

Nuclear Constants for NAA and their Relation to Differential Cross Sections

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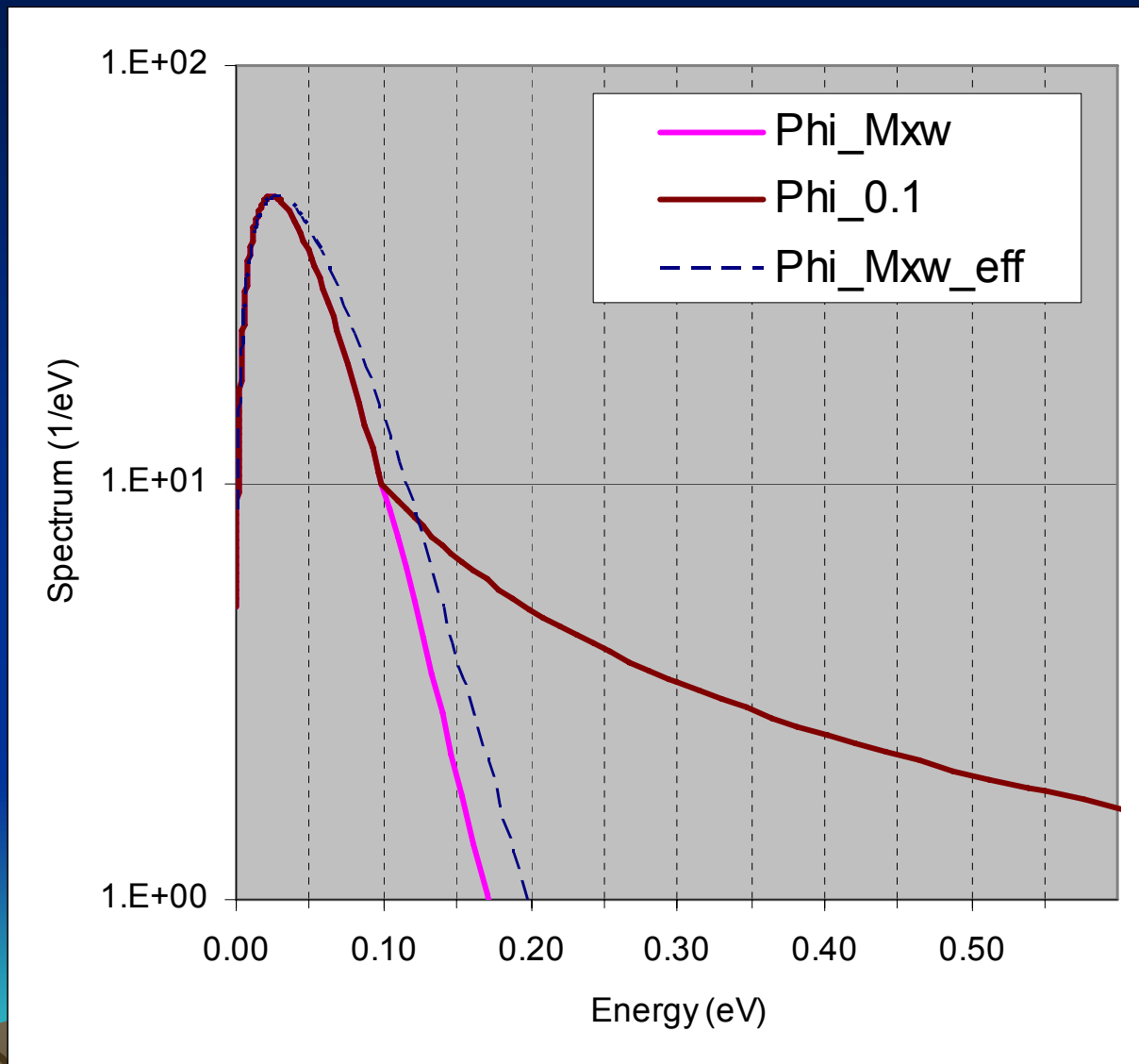
Scope

- Definitions of constants
- Neutron self-shielding
- Spectrum characterisation
- Nuclear data updating strategy

Definitions of Constants

- Paper in preparation (still!)
- Influence of $1/E$ spectrum on thermal constants
- Cadmium method to determine resonance integrals

The 1/E spectrum contribution



The 1/E spectrum contribution

$\Phi = v_T \int n(v) dv$; consider $E < 0.55$ eV

For $E_{th} = 0.1$, $v_T/v_0 = 1.148$

→ Effective spectrum temperature changes significantly (depends on E_{th})

Why does k_0 method still work?

- The difference is transferred to an effective f-factor, which is measured

$$\varphi_f \sigma_0 [f g G_t + (Q G_f + H h)]$$

The 1/E spectrum contribution

$$A = \varphi_f \sigma_0 [f g G_t + (Q G_f + H h)]$$

Generalised g-factor

$$g = \frac{\int_0^{E_{cd}} \sigma(E) \varphi(E) dE}{\sigma_0 \frac{\sqrt{\pi}}{2} \int_0^{E_{cd}} \varphi(E) dE} = \frac{2}{\sqrt{\pi}} \frac{\sigma_{th}}{\sigma_0}$$

Cadmium method for resonance integrals

Instead of applying the cadmium ratio method, measure the ratio of sub-cadmium reaction rates relative to a standard (irradiated simultaneously)

Advantage: no reliance on “same irradiation conditions”, spectral perturbations, etc.

Disadvantage: introduction of dependence on gamma emission probabilities

Simultaneous use of both methods may be used as a cross-check of consistency.

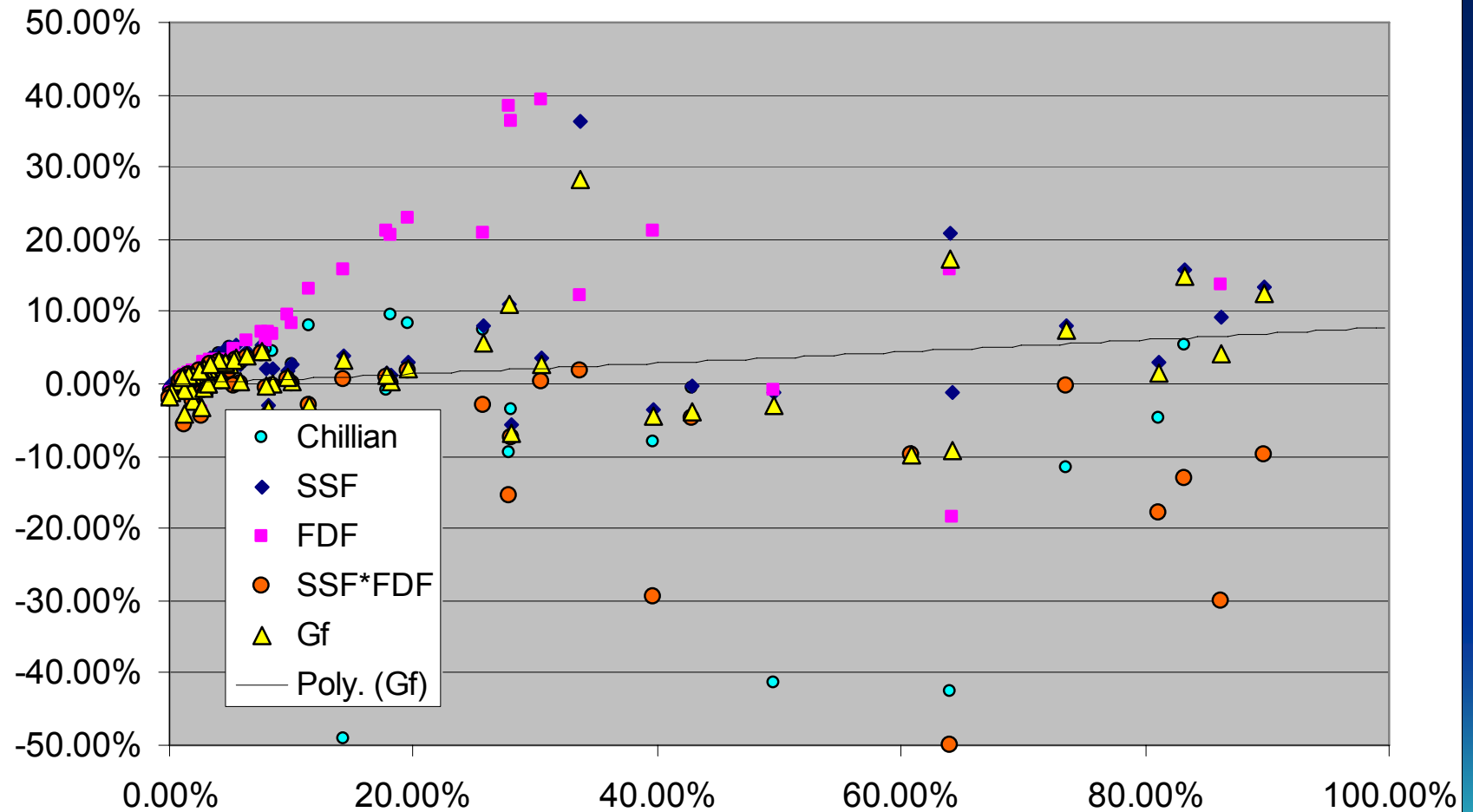
Neutron self-shielding factors

- Paper published by Chillian et al
- MATSSF – interpolation of rigorously-calculated epithermal self-shielding factors
- Monte Carlo simulation → flux depression is important (alloys)

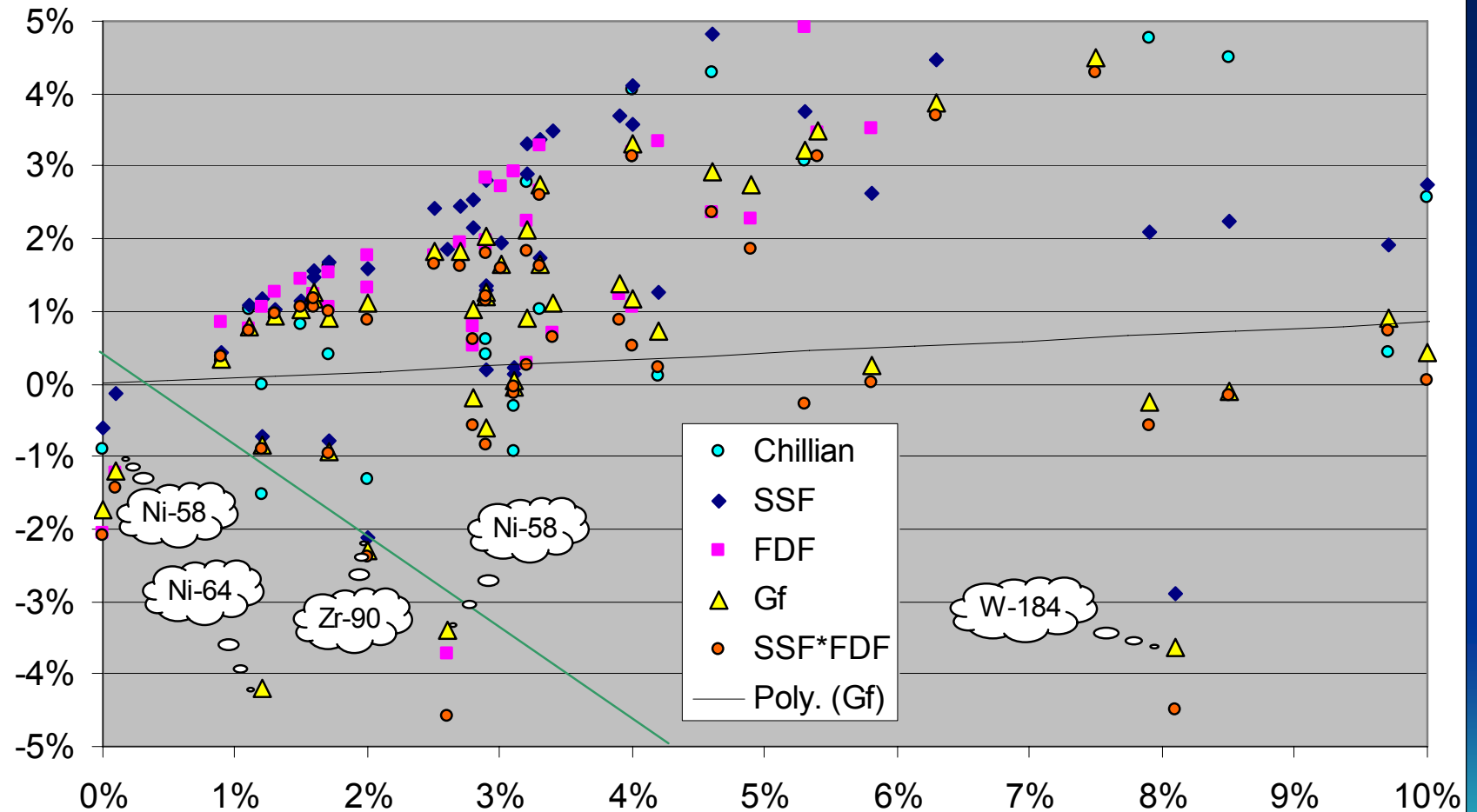
Neutron self-shielding factors

- For single isotopes the MATSSF method works well, compared to MC simulation
- Considering the crudeness and simplicity, the method of Chillian is not so bad.
- Unfortunately there is more to it: multi-isotope matrices give rise to flux depression, which influences self-shielding significantly

dGf as a function of 1-G(MCNP)



dGf as a function of 1-G(MCNP)



Spectrum Characterisation

- TRIGA IJS
- Nigerian reactor
- Collaboration with CEA

Main difficulty: reliable differential cross sections of monitor reactions

Nuclear Data Updating Strategy

Suggestion (ref. M. Blaauw):

- Do not update by data type but by nuclide

Review all data for one nuclide rather than one data type for all nuclides to avoid introducing inconsistencies

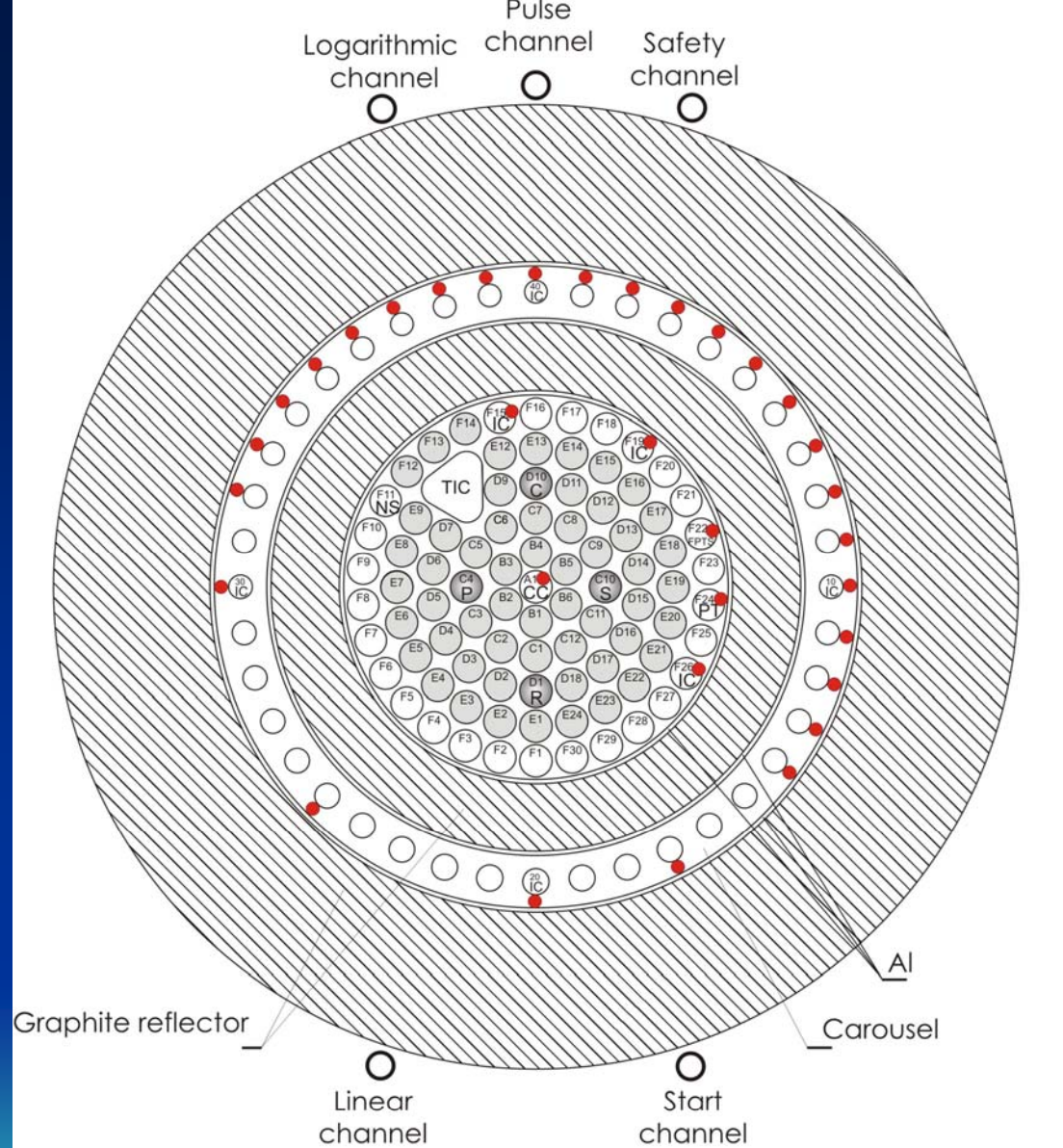
CRP Outputs

Contributions:

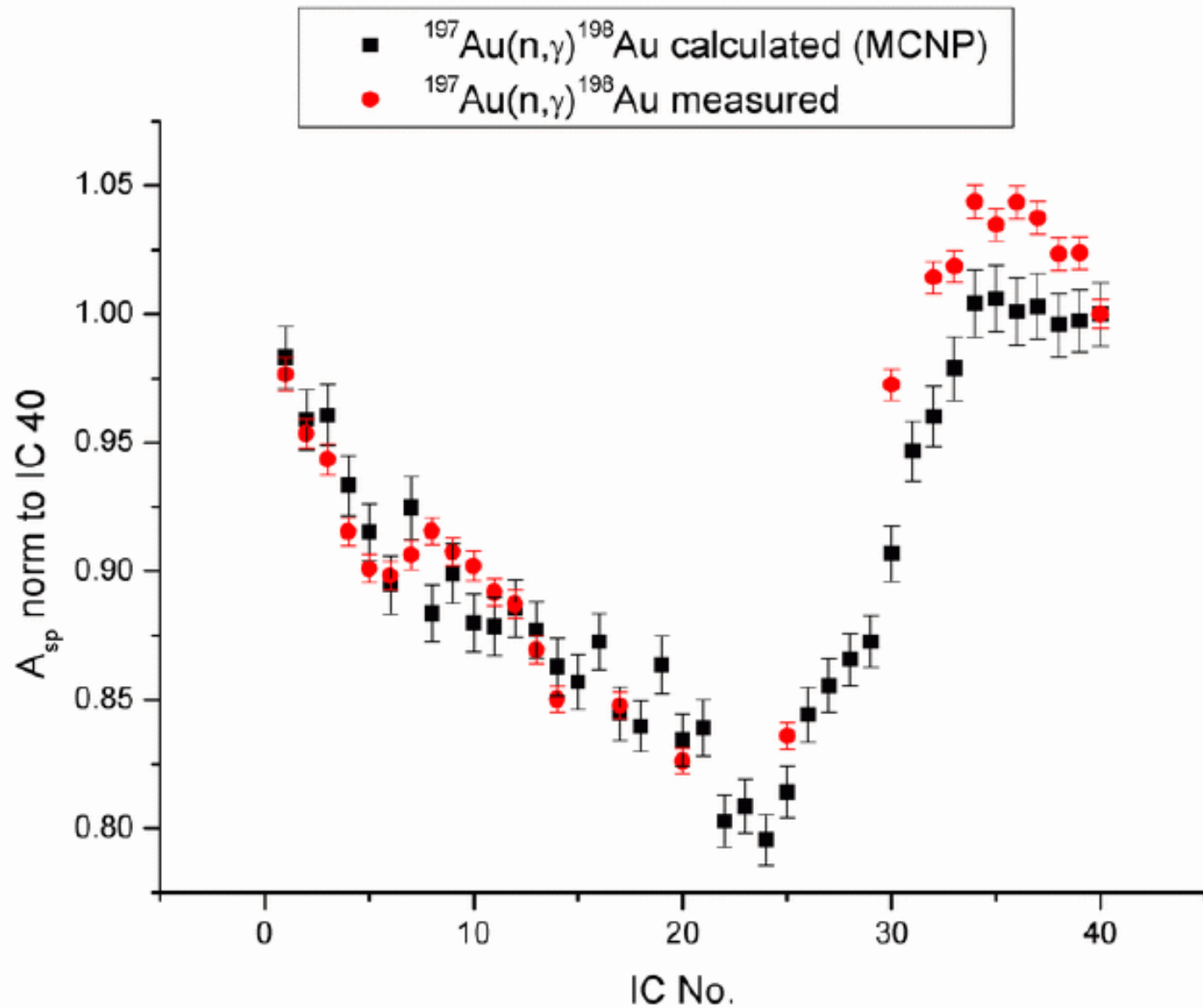
- MATSSF for self-shielding calculations
- GRUPINT for spectrum analysis
- Starter “activation” library
- Document with rigorous and self-consistent definitions of constants

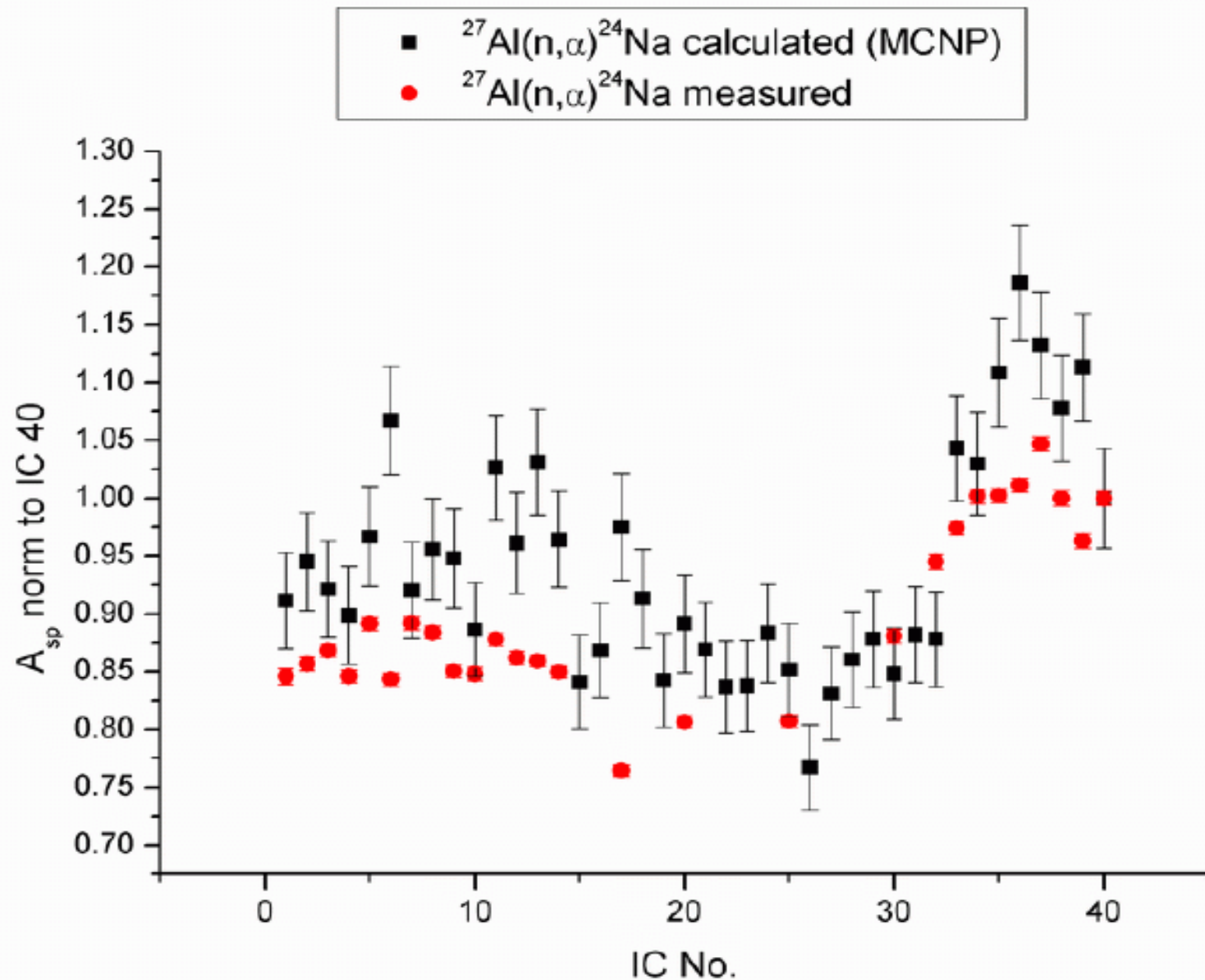
Neutron spectrum

- Detailed Monte Carlo model developed.
- Al-Au(0.1%) foils irradiated in 33 channels (6 in the core, 27 in the reflector)
- $^{197}\text{Au}(n,\gamma)^{197}\text{Au}$ (thermal flux monitor) and $^{27}\text{Al}(n,\alpha)^{23}\text{Na}$ (fast flux monitor) activities measured.
- Calculated and measured values compared.



- | | | | |
|---|-------------------------------------|---|-------------------------------------|
|  | Fuel elements 20 % ^{235}U |  | FPTS Fast pneumatic transfer system |
|  | Control rods |  | PT Pneumatic transport tube channel |
|  | NS Neutron source |  | CC Central irradiation channel |
|  | IC Irradiation channels |  | TIC Triangular irradiation channel |





Neutron spectrum (cont.)

- Computational model validated by flux distribution measurements.
- Spectrum parameterised by analytic function, modulated by the fine structure of the calculated spectrum.
- Multi-monitor irradiation in 4 selected channels.
- Parameters adjusted to match measured activities.

Neutron spectrum (cont.)

$$\psi_t = C_t E \left[e^{-E/kT} + C_{t1} e^{-E/kT_1} + C_{t2} e^{-E/kT_2} \right]$$

$$\psi_e = E^{-[1+\alpha_0+\alpha_1 \log E + \alpha_2 (\log E)^2]}$$

$$\psi_f = C_f e^{-E/W} \sinh(\sqrt{EW_b}) \cdot \frac{1}{E^m} \quad \text{or}$$

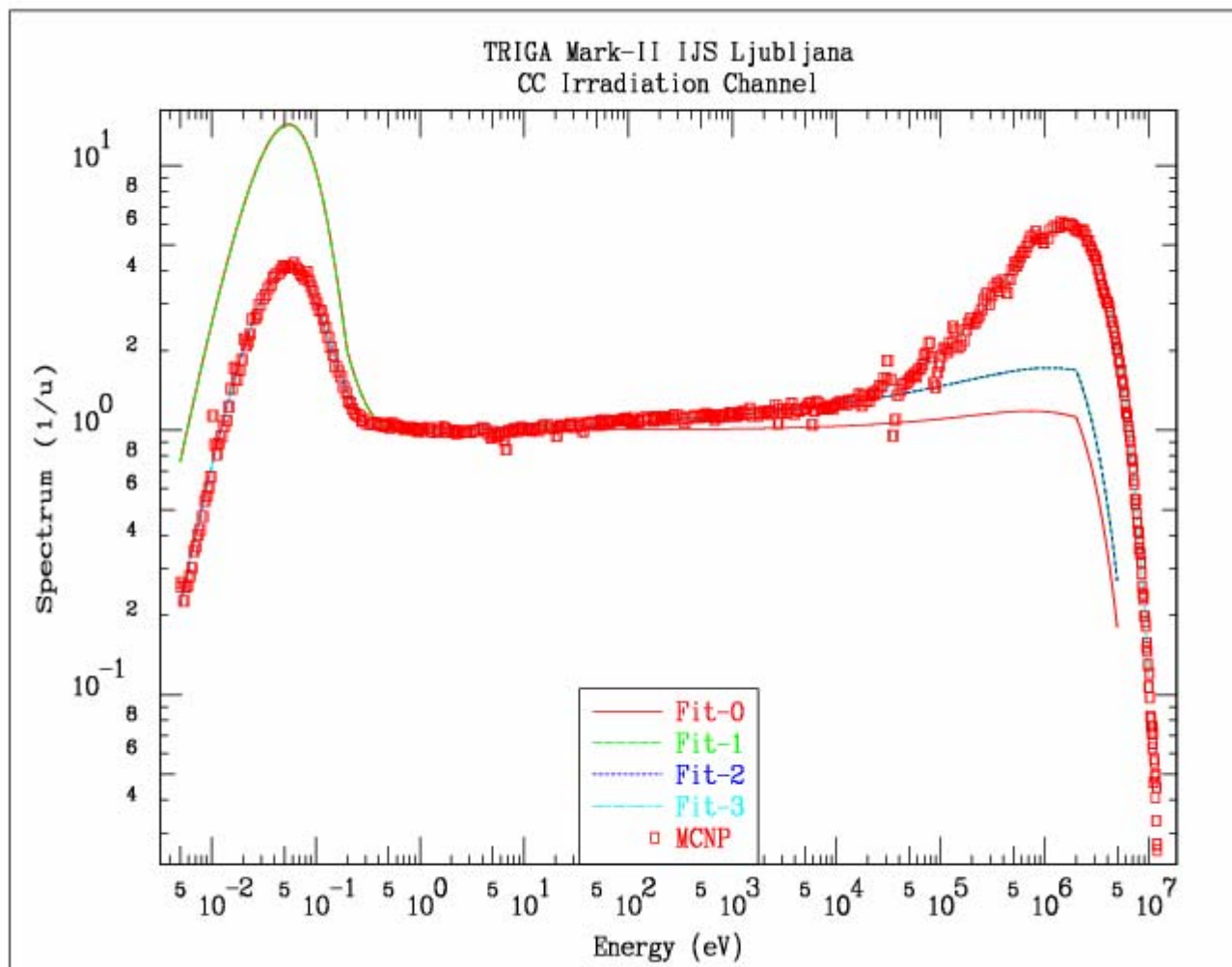
$$C_f \sqrt{E} e^{-E/E_T} \cdot \frac{1}{E^{m_0+m_1 E}}$$

$$\psi = K_t \psi_t + K_e \psi_e + K_f \psi_f$$

$$K_e = \begin{cases} 1 & \text{for } E_t < E < E_f \\ 0 & \text{otherwise} \end{cases}$$

$$K_t = 1 + O_t - K_e$$

$$K_f = 1 + O_f - K_e$$



Neutron spectrum (cont.)

Results:

- Selected reaction rate ratios were considered.
- Some reactions excluded – cross sections not reliable enough.

Basis:

- Use computed spectrum on fine group structure.
- Calculate reaction rates by multiplying flux with cross-sections from IRDF-2002.
- Consider epithermal self-shielding.

CC

Target	Product	Ratio	Target	Product	Ratio	Dif. [%]	Unc. [%]
79-Au-197g	79-Au-198	Tot/Tot	13-Al- 27g	13-Al- 28	947.79	-15.5*(	3.4)
13-Al- 27g	12-Mg- 27	Tot/Tot	13-Al- 27g	13-Al- 28	1.629E-2	-13.8*(	2.9)
13-Al- 27g	11-Na- 24	Tot/Tot	13-Al- 27g	13-Al- 28	3.175E-3	-12.5*(	2.9)
13-Al- 27g	12-Mg- 27	Tot/Tot	79-Au-197g	79-Au-198	1.718E-5	1.9 (	3.1)
13-Al- 27g	11-Na- 24	Tot/Tot	79-Au-197g	79-Au-198	3.350E-6	3.5 (	3.2)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 94g	40-Zr- 95	2780	-2.5 (	3.2)
40-Zr- 96g	40-Zr- 97	Tot/Tot	40-Zr- 94g	40-Zr- 95	6.3610	-0.3 (	2.5)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 94g	40-Zr- 95	1.520E-3	-0.2 (	3.2)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 96g	40-Zr- 97	437.03	-2.2 (	3.2)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 96g	40-Zr- 97	2.390E-4	0.1 (	3.0)
30-Zn- 64g	30-Zn- 65	Tot/Tot	30-Zn- 68g	30-Zn- 69m	9.6714	2.2 (	2.2)
30-Zn- 70g	30-Zn- 71m	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.1022	-52.8*(	1.3)
30-Zn- 64g	29-Cu- 64	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.3832	-16.4*(	1.0)
28-Ni- 64g	28-Ni- 65	Tot/Tot	79-Au-197g	79-Au-198	7.114E-3	16.5*(	1.2)
25-Mn- 55g	25-Mn- 56	Tot/Tot	79-Au-197g	79-Au-198	6.471E-2	22.8*(	0.2)
42-Mo- 98g	42-Mo- 99	Tot/Tot	79-Au-197g	79-Au-198	3.785E-3	2.1*(	2.2)
42-Mo-100g	42-Mo-101	Tot/Tot	79-Au-197g	79-Au-198	2.564E-3	-14.0*(	2.5)
74-W -186g	74-W -187	Tot/Tot	79-Au-197g	79-Au-198	0.3288	-11.1*(	2.3)
28-Ni- 58g	27-Co- 58	Tot/Tot	79-Au-197g	79-Au-198	4.598E-4	-1.1 (	2.2)
42-Mo- 92g	41-Nb- 92	Tot/Tot	79-Au-197g	79-Au-198	4.694E-5	-4.9*(	2.5)
28-Ni- 58g	27-Co- 58	Tot/Tot	28-Ni- 64g	28-Ni- 65	6.464E-2	-15.1*(	1.4)
42-Mo- 92g	41-Nb- 92	Tot/Tot	42-Mo- 98g	42-Mo- 99	1.240E-2	-6.8*(	4.3)

FPTS

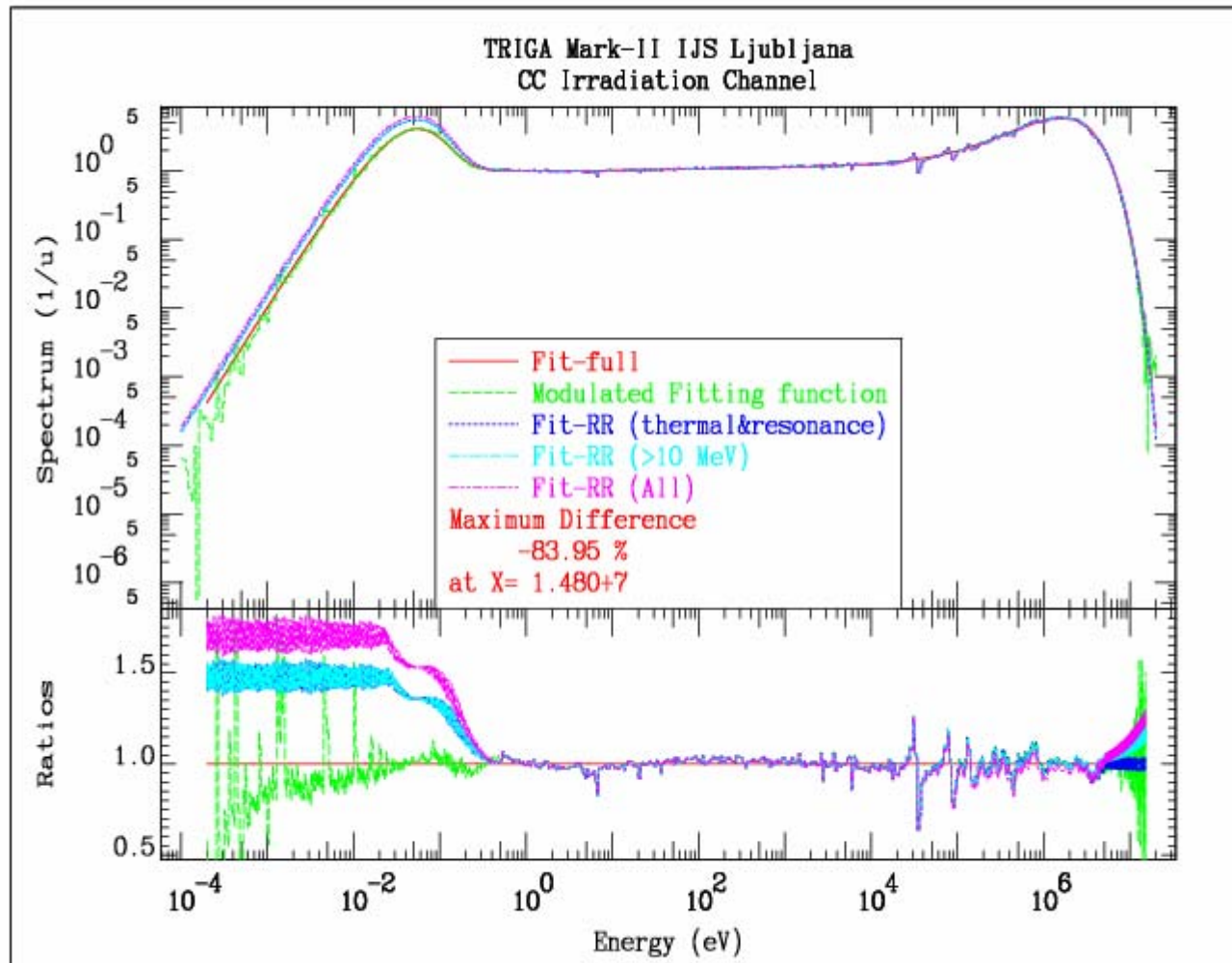
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79-Au-197g	79-Au-198	Tot/Tot	13-Al- 27g	13-Al- 28	680.89	-8.7*(	3.5)
13-Al- 27g	12-Mg- 27	Tot/Tot	13-Al- 27g	13-Al- 28	5.898E-3	-7.5*(	3.0)
13-Al- 27g	11-Na- 24	Tot/Tot	13-Al- 27g	13-Al- 28	1.188E-3	-9.6*(	3.1)
13-Al- 27g	12-Mg- 27	Tot/Tot	79-Au-197g	79-Au-198	8.662E-6	1.2 (	3.3)
13-Al- 27g	11-Na- 24	Tot/Tot	79-Au-197g	79-Au-198	1.745E-6	-1.1 (	3.3)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 94g	40-Zr- 95	2600	3.1 (	3.3)
40-Zr- 96g	40-Zr- 97	Tot/Tot	40-Zr- 94g	40-Zr- 95	3.6928	0.8 (	2.6)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 94g	40-Zr- 95	8.125E-4	0.6 (	5.7)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 96g	40-Zr- 97	704.09	2.3 (	3.1)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 96g	40-Zr- 97	2.200E-4	-0.2 (	5.6)
30-Zn- 64g	30-Zn- 65	Tot/Tot	30-Zn- 68g	30-Zn- 69m	10.13	2.3 (	2.2)
30-Zn- 70g	30-Zn- 71m	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.1086	-17.8*(	2.3)
30-Zn- 64g	29-Cu- 64	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.1542	-13.6*(	1.7)
28-Ni- 64g	28-Ni- 65	Tot/Tot	79-Au-197g	79-Au-198	9.856E-3	5.4*(	1.5)
25-Mn- 55g	25-Mn- 56	Tot/Tot	79-Au-197g	79-Au-198	8.818E-2	15.8*(	0.3)
42-Mo- 98g	42-Mo- 99	Tot/Tot	79-Au-197g	79-Au-198	2.710E-3	6.0*(	2.7)
42-Mo-100g	42-Mo-101	Tot/Tot	79-Au-197g	79-Au-198	2.187E-3	5.1*(	5.8)
74-W -186g	74-W -187	Tot/Tot	79-Au-197g	79-Au-198	0.3415	-12.7*(	3.8)
28-Ni- 58g	27-Co- 58	Tot/Tot	79-Au-197g	79-Au-198	2.263E-4	0.2 (	2.5)
42-Mo- 92g	41-Nb- 92	Tot/Tot	79-Au-197g	79-Au-198	2.320E-5	49.8*(	9.1)
28-Ni- 58g	27-Co- 58	Tot/Tot	28-Ni- 64g	28-Ni- 65	2.296E-2	-5.0*(	1.4)
42-Mo- 92g	41-Nb- 92	Tot/Tot	42-Mo- 98g	42-Mo- 99	8.561E-3	41.3*(	9.1)

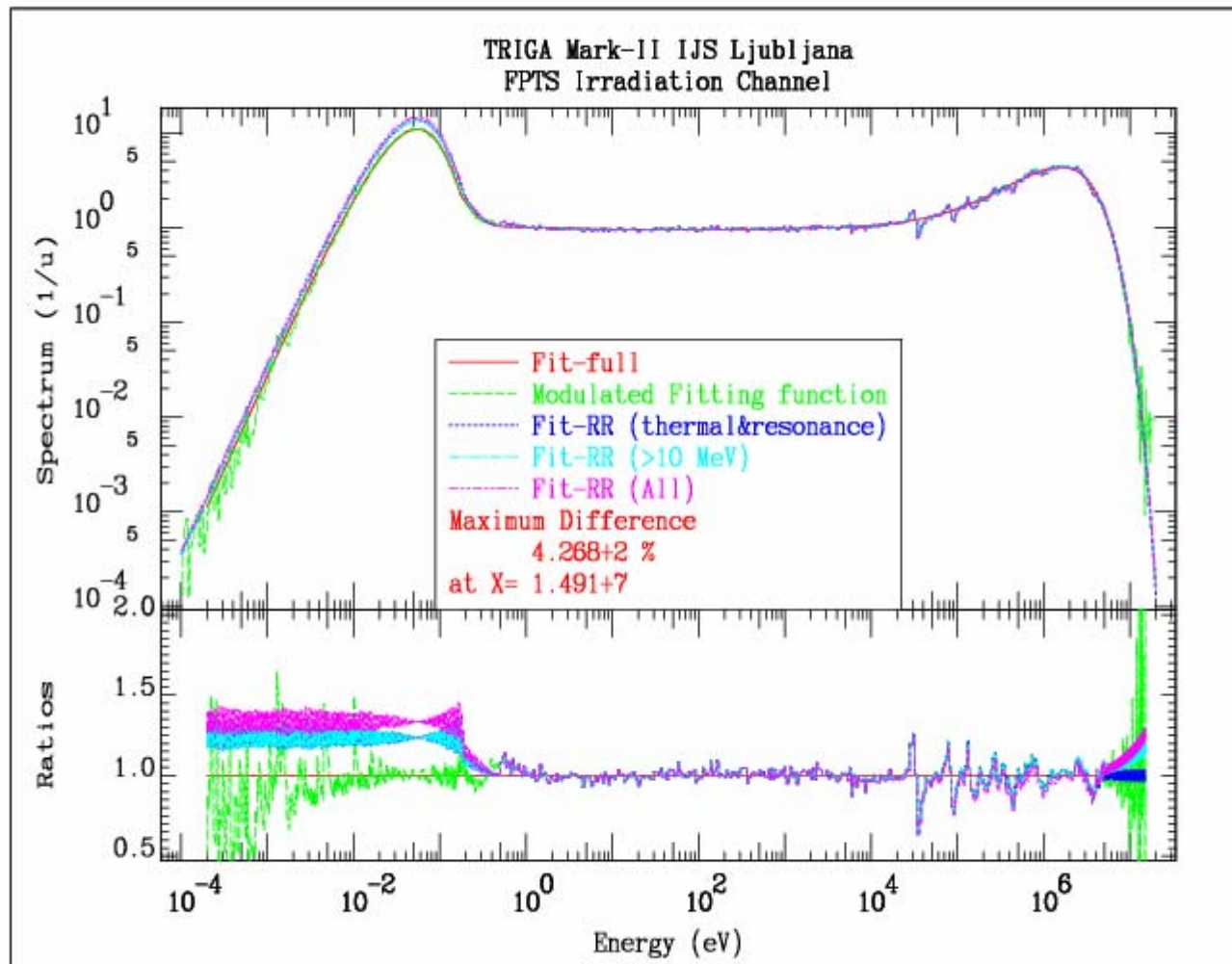
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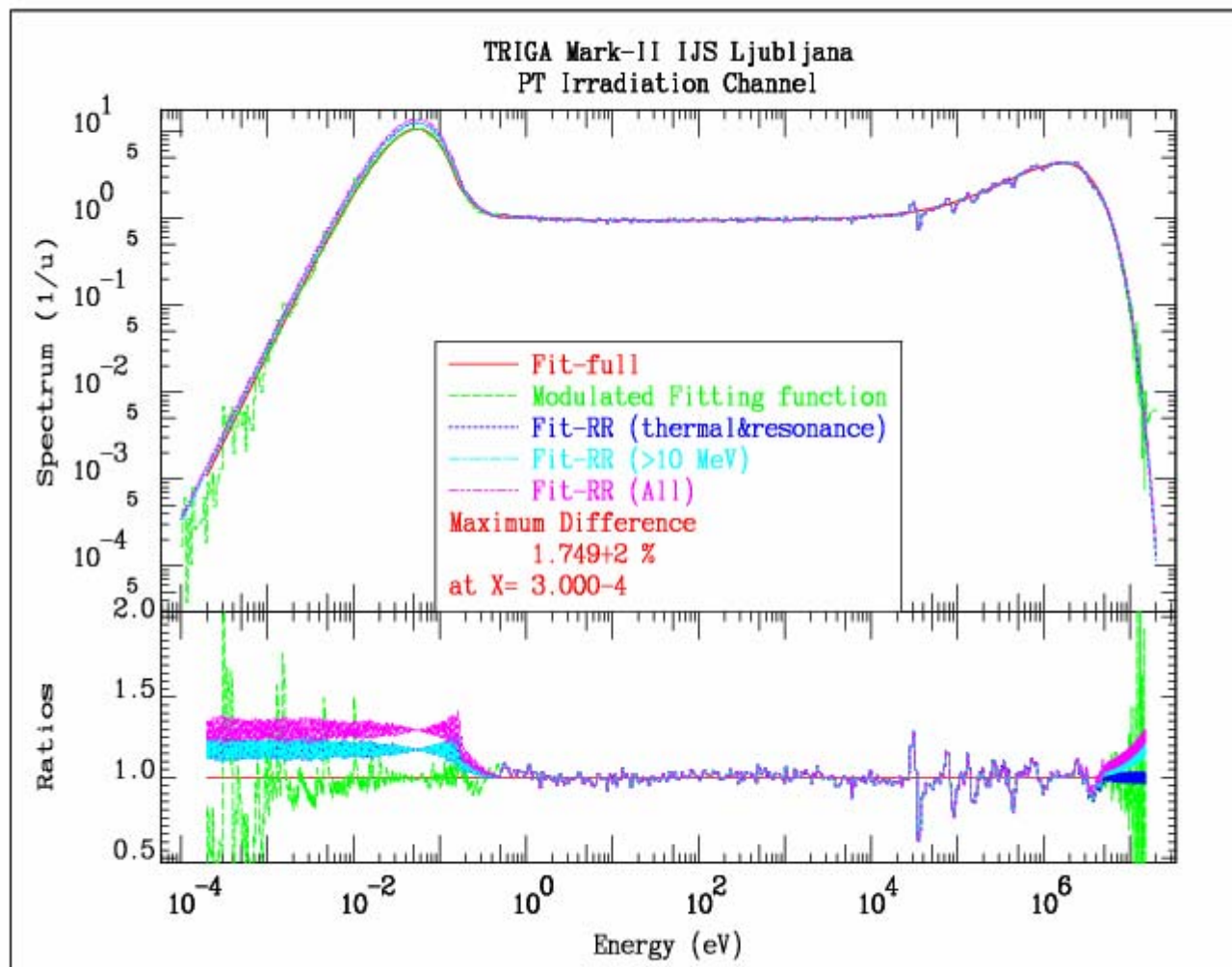
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79-Au-197g	79-Au-198	Tot/Tot	13-Al- 27g	13-Al- 28	695.88	-6.2*(	3.5)
13-Al- 27g	12-Mg- 27	Tot/Tot	13-Al- 27g	13-Al- 28	6.356E-3	-6.2*(	3.0)
13-Al- 27g	11-Na- 24	Tot/Tot	13-Al- 27g	13-Al- 28	1.280E-3	-4.9*(	3.1)
13-Al- 27g	12-Mg- 27	Tot/Tot	79-Au-197g	79-Au-198	9.133E-6	0.1 (	3.3)
13-Al- 27g	11-Na- 24	Tot/Tot	79-Au-197g	79-Au-198	1.840E-6	1.3 (	3.3)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 94g	40-Zr- 95	2623	4.6 (	3.3)
40-Zr- 96g	40-Zr- 97	Tot/Tot	40-Zr- 94g	40-Zr- 95	3.7522	0.8 (	2.6)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 94g	40-Zr- 95	8.558E-4	-0.3 (	5.7)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 96g	40-Zr- 97	699.18	3.8 (	3.1)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 96g	40-Zr- 97	2.281E-4	-1.1 (	5.6)
30-Zn- 64g	30-Zn- 65	Tot/Tot	30-Zn- 68g	30-Zn- 69m	10.08	0.5 (	2.2)
30-Zn- 70g	30-Zn- 71m	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.1081	-14.5*(	2.3)
30-Zn- 64g	29-Cu- 64	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.1651	-8.7*(	1.7)
28-Ni- 64g	28-Ni- 65	Tot/Tot	79-Au-197g	79-Au-198	9.650E-3	1.7*(	1.5)
25-Mn- 55g	25-Mn- 56	Tot/Tot	79-Au-197g	79-Au-198	8.636E-2	9.8*(	0.2)
42-Mo- 98g	42-Mo- 99	Tot/Tot	79-Au-197g	79-Au-198	2.718E-3	3.4*(	2.6)
42-Mo-100g	42-Mo-101	Tot/Tot	79-Au-197g	79-Au-198	2.228E-3	3.8*(	3.8)
74-W -186g	74-W -187	Tot/Tot	79-Au-197g	79-Au-198	0.3402	-12.2*(	3.7)
28-Ni- 58g	27-Co- 58	Tot/Tot	79-Au-197g	79-Au-198	2.387E-4	0.0 (	2.4)
42-Mo- 92g	41-Nb- 92	Tot/Tot	79-Au-197g	79-Au-198	2.447E-5	51.3*(	7.1)
28-Ni- 58g	27-Co- 58	Tot/Tot	28-Ni- 64g	28-Ni- 65	2.473E-2	-1.7*(	1.4)
42-Mo- 92g	41-Nb- 92	Tot/Tot	42-Mo- 98g	42-Mo- 99	9.003E-3	46.3*(	7.1)

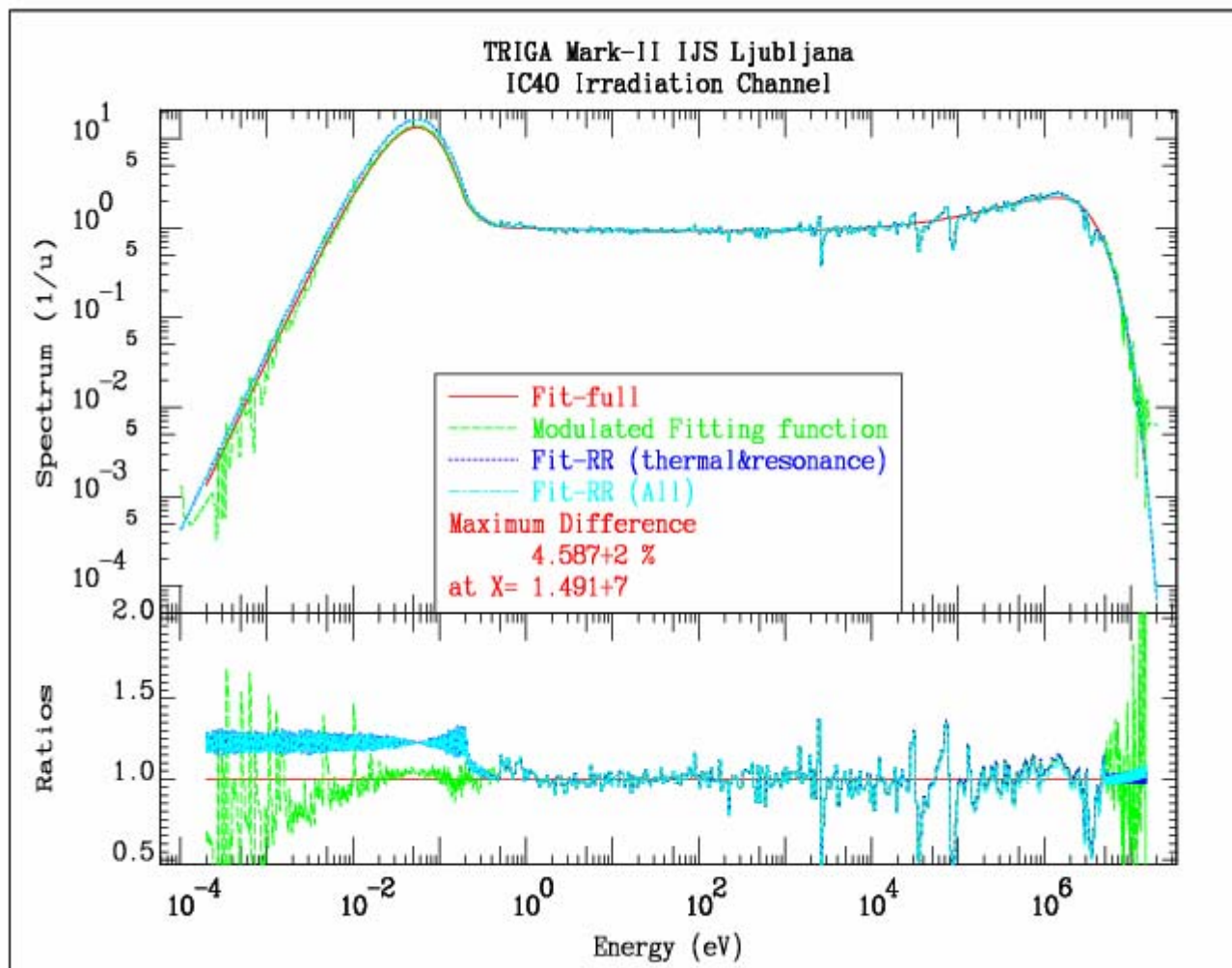
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Target	Product	Ratio	Target	Product	Ratio	Dif. [%]	Unc. [%]
79-Au-197g	79-Au-198	Tot/Tot	13-Al- 27g	13-Al- 28	652.67	-5.3*(	3.5)
13-Al- 27g	12-Mg- 27	Tot/Tot	13-Al- 27g	13-Al- 28	2.110E-3	-9.4*(	3.6)
13-Al- 27g	11-Na- 24	Tot/Tot	13-Al- 27g	13-Al- 28	4.221E-4	0.7*(	4.3)
13-Al- 27g	12-Mg- 27	Tot/Tot	79-Au-197g	79-Au-198	3.233E-6	-4.4 (	3.9)
13-Al- 27g	11-Na- 24	Tot/Tot	79-Au-197g	79-Au-198	6.468E-7	6.3 (	4.5)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 94g	40-Zr- 95	2624	4.0 (	3.5)
40-Zr- 96g	40-Zr- 97	Tot/Tot	40-Zr- 94g	40-Zr- 95	3.1672	0.9 (	2.9)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 94g	40-Zr- 95	3.090E-4	-2.0 (	11.0)
79-Au-197g	79-Au-198	Tot/Tot	40-Zr- 96g	40-Zr- 97	828.50	3.1 (	3.2)
40-Zr- 90g	40-Zr- 89	Tot/Tot	40-Zr- 96g	40-Zr- 97	9.757E-5	-2.8 (	10.0)
30-Zn- 64g	30-Zn- 65	Tot/Tot	30-Zn- 68g	30-Zn- 69m	10.23	4.3 (	2.5)
30-Zn- 70g	30-Zn- 71m	Tot/Tot	30-Zn- 68g	30-Zn- 69m	0.1106	-16.5*(	4.4)
30-Zn- 64g	29-Cu- 64	Tot/Tot	30-Zn- 68g	30-Zn- 69m	5.543E-2	-6.5*(	8.3)
28-Ni- 64g	28-Ni- 65	Tot/Tot	79-Au-197g	79-Au-198	1.027E-2	-0.7*(	1.4)
25-Mn- 55g	25-Mn- 56	Tot/Tot	79-Au-197g	79-Au-198	9.179E-2	11.1*(	0.2)
42-Mo- 98g	42-Mo- 99	Tot/Tot	79-Au-197g	79-Au-198	2.487E-3	9.0*(	3.2)
42-Mo-100g	42-Mo-101	Tot/Tot	79-Au-197g	79-Au-198	2.185E-3	-8.8*(	3.3)
74-W -186g	74-W -187	Tot/Tot	79-Au-197g	79-Au-198	0.3450	-11.1*(	3.8)
28-Ni- 58g	27-Co- 58	Tot/Tot	79-Au-197g	79-Au-198	8.439E-5	0.6 (	3.1)
42-Mo- 92g	41-Nb- 92	Tot/Tot	79-Au-197g	79-Au-198	8.665E-6	-12.0*(	100.0)
28-Ni- 58g	27-Co- 58	Tot/Tot	28-Ni- 64g	28-Ni- 65	8.217E-3	-37.5*(	1.7)
42-Mo- 92g	41-Nb- 92	Tot/Tot	42-Mo- 98g	42-Mo- 99	3.484E-3	-19.2*(	100.0)









Conclusions

Results:

- Method seems to work.
- Generally, adjusted spectrum reproduces well the measured reaction rates.
- Calculated thermal spectrum lower than measured by gold activation, particularly in the core centre.

Question:

- Do we trust the results?

Conclusions (cont.)

Weaknesses:

- Threshold cross sections abundant and well-known, resonance cross sections are not.
- Thermal spectrum in present analysis based on Au and Zr.

Further work :

- Include ^{232}Th , ^{238}U and ^{58}Fe
- Add measurements under Cd cover

Summary

- Rigorous definitions of constants for NAA from differential cross sections.
- Spectrum calculation using validated computational model.
- Parameterisation of the calculated spectrum.
- Adjustment of parameters to match measured monitor activities.
- Spectrum can be used to validate/adjust cross sections of other nuclides.