

Measurement Uncertainty Templates and WPEC SG50

Amanda Lewis¹, Denise Neudecker², Arjan Koning³ Technical Meeting on the International Network of Nuclear Reaction Data Centres May 07, 2021

- ¹ Naval Nuclear Laboratory
- ² Los Alamos National Laboratory
- ³ International Atomic Energy Agency

Templates of Expected Measurement Uncertainties and Covariances

- This effort is led by the CSEWG covariance session, but is an international collaboration of evaluators, experimentalists and EXFOR compilers
- Templates contain typical, "expected" sources of uncertainty for many neutron-induced reaction measurements
 - Where possible, there are recommended uncertainty values or ranges
 - In addition, the metadata helpful for assessing experiments is detailed
- A neutron-induced fission template is already published [1].
- Our current work is on (n,tot), (n,g), (n,xn), (n,cp), PFNS, FY and nu-bar.



The templates were constructed by both evaluators and experimentalists, and will be submitted to NDS this year

- They summarize the needs of the EXFOR users and the knowledge of those who provide the EXFOR input.
 - Evaluators can use them to check or fill in missing/underestimated uncertainties

(as a last resort)

- Experimentalists can use them to ensure they are not missing uncertainties or metadata and to provide uncertainties in a consistent format and language,
- EXFOR Compilers could use them to ask authors for needed information to make the ERR-ANALYS section more consistent and complete for evaluation purposes.

Templates of Expected Measurement Uncertainties

D. Neudecker, ** A.M. Lewis, ** E.F. Matthews, ** J. Vanhoy, ** R.C. Haight, ** A.D. Carlson, ** D.L. Smith, 6 S. Croft, ** B. Pierson, ** A. Wallner, **, ** 10 A. Al-Adili, ** 11 D.P. Barry, ** 2 L. Bernstein, **, ** 12 R.C. Block, ** 13 D. Brown, ** 14 R. Capote, ** 15 Y. Danon, ** 13 M. Devlin, ** 1 D.L. Duke, ** 1 S. Finch, ** 16, ** 17 M. Fleming, ** 18 B.L. Goldblum, ** 12 G.M. Hale, ** 18 S. Halfon, ** 19 M.W. Herman, ** 18 L. Kelly, ** 1 A. Koning, ** 15 H.Y. Lee, ** 19 P. Lisowski, ** 1 A.E. Lovell, ** 1 P. Marini, ** 20 K. Montoya, ** 1 R.O. Nelson, ** 1 G.P.A. Nobre, ** 14 G. Noguere, ** 21 S. Okumura, ** 15 N. Otuka, ** 15 M. Paris, ** 1 B. Pritychenko, ** 14 H. Sjöstrand, ** 11 L. Snyder, ** 22 V. Sobes, ** 23 A. Solders, ** 11 A. Sonzogni, ** 14 J. Taieb, ** 24 P. Talou, ** 1 I. Thompson, ** 22 F. Tovesson, ** and M.C. White ** 12 M. Solders, ** 12 M. Solders, ** 13 L. Snyder, ** 14 J. Taieb, ** 24 P. Talou, ** 1 I. Thompson, ** 22 F. Tovesson, ** and M.C. White ** 13 M. Paris, ** 14 J. Taieb, ** 24 P. Talou, ** 1 I. Thompson, ** 22 F. Tovesson, ** 6 and M.C. White ** 14 M. Spirate, ** 15 M. Paris, ** 16 M. Paris, ** 17 M. Paris, ** 17 M. Paris, ** 16 M. Paris, ** 17 M. Paris, ** 17 M. Paris, ** 18 M. Paris, ** 1

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²⁴CEA DAM Bruyères-le-Châtel, F-91297, Arpajon, France

TABLE I. Uncertainty template for total cross section measurements. The values are relative uncertainties on the parameter, and given in percent. The uncertainties for which the sensitivity must be provided for propagation are in bold. The important uncertainties that cannot be estimated as easily are counting statistics, uncertainty on neutron-energy resolution, uncertainty in the room return correction (β) and other background neutrons (γ_1 , γ_2 , ζ), the resolution function and on F_T .

Uncertainty Source	TOF	Mono-energetic
Background Constant (K)	> 3	_
In-scattering Correction (ΔT)	557	20
Target areal number density (n) , metal	0.1-1	0.1–1
Target areal number density, (n) , powder	2-5	2–5
Target areal number density (n) , liquid	0.1-1	0.1–1
Target areal number density (n) , diluted liquid	2-5	2–5
Flux normalization (N_T) with cycling	< 1	_
Flux normalization (N_T) without cycling	4	_



TABLE I. Uncertainty template for total cross section measurements. The values are relative uncertainties on the parameter, and given in percent. The uncertainties for which the sensitivity must be provided for propagation are in bold. The important uncertainties that cannot be estimated as easily are counting statistics, uncertainty on neutron-energy resolution, uncertainty in the room return correction (β) and other background neutrons (γ_1 , γ_2 , ζ), the resolution function and on F_T .

Uncertainty Source	TOF	Mono-energetic
Background Constant (K)	> 3	==
In-scattering Correction (ΔT)	55	20
Target areal number density (n) , metal	0.1-1	0.1 - 1
Target areal number density, (n) , powder	2-5	2–5
Target areal number density (n) , liquid	0.1-1	0.1–1
Target areal number density (n) , diluted liquid	2-5	2-5
Flux normalization (N_T) with cycling	< 1	_
Flux normalization (N_T) without cycling	4	_

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20200206 · · · · · · 1460
SUBENT ........14576001 ... 20191005
                        ..20200218...20200206....1460
TITLE Tantalum, titanium, and zirconium neutron total
cross-section measurements from 0.4 to 25 MeV
AUTHOR .... (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
T.H.Trumbull, Y.Danon)
ERR-ANALYS (DATA-ERR) Total uncertainty determined by
···· propagating uncertainties
   ··········measurement
     ·····-uncertainties in sample and open background
    ···· measurements
    - - - uncertainties in sample and open monitor counts
METHOD···· (TOF)
Repetition rate 400 Hz
Pulse width 9.8 ns
Flight path (99.95+-0.01) m
SAMPLE ... Natural Ta metallic sample
Purity ---->99.95.% Ta
Sample mass (3041.8+-0.1) q
        Nominal thickness (39.98+-0.01) mm
···· Temperature ····· 293 · K
```



TABLE I. Uncertainty template for total cross section measurements. The values are relative uncertainties on the parameter, and given in percent. The uncertainties for which the sensitivity must be provided for propagation are in bold. The important uncertainties that cannot be estimated as easily are counting statistics, uncertainty on neutron-energy resolution, uncertainty in the room return correction (β) and other background neutrons (γ_1 , γ_2 , the resolution function and on F_T .

¥	_	
Uncertainty Source	TOF	Mono-energetic
${\bf Background\ Constant}\ (K)$	> 3	_
In-scattering Correction (ΔT)	=	20
Target areal number density (n) , metal	0.1-1	0.1–1
Target areal number density, (n) , powder	2-5	2–5
Target areal number density (n) , liquid	0.1-1	0.1 - 1
Target areal number density (n) , diluted liquid	2-5	2–5
Flux normalization (N_T) with cycling	< 1	
Flux normalization (N_T) without cycling	4	_

```
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                                       20200206 · · · · · · 1460
SUBENT . . . . . . . . . 14576001 . . . 20191005 .
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AUTHOR .... (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
    T.H.Trumbull, Y.Danon)
ERR-ANALYS (DATA-ERR) Total uncertainty determined by
          - statistical uncertainties in sample and open
           ··· measurement
          - uncertainties in sample and open background
            measurements
          - uncertainties in sample and open monitor counts
METHOD···· (TOF)
Repetition rate 400 Hz
Pulse width 9.8 ns
Flight path (99.95+-0.01) m
SAMPLE ... Natural Ta metallic sample
Purity ---->99.95.% Ta
         Sample mass ... (3041.8+-0.1) q
         Nominal thickness (39.98+-0.01) mm
         Diameter .... (76.24+-0.01) mm
···· Temperature ····· 293 · K
```



TABLE I. Uncertainty template for total cross section measurements. The values are relative uncertainties on the parameter, and given in percent. The uncertainties for which the sensitivity must be provided for propagation are in bold. The important uncertainties that cannot be estimated as easily are counting statistics, uncertainty on neutron-energy resolution, uncertainty in the room return correction (β) and other background neutrons (γ_1 , γ_2 , the resolution function and on F_T .

- Y	_	
Uncertainty Source	TOF	Mono-energetic
Background Constant (K)	> 3	-
In-scattering Correction (ΔT)	=	20
Target areal number density (n) , metal	0.1-1	0.1-1
Target areal number density, (n) , powder	2-5	2-5
Target areal number density (n) , liquid	0.1-1	0.1–1
Target areal number density (n) , diluted liquid	2-5	2–5
Flux normalization (N_T) with cycling	< 1	_
Flux normalization (N_T) without cycling	4	_

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20200206 · · · · · · 1460
SUBENT . . . . . . . . . . . . 14576001 . . . 20191005 .
                            . . 20200218 . . . 20200206 . . . . . . . . 1460
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 cross-section measurements from 0.4 to 25 MeV
AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
    T.H.Trumbull, Y.Danon)
RR-ANALYS (DATA-ERR) Total uncertainty determined by
          - statistical uncertainties in sample and open
            · · measurement
          - uncertainties in sample and open background
     - uncertainties in sample and open monitor counts
METHOD···· (TOF)
Repetition rate 400 Hz
Pulse width 9.8 ns
Flight path (99.95+-0.01) m
SAMPLE ... Natural Ta metallic sample
Purity ---->99.95 % Ta
         Sample mass (3041.8+-0.1) q
         Nominal thickness (39.98+-0.01) mm
         Diameter (76.24+-0.01) mm
···· Temperature ····· 293 · K
```



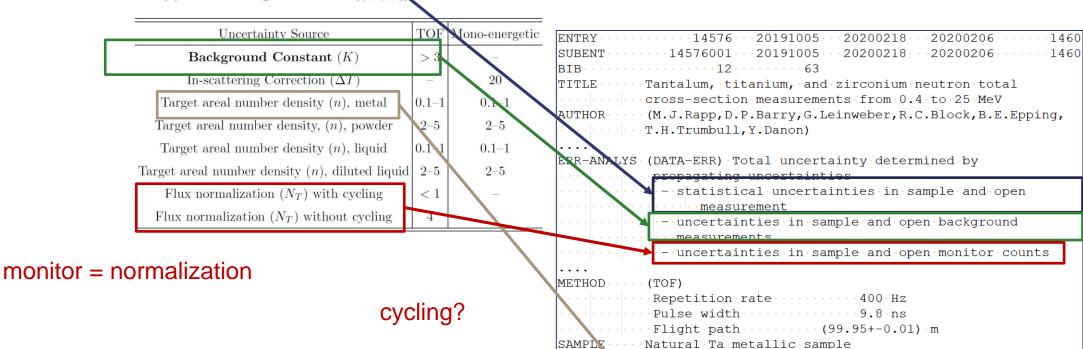
TABLE I. Uncertainty template for total cross section measurements. The values are relative uncertainties on the parameter, and given in percent. The uncertainties for which the sensitivity must be provided for propagation are in bold. The important uncertainties that cannot be estimated as easily are counting statistics, uncertainty on neutron-energy resolution, uncertainty in the room return correction (β) and other background neutrons (γ_1 , γ_2 , the resolution function and on F_T .

Uncertainty Source	TOF	Mono-energet
Background Constant (K)	> 3	-
In-scattering Correction (ΔT)	==	20
Target areal number density (n) , metal	0.1-1	0.1-1
Target areal number density, (n) , powder	2–5	2-5
Target areal number density (n) , liquid	0.1 1	0.1-1
Target areal number density (n) , diluted liquid	2-5	2–5
Flux normalization (N_T) with cycling	< 1	-
Flux normalization (N_T) without cycling	4	

```
20200206 · · · · · · 1460
     .....14576001...20191005.
                             ..20200218...20200206....1460
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cross-section measurements from 0.4 to 25 MeV
AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
    T.H.Trumbull, Y.Danon)
RR-ANALYS (DATA-ERR) Total uncertainty determined by
           - statistical uncertainties in sample and open
            ··measurement
          - uncertainties in sample and open background
           - uncertainties in sample and open monitor counts
METHOD
         (TOF)
          Repetition rate · · · · · · · 400 · Hz
         Pulse width .... 9.8 ns
         Flight path (99.95+-0.01) m
         Natural Ta metallic sample
         ·Purity·····>99.95·%·Ta
         Sample mass .... (3041.8+-0.1) q
         Nominal thickness (39.98+-0.01) mm
         Diameter (76.24+-0.01) mm
         ·Temperature·····293·K
```



TABLE I. Uncertainty template for total cross section measurements. The values are relative uncertainties on the parameter, and given in percent. The uncertainties for which the sensitivity must be provided for propagation are in bold. The important uncertainties that cannot be estimated as easily are counting statistics, uncertainty on neutron-energy resolution, uncertainty in the room return correction (β) and other background neutrons (γ_1 , γ_2 , β), the resolution function and on F_T .





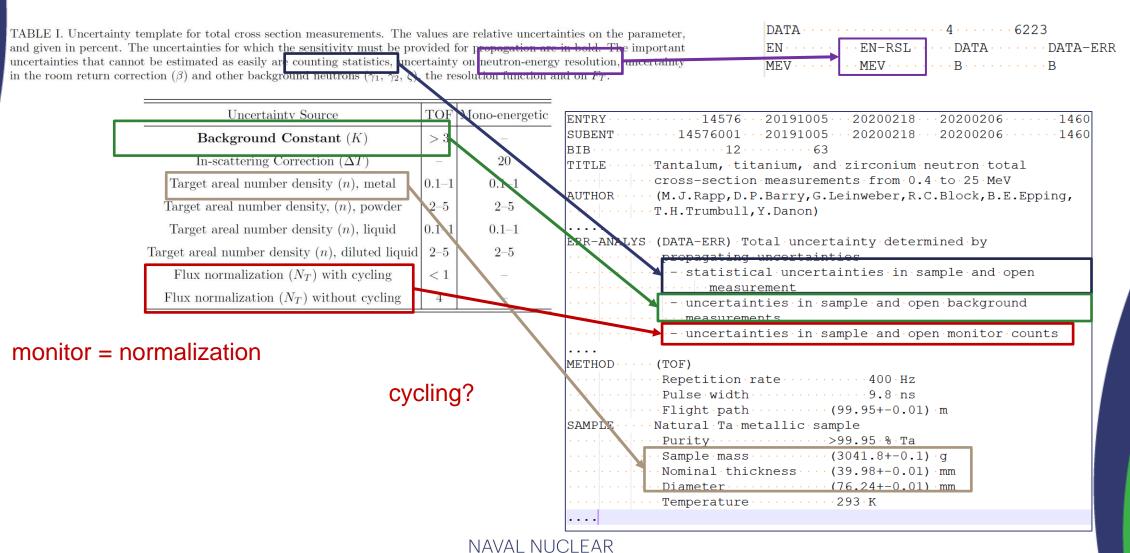
Purity > 99.95 % Ta

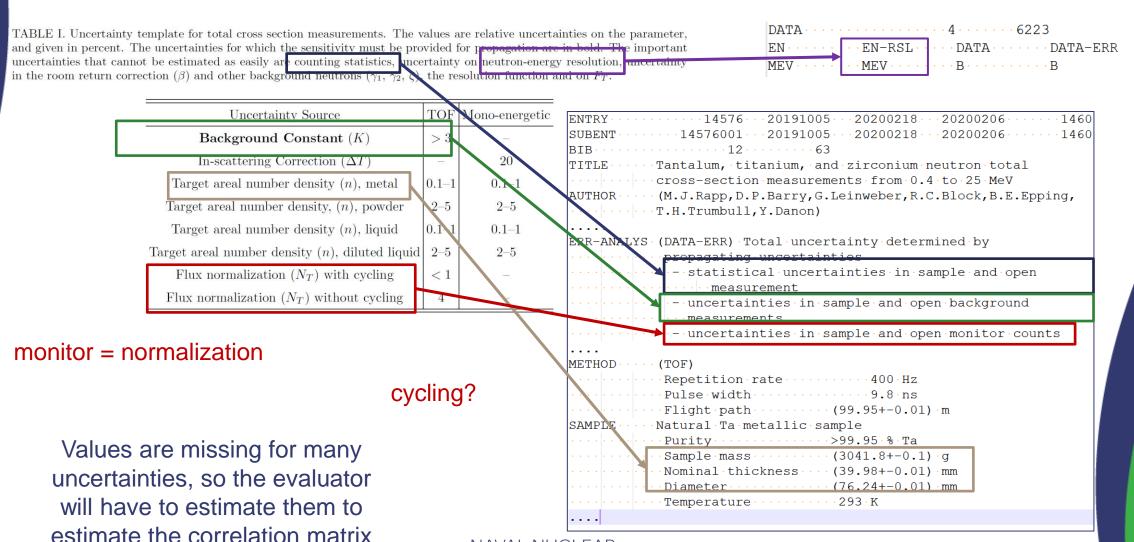
Sample mass (3041.8+-0.1) g

Nominal thickness (39.98+-0.01) mm

Diameter (76.24+-0.01) mm

·Temperature······293·K





NAVAL NUCLEAR LABORATORY

WPEC SG50: Developing an Automatically Readable, Comprehensive, and Curated Experimental Reaction Database



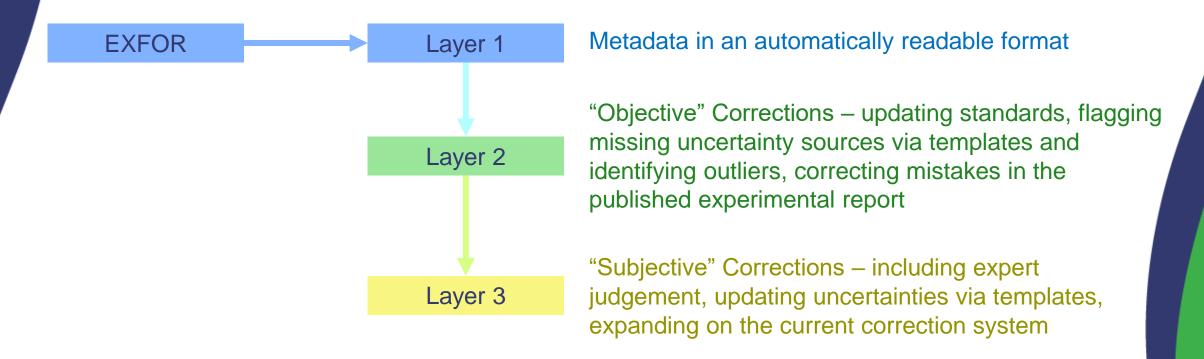
WPEC SG50: Developing an Automatically Readable, Comprehensive, and Curated Experimental Reaction Database

- Our goal is to create a new database for experimental data that will build on EXFOR and will store "subjective" corrections to the data sets made by people other than the authors.
- Approved at the May 2020 WPEC meeting
- 57 members (and counting) from 11 countries and the NEA and IAEA, representing 5 libraries
- Coordinators: A. Lewis (Naval Nuclear Laboratory), D. Neudecker (LANL)
- Monitor: A. Koning (IAEA)



The subgroup will develop a 3-layer experimental reaction database

- Create a format and structure for the database
- Produce example files for each layer and publish conversion codes





So far, we have split up into smaller groups, held 5 meetings, and have started to develop our format

- We have split up into several smaller groups to target specific aspects of this large undertaking:
 - Keywords and metadata
 - NRDC-coordination
 - Codes and database

- Corrections and quality flags
- Testing outputs
- We have made progress on a requirements document and the format
 - Details the experimental metadata that we would like to be automatically parsable in the new database – expand on the EXFOR keywords and codes.

```
"incident particles":{
   "facility": {
           "code":"LINAC",
           "description": "Linear accelerator",
           "text": " measurement was conducted at Rensselaer Polytechnic Institute using the electron linear
   "incident source": {
       "code": "PHOTO",
       "description": "Photo-neutron",
       "text": "Neutrons were produced through a (gamma,n) reaction when electrons from the RPI 60-MeV elect
   "beam height":{"constant":{
        "value": 35.6.
        "unit": "cm",
       "type": "average",
        "uncertainty": "unknown",
                                             NAVAL NUCLEAR
        "uncertainty unit": "N/A",
                                             LABORATOR
       "uncertainty type":"N/A"}},
```

We will be developing a stringent format for the database

- We think that some parts of this format might be of interest to the NRDC, and will present the format in this setting to see if it is helpful
- We have not finalized the format yet, but instead have a few examples of the types of format specifics that might be of interest:
 - INC-SOURCE and Incident Reaction
 - STATUS ALTER
 - Compiling types of background when given



INC-SOURCE and Incident Reaction

```
SUBENT . . . . . . . . . 14576001 . . . 20191005 . .
                              20200218 - - 20200206 - - - - 1460
  TITLE .... Tantalum, titanium, and zirconium neutron total
   ····· cross-section measurements from 0.4 to 25 MeV
6 AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
   T.H.Trumbull, Y.Danon)
  REACTION (73-TA-181(N,TOT),SIG)
  INC-SOURCE (PHOTO) Neutrons were produced through a (gamma, n)
   reaction when electrons from the RPI 60-MeV electron
           linac interact with a tantalum target,
14
   .... Nominal beam energy .... 53 MeV
15
   ·················Nominal·beam·power······<u>····330·W</u>
16
                                  Neutron target ba
                                  SUBENT . . . . . . . . . 14576001 . . . . 20 91005 . . . . 20200218 . . . 20200206 . . . . . . . . 1460
                                  TITLE ... Tantalum, titanium and zirconium neutron total
                                  ··················cross-section·measurements·from·0.4·to·25·MeV
                                6 AUTHOR (M.J.Rapp, D.P.Barry, & Leinweber, R.C.Block, B.E.Epping,
                                7 .....T.H.Trumbull, Y.Danon)
                                  . . . . . . .
                                                              20200218 - - - 20200206 - - - - - 1460
                                  SUBENT ..... 14576002 ... 20191005.
                                  REACTION (73-TA-181(N,TOT),SIG)
                                  INC-SOURCE (PHOTO) [Ta(gamma, n)] Neutrons were produced through
                               13
                                   a (gamma, n) reaction when electrons from the RPI
                                   60-MeV electron linac interact with a tantalum target,
                               14
                                   .... Nominal beam energy .... 53 MeV
                               15
                                   ....Nominal·beam·power·····330·W
                                   Neutron target bare bounce target, no moderator
                               17
```

STATUS: Altered by Author

```
. . . . . . . 14576 . . . 20191005
                                       20200218
                                                   20200206 · · · · · · 1460
  ENTRY ·
             ....14576001...20191005...
                                       20200218
  SUBENT
                                                  .20200206.....1460
             BIB····
             ·Tantalum, titanium, and zirconium neutron total
4 TITLE · ·
              cross-section measurements from 0.4 to 25 MeV
6 AUTHOR
              (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
7 . . . . . . . . .
             T.H.Trumbull, Y.Danon)
  . . . . . . .
              (ALTER) Data sent has different bin structure
  STATUS ·
              than the plots in the reference
              (TABLE) Data sent by author (M.C.R.)
```



```
TITLE Tantalum, titanium, and zirconium neutron total
cross-section measurements from 0.4 to 25 MeV
AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
T.H.Trumbull, Y.Danon)
METHOD (TOF) Electron linac was operated in pulse mode,
···· detectors were placed at a distance of 100m and 250m.
····· Neutrons migrated through evacuated neutron flight
···· tubes and gamma rays were filtered using depleted
····· uranium. Neutrons were collimated using series of
···· lead, boron carbide, polyethylene, and iron
collimators. Neutron energy was calculated using
relativistic kinematics.
   ····· (TRN) ·Total · cross · