

### **International Atomic Energy Agency**

### **Progress in TOF Spectra Compilation**

#### Naohiko Otsuka

**IAEA Nuclear Data Section** 

in collaboration with

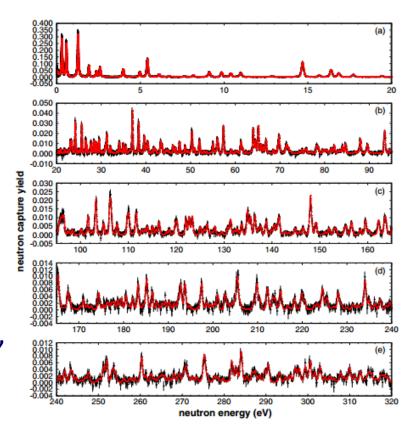
**Emmeric Dupont (CEA Saclay)** 

**Hyeong II Kim (KNDC)** 

Peter Schillebeeckx, Carlos Paradela, Stefan Kopecky (EC-JRC Geel)
Oscar Cabellos (NEA Data Bank)

## **Compilation of TOF Spectra - Background**

- Archiving of energy dependent data sets for future resonance analysis (e.g., simultaneous fitting of transmission and capture yield)
- Not tabulated in publications.
- Not for digitization (~10000 points, input for least-square analysis)
- Submission of data by authors is essential.



<sup>241</sup>Am(n,γ) capture yield-K. Fraval et al., PRC89(2014)044609

## **Consultant's Meeting (October 2013, Vienna)**



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Databases » EXFOR ENDF CINDA IBANDL Medical PGAA NGAtlas RIPL FENDL IRDFF

#### 

Yaron Danon Klaus H. Guber Frank Gunsing Atsushi Kimura Gilles Noguere Peter Schillebeeckx Gasper Zerovnik

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#### ★ Links

Nuclear Data Services Nuclear Data Section NRDC Network IAEA

#### EXFOR Data in Resonance Region and Spectrometers' Response Function

(Consultants' Meeting, 8 to 10 October 2013, IAEA Headquarters, Vienna, Austria)

#### Background

Neutron-induced reaction cross section data in the resonance region are important for many fields of science and technology. Regarding nuclear energy applications such data are needed for analysis of nuclear criticality safety, advanced fuel cycle developments, nuclear safeguards applications, nuclear waste managements, etc. There are very few experimental facilities worldwide providing data in the resonance region: GELINA (IRMM, Belgium), J-PARC (Japan), n\_TOF (CERN), ORELA (Oak Ridge, USA), RPI (NY, USA), each with its own characteristics. All facilities employ accelerators for production of neutrons in a broad energy range and data are obtained by transmission and capture measurements by time-of-flight method. Proper analysis of the data in the resonance region (total, capture, fission cross sections, and time-of-flight spectra) requires knowledge of response functions of the complete target to detector set-up

The IAEA Nuclear Data S library in collaboration with Data Centres (NRDC). Data of the EXFOR database. Ho without energy dependent of such experiments for an observables available in a fo such time-of-flight data mu recolution function) in order

#### 

Go

NIM A618 (2010) 54 NIM A555 (2005) 329 NIM A489 (2002) 346 NDS 113 (2012) 3054 JNST 7 (2012) P11002 KPS 59 (2011) 1314 NIM A736 (2014) 66 Memo CP-D/772

 Codes SAMMY

REFIT

 ★ Templates Template TOF



## **Major Facilities Providing TOF Spectra**

- Major facilities providing TOF spectra for resonance analysis:
  - n\_TOF (2ZZZCER)
  - GELINA (2ZZZGEL)
  - RPI (1USARPI)
  - ORELA (1USAORL)
  - Some other TOF facilities providing point-wise neutron-induced cross sections in the resonance regions (e.g., J-PARC, LANSCE).
- NDS and NEA DB continue compilation for GELINA and n\_TOF.
- NNDC continues compilation for RPI and ORELA.

