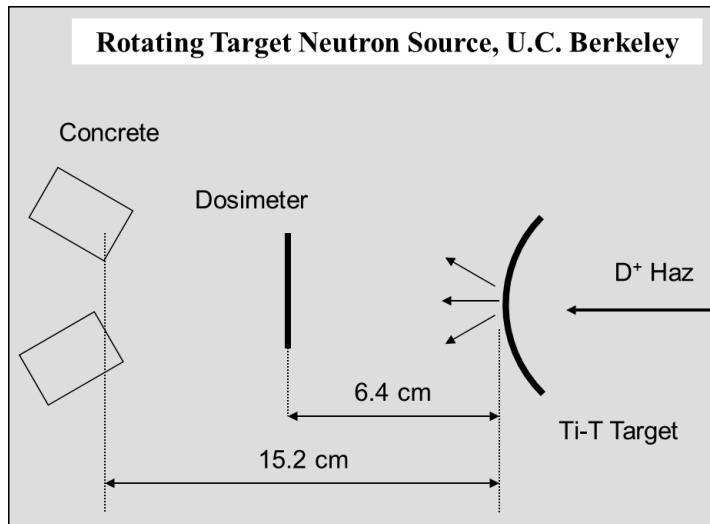
	IEAF2001			EAF2005				IEAF2001			EAF2005				
Isotope	C/E	C/E	Rel Err %	Isotope	C/E	C/E	Rel Err %	Isotope	C/E	C/E	Rel Err %	Isotope	C/E	C/E	Rel Err %
Sc 46	1.66	2.46	10.8	Ni 57	1.19	1.22	0.5								
Sc 48	1.28	1.74	15.3	Y 87m	0.07	0.15	77.3								
V 48	2.72	1.28	29.6	Y 87	0.46	0.82	63.3								
Cr 48	3.73	1.54	82.9	Y 88	1.00	0.61	25.4								
Cr 49	0.82	0.71	3.8	Zr 86	0.03	0.03	73.7								
Cr 51	1.00	1.09	3.3	Zr 88	0.99	0.45	9.2								
Mn 52	3.13	1.31	26.8	Zr 89	1.20	1.48	5.4								
Mn 54	1.02	1.16	9.4	Zr 97	0.03	0.02	3.0								
Mn 56	1.23	1.22	0.6	Nb 90	0.58	1.80	34.2								
Fe 52	1.44	4.34	19.9	Nb 92m	1.51	1.46	4.1								
Fe 59	1.07	1.18	17.8	Nb 95	1.53	1.65	6.1								
Co 55	2.12	1.45	64.1	Nb 95m	1.18	0.84	9.4								
Co 56	2.98	1.31	27.7	Nb 96	1.65	1.88	8.5								
Co 57	1.02	1.12	0.9	Mo 90	1.64	1.97	71.5								
Co 58	1.20	1.19	3.0	Mo 93m	2.50	1.81	44.2								
Co 60	1.12	1.03	3.3	Mo 99	1.18	1.29	1.7								
Co 61	2.26	2.66	17.5	Tc 99m	1.19	1.30	1.4								
Ni 56	1.57	0.37	45.2												

Source: EFFDOC-1101, "Improvements in the prediction capability of ACAB code to transmutation Analysis in IFMIF", O. Cabellos (2009)

Integral experiments: Concrete activation

Source: ACAB Code Manual: Validation. (2008)

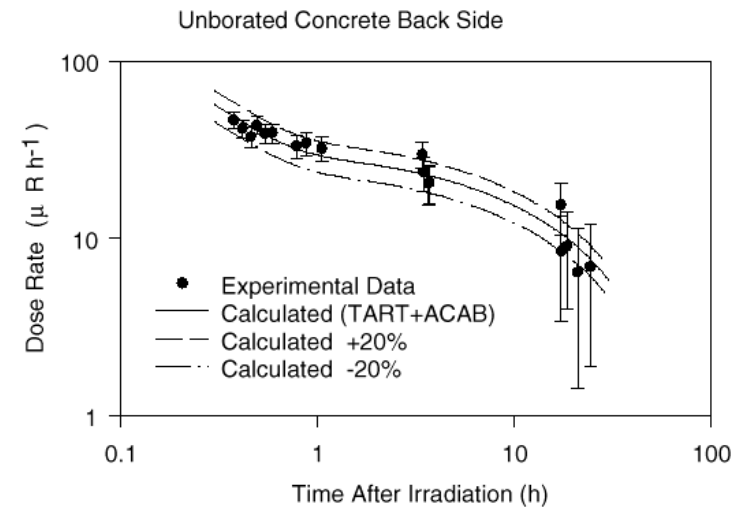
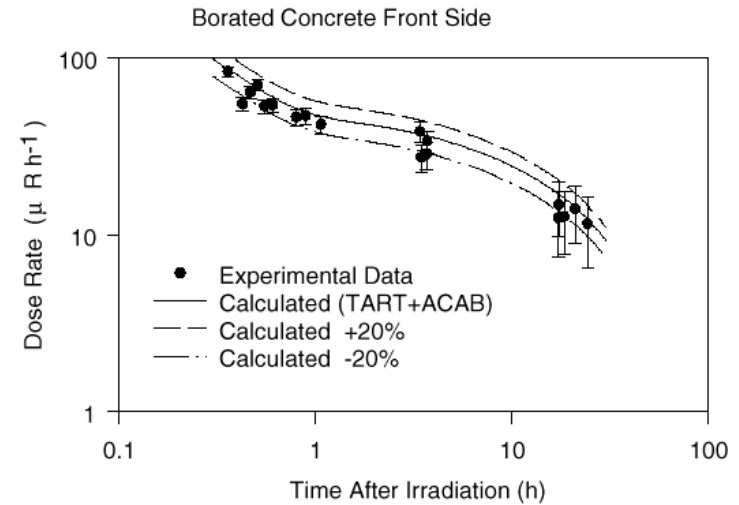
Source: "Experimental studies of concrete activation at the National Ignition Facility using the Rotating Target Neutron Source", A.P. Belian, J.F. Latkowski, E.C. Morse (1997)
17th IEEE/NPSS Symposium Fusion Engineering



Neutron intensity: $2.2 \cdot 10^{10}$ n/s \pm 20%

$\langle E \rangle = 15.2 \pm 0.1$ MeV

Irradiation time = 1h. 35 min.



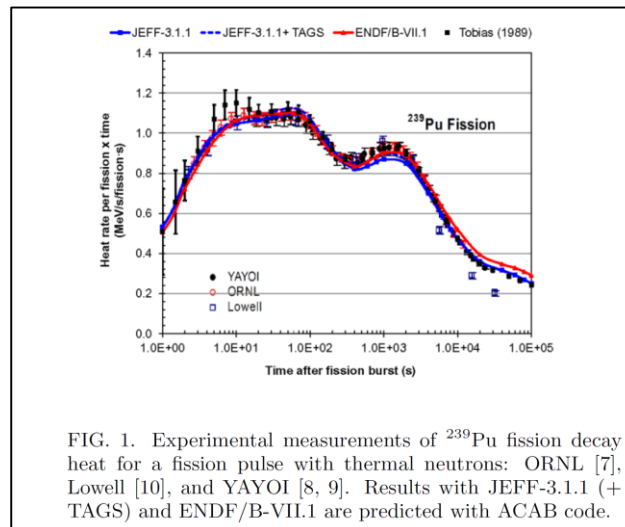
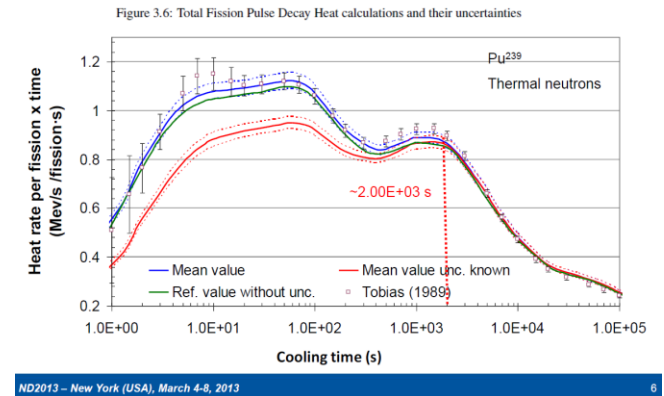
□ FPDH

“Comparison using Tobias data”

Source: ND2013, “Testing JEFF-3.1.1 and ENDF/B-VII.1 DD&FY nuclear data libraries with Fission Pulse Neutron Emission and Decay Heat Experiments”, O. Cabellos et al. (2013)



FP decay heat: Pu²³⁹-thermal



“Meta-analysis performed by Tobias”

- [6] A. Tobias, CEBG Report No. RD/B/6210/R89 (1989).
- [7] J. K. Dickens *et al.*, NUCL. SCI. ENG. **78**, 126 (1981).
- [8] M. Akiyama *et al.*, J. ATOMIC ENER. SOC. JAPAN **24**, 709 (1982) - in Japanese.
- [9] M. Akiyama *et al.*, J. ATOMIC ENER. SOC. JAPAN **24**, 803 (1982) - in Japanese.
- [10] W.A. Schier *et al.*, DOE/ER/40723-4 (1997).

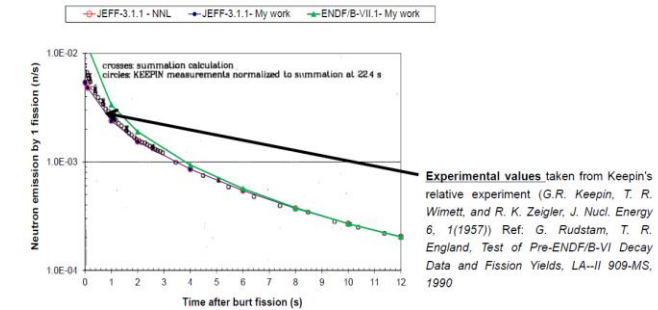
“Experiments”

Delayed neutron emission

Source: ND2013, "Testing JEFF-3.1.1 and ENDF/B-VII.1 DD&FY nuclear data libraries with Fission Pulse Neutron Emission and Decay Heat Experiments", O. Cabellos et al. (2013)

3.1 $n_{emit}(t)$ comparison with EXP.

Figure 3. An UPM/NNL comparison for the delayed neutron emission rate, $n_{emit}(t)$, from the activity of precursors after a fission pulse in ^{235}U .

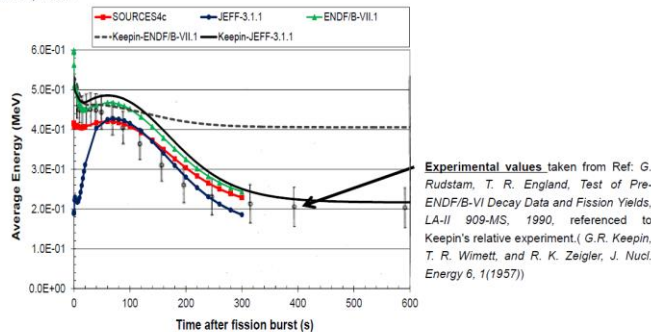


ND2013 – New York (USA), March 4-8, 2013

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3.3 Delayed Neutron Average Energy

Figure 6. Delayed neutron average energy calculated with JEFF-3.1.1, ENDF/B-VII.1 and SOURCES4C (LANL) code.

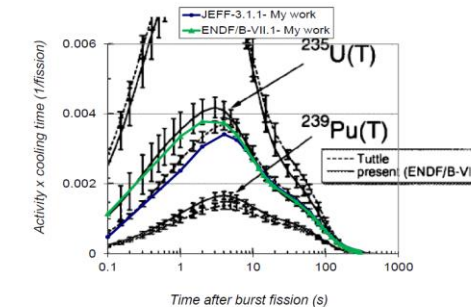


ND2013 – New York (USA), March 4-8, 2013

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3.1 $n_{emit}(t)$ comparison with EXP.

Figure 4. A comparison of UPM work with Ref: "Delayed Neutron Spectra and their Uncertainties in Fission Product Summation Calculations", T. Miyazono, M. Sagisaka, H. Ohta, K. Oyamatsu and M. Tamaki, PROCEEDINGS OF THE 1996 SYMPOSIUM ON NUCLEAR DATA, November 21-22, 1996, JAERI, Tokai, Ibaraki, Japan.



ND2013 – New York (USA), March 4-8, 2013

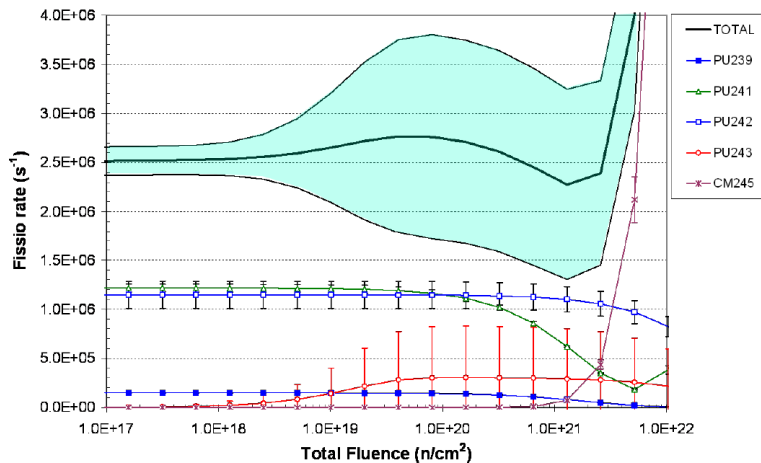
27

Other experimental data measurements ?

Source: EFFDOC-1117, "Assessment of fissionable material behaviour in fission chambers", O. Cabellos (2010)

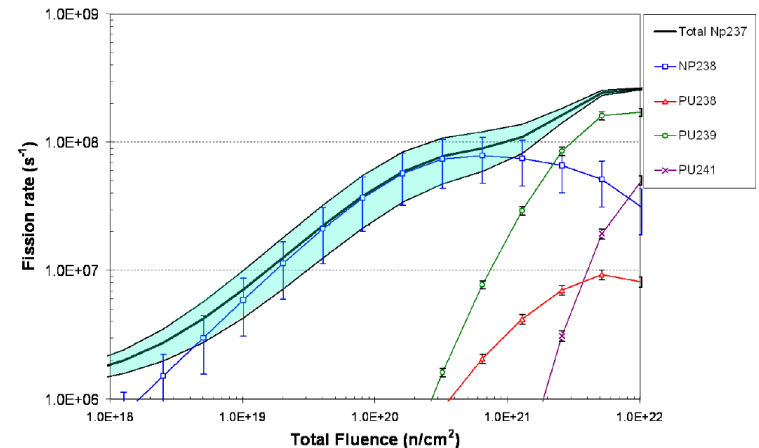
PART IV: Relative error in fission rates Pu242#1 in BR2

Figure 11. Contribution and error bars (one standard deviation) of each isotope in the total fission rate for a deposit of Pu242#2 (see Table I for initial composition) in a typical high flux thermal neutron environment (BR2).



PART IV: Relative error in fission rates Np237 in BR2

Figure 12. Total fission rate and error bars (one standard deviation) for initially pure Np237 deposit irradiated in a high flux thermal environment (e.g. BR2). Contributions of each isotope to the total fission rate and errors are shown.



The need of a database on Radiation Characterization Benchmark Experiments:

□ Decay Heat and Inventory Predictions

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Procedure:

- Exp. Data:
FPDH, DH experiments
- PREPRO/NJOY codes
- Activation codes: FISPACT, ACAB, ...
- Input data:
Initial inventory and irradiation history
- Calculate: $DH + \Delta DH$



The database on **Radiation Characterization Benchmark Experiments**:

- Quantities not yet included in other Handbooks for “**Radiation Characterization**”
- Pointing out **gaps in experiments/databases** that need to be filled for ND validation
 - ❑ IRCBEP benchmarks cover an area not at all covered by ICSBEP, IRPhE or SINBAD because in Radiation Characterization what matter is the “radiation” itself, i.e. the value of the cross sections which allow the description of the radiation
 - ❑ How to join efforts: e.g. decay heat for fission events and fusion .vs. SFCOMPO
 - ❑ Example of duplicate information:
 - *WP2016-24. Assess the difference of RIKEN neutron spectra compiled in EXFOR E2298 and SINBAD NEA-1552/14 summarized in WP2015-17. N. Otsuka*
- **Automatize the process of validation**
 - ❑ Repository of benchmarks, results... “reproducibility, traceability and transparency”
 - ❑ Tools ... scripts, updated codes, standard outputs... helping to quickly assess new results
 - ❑ Procedures ... guidelines to judge validation,...
- **Improve cross-communication** between experts on ND evaluation & experiments