## n\_TOF

- n\_TOF (Emmeric Dupont) monitors dissemination status since June 2015. He collects data and also review EXFOR entries.
- Now we usually receive numerical data without delay after final publication.

#### **Examples:**

<sup>238</sup>U(n,γ) C6D6 by Mingrone et al. (EXFOR 23234): Compiled in December 2016 and published in March 2017.)

<sup>25</sup>Mg(n,γ) C6D6 by Massimi et al. (EXFOR 23327): Compiled in March 2017 and published in May 2017.)

 EXFOR is complete for (n,f) and (n,cp) data sets finalized / published by n\_TOF.

## Retroactive compilation of n\_TOF (n,γ) data sets

There is also a major progress in retroactive compilation of  $(n,\gamma)$  capture yields finalized / published by n\_TOF many years ago.

#### **Examples:**

- <sup>209</sup>Bi(n,γ) C6D6 by Domingo-Pardo et al. (EXFOR 22944): Published in August 2006 and compiled in February 2017.
- <sup>207</sup>Pb(n,γ) C6D6 by Domingo-Pardo et al. (EXFOR 22946): Published in Nov.
   2006 and compiled in April 2017.
- <sup>90,94,96</sup>Zr(n,γ) C6D6 by Tagliente et al. (EXFOR 23329, 23330, 23331): Published in March 2008 and July+Nov. 2011, and compiled in March 2017.

## Reprocessing of Raw Data for EXFOR Compilation

From: emmeric.dupont@cea.fr [mailto:emmeric.dupont@cea.fr]

Sent: Thursday, 16 March 2017 11:43

To: OTSUKA, Naohiko < N.Otsuka@iaea.org>; Oscar.Cabellos@oecd.org

Cc: Giuseppe Tagliente < Giuseppe. Tagliente @ba.infn.it>

Subject: n\_TOF C6D6 capture data for Zr-90,94,96

Dear Naohiko and Oscar,

Thanks to Pino (in Cc) I'm pleased to forward you the yields of the capture reaction on Zr-90,94,96 isotopes measured during n\_TOF Phase-I. Well, I don't want to make a long story too short, but you have to know at least that the reduced data are no longer available and that Pino kindly agreed to process the yields again starting from the raw data stored at CERN!

You will find attached three files:



- Zr90\_v2\_highres\_Normalized4SAMMY\_full\_range\_4EXFOR.txt with data published in PRC 77 (2008) 035802
- Zr94\_v1\_highres\_Normalized4SAMMY\_full\_range\_4EXFOR.txt with data published in PRC 84 (2011) 015801
- Zr96\_v3\_highres\_Normalized4SAMMY\_full\_range\_4EXFOR.txt with data published in PRC 84 (2011) 055802

These files share the same characteristics:

The energy values (in eV) correspond to the middle of the hins.

## Reproduction of n\_TOF Thesis as INDC Report



INDC(SPN)-3 Distr. G

#### INDC International Nuclear Data Committee

New Radiative Neutron Capture Measurement of 207Pb and 209Bi

> Doctor Thesis submitted to Universidad de Valencia (December 2004)

> > César Domingo Pardo

Departamento de Física Atómica, Molecular y Nuclear Instituto de Física Corpuscular Universidad de Valencia

February 2017

Publication of the thesis as an INDC report.

- Addition as a secondary reference in EXFOR.
- Guaranteed access for future EXFOR users.

This is an option not only for n\_TOF experiments but for any EXFOR related theses as long as the authors permit reproduction.

(Another example: INDC(FR)-73 for EXFOR 23114)

IAEA Nuclear Data Section, Vienna International Centre, A-1400 Vienna, Austria



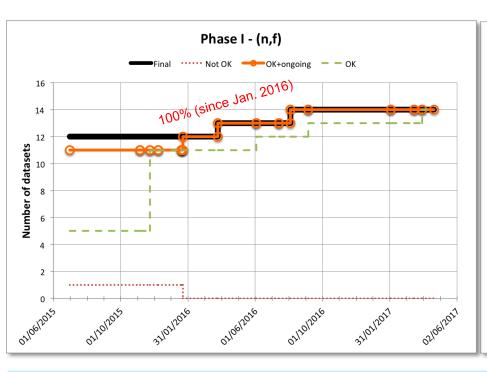
#### STATUS OF DATA DISSEMINATION

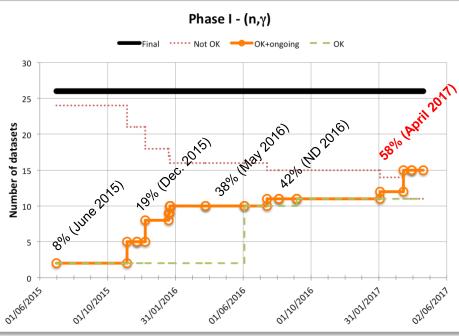


### Progress in data dissemination



n\_TOF monitoring of the dissemination status since June 2015 (https://twiki.cern.ch/NTOFPublic/DataDissemination)





## **EXFOR Coverage for n\_TOF Point-wise Data**

As of	Phase I experiments (2001-2004)		Phase II experiments (2009-2012)		
	Capture	Fission	Capture	Fission	
NRDC 2016	38%	100%	25%	100%	
NRDC 2017	<b>62</b> %	100%	71%	100%	

### **GELINA**

- Cd transmission entry (S. Kopecky+ 2009, EXFOR 23077)
  - A milestone for TOF spectrum compilation in EXFOR

- Intensive EXFOR compilation of TOF spectra available agreed with GELINA in 2015.
- Hyeong-Il Kim (KNDC) created 6 entries for <sup>238</sup>U (his own work), <sup>197</sup>Au, <sup>nat</sup>Ce, <sup>nat</sup>W, <sup>nat,63,65</sup>Cu in 2016.



## **GELINA EXFOR Entries Prepared by H.I. Kim**

Entry	Subent	Target	Quantity	primary reference	En min	En max	points
23302	1			J,EPJ/A,52,170,2016			
	2	238 <b>U</b>	average capture cross section		3.5 keV	80 keV	27
	3	238U	LSQ capture cross section		3 keV	8.5 MeV	16
	4	238U	Resonance parameter		6.674 eV	1.211 keV	108
	5	238U	kT=25.3 keV qMACS				1
	6	238U	capture yield		3.5 eV	1.2 keV	16108
	7	238 <b>U</b>	capture yield		150 eV	1.2 keV	10229
	8	238U	capture yield		3.5 eV	1.2 keV	22252
23253	1			J,EPJ/A,50,124,2014			
	2	<sup>197</sup> Au	average capture cross section		3.5 keV	84 keV	
23322	1		general description	R,EUR-28223,2016			
	2	<sup>nat</sup> Ce	transmission		300 eV	200 keV	32399
	3	<sup>nat</sup> Ce	transmission		300 eV	200 keV	32399

## **GELINA EXFOR Entries Prepared by H.I. Kim**

Entry	Subent	Target	Quantity	primary reference	En min	En max	points
23323	1		general description	J. EPJ/P,129,58,2014			
	2	nat₩	transmission		8.20 eV	2260 eV	31516
23324	1		general description	J,NIM/A,767,364,2014			
	2	<sup>nat</sup> Cu	transmission		150 eV	90 keV	37673
	3	<sup>nat</sup> Cu	transmission		150 eV	90 keV	37673
	4	<sup>nat</sup> Cu	transmission		150 eV	90 keV	37673
	5	<sup>nat</sup> Cu	transmission		150 eV	90 keV	37673
23325	1		general description	R,EUR-26479,2013			
	2	<sup>63</sup> Cu	transmission		150 eV	90 keV	21528
	3	<sup>63</sup> Cu	transmission		150 eV	90 keV	21528
	4	<sup>63</sup> Cu	transmission		150 eV	90 keV	21528
	5	<sup>65</sup> Cu	transmission		150 eV	90 keV	21528
	6	<sup>65</sup> Cu	transmission		150 eV	90 keV	21528
	7	<sup>nat</sup> Cu	transmission		150 eV	90 keV	21528

## **TOF Spectra in AGS Vectors (EXFOR 23302.006)**

```
(92-U-238 (N,G) 92-U-239, ,RYL)
REACTION
ERR-ANALYS Standard uncertainties at 1 standard deviation.
             (ERR-T) Total uncertainty
             (ERR-1,,,U) Uncorrelated uncertainty due to counting statistics
             (ERR-2,,,F) Correlated uncertainty due to background of capture, Bw (3%)
             (ERR-3,,,F) Correlated uncertainty due to background of flux, Bphi (3%)
             (ERR-4,,,F) Normalisation factor, Nc (1.5%)
...
EN
            TOF-MIN
                        TOF-MAX
                                    DATA
                                                ERR-T
                                                            ERR-1
                                                                        ERR-2
                                                                                     ERR-3
                                                                                                 ERR-4
EV
            NSEC
                        NSEC
                                    NO-DIM
                                                NO-DIM
                                                            NO-DIM
                                                                        NO-DIM
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                                                                                                 NO-DIM
3.501
            499968
                        500096
                                    .001018
                                                .007551
                                                             .0041841
                                                                        -0.006286
                                                                                     -2.718e-6
                                                                                                 1.528e-5
3.503
            499840
                        499968
                                    .001774
                                                .007583
                                                             .0042189
                                                                        -0.0063
                                                                                     -4.75e-6
                                                                                                 2.662e-5
3.505
            499712
                                                                        -0.006316
                                                                                                 6.634e-5
                        499840
                                    .004423
                                                .007591
                                                             .0042107
                                                                                     -1.193e-5
                        499712
3.506
            499584
                                    .002119
                                                .007539
                                                             .0041944
                                                                        -0.006265
                                                                                     -5.692e-6
                                                                                                 3.179e-5
3.508
            499456
                        499584
                                    .006625
                                                .007415
                                                             .0041481
                                                                        -0.006145
                                                                                     -1.777e-5
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3.51
            499328
                        499456
                                    .006378
                                                .007458
                                                             .0041562
                                                                         -0.006191
                                                                                     -1.709e-5
                                                                                                 9.567e-5
3.512
            499200
                        499328
                                    .006912
                                                .007361
                                                             .0041143
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3.514
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                        499200
                                    .003434
                                                .007437
                                                             .0041275
                                                                         -0.006186
                                                                                     -9.167e-6
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            498944
                        499072
                                                             .004135
                                                                         -0.006165
                                                                                                 9.02e-5
3.515
                                    .006013
                                                .007424
                                                                                     -1.602e-5
3.517
            498816
                        498944
                                    .004244
                                                .007539
                                                             .0041992
                                                                         -0.006261
                                                                                     -1.135e-5
                                                                                                 6.366e-5
3.519
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                        498816
                                    .004974
                                                .007509
                                                             .004181
                                                                         -0.006237
                                                                                     -1.33e-5
                                                                                                 7.46e-5
3.521
            498560
                                                                                                 1.659e-5
                        498688
                                    .001106
                                                .007418
                                                             .0041106
                                                                         -0.006175
                                                                                     -2.942e-6
3.523
            498432
                        498560
                                    -0.001017
                                                .007534
                                                             .0041811
                                                                         -0.006267
                                                                                      2.717e-6 -1.526e-5
3.524
            498304
                        498432
                                                .007499
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3.526
                        498304
                                    .002477
                                                .007422
                                                             .0041091
                                                                                     -6.593e-6
3.528
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                        498176
                                    .006016
                                                .007305
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                                                                         -0.006074
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3.53
            497920
                        498048
                                    .005343
                                                .007406
                                                             .0040963
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                                                                                     -1.433e-5
                                                                                                 8.015e-5
                                                                                                -2.25e-6
3.532
            497792
                        497920
                                    -0.00015
                                                .007461
                                                             .0041365
                                                                         -0.00621
                                                                                      4.01e-7
3.534
            497664
                        497792
                                    -0.001106
                                                .007495
                                                             .0041573
                                                                         -0.006236
                                                                                      2.969e-6
                                                                                               -1.659e-5
3.535
            497536
                                                .007562
                                                             .0042205
                                                                         -0.006275
                                                                                     -9.674e-6
                                                                                                 5.365e-5
                        497664
                                    .003577
3.537
            497408
                        497536
                                    .001442
                                                .007606
                                                             .004196
                                                                         -0.006344
                                                                                     -3.914e-6
                                                                                                 2.163e-5
```

## **Detailed Documentation (K. Guber+, EUR-28223)**



JRC TECHNICAL REPORTS

# Results of time-of-flight transmission measurements for <sup>nat</sup>Ce samples at GELINA

Description of GELINA data to be stored in the EXFOR data base

Klaus Guber Gery Alaerts Jan Heyse Stefan Kopecky Carlos Paradela Peter Schillebeecko Ruud Wynants

2016



#### A. SUMMARY OF EXPERIMENTAL DETAILS

1.	Main Reference		[1,	
2.	Facility	GELINA	[3]	
3.	Neutron production		$\top$	
	Neutron production beam	Electron	1	
	Nominal average beam energy	100 MeV	1	
	Nominal average peak current	55 μA	1	
	Repetition rate (pulses per second)	800 Hz	1	
	Pulse width	1 ns		
	Primary neutron production target	Mercury cooled depleted uranium	ı	
	Target nominal neutron production intensity	3.4 x10 <sup>13</sup> s <sup>-1</sup>	ı	
4.	Moderator		-	
	Primary neutron source position in moderator	Above and below uranium target	ı	
	Moderator material	2 water filled Be-containers around U-	ı	
		target	1	
	Moderator dimensions (internal)	2 x (14.6 cm x 21 cm x 3.9 cm)	ı	
	Density (moderator material)	1 g/cm <sup>3</sup>	1	
	Temperature (K)	Room temperature	ı	
	Moderator-room decoupler (Cd, B,)	None	ı	
5.	Other experimental details		Т	
	Measurement type	Transmission	ı	
	Method (total energy, total absorption,)	Good transmission geometry	[4]	
	Flight Path length (m)	L = 47.669(4) m	Ι-	
	(centre moderator - detector front face)		ı	
	Flight path direction	9° with respect to normal of the	ı	
	Neutron beam dimensions at sample position	moderator face viewing the flight path 45 mm in diameter	ı	
	Neutron beam profile	-	ı	
	Overlap suppression	18 overlap filter (8x10 <sup>-3</sup> at/b)	ı	
	Other fixed beam filters	Co. W. Pb (16 mm)	ı	
_	Detector	CO, 11, 10 (10 mm)	₩	
٥.		Calabillator (NEO12)	ı	
	Type Material	Scintillator (NE912)	ı	
	1 14 14 14 14 14 14 14 14 14 14 14 14 14	Li-glass	1	
	Surface Dimensions	151.6 mm diameter	1	
	Thickness (cm)	6.35 mm		
_	Detector(s) position relative to neutron beam	In the beam	₩	
7.	Sample			
	Type (metal, powder, liquid, crystal)	Metal	1	
	Chemical composition	natCe (88.450% 140Ce, 11.114 % 142Ce; 0.251 % 138Ce; 0.185 136Ce)	1	
	Sample composition (at/b)	0.251 % <sup>1</sup> Ce; 0.185 <sup>1</sup> Ce) <sup>nat</sup> Ce (5.534 ± 0.025) 10 <sup>-3</sup> at/b and	1	
	(440)	natCe (28.713 ± 0.025) 10 <sup>-3</sup> at/b		
			1	
	Temperature	22°C	1	
	Sample mass (g)	34.975 g and 187.954 g	1	
	Geometrical shape (cylinder, sphere,)	Cylinder		

### **Summary**

### n\_TOF

- Newly finalized and published data are received without delay.
- Significant progress in retroactive compilation of capture yields.

#### **GELINA**

 Intensive compilation created 6 new entries done in 2016. (They have been already in EXFOR Master.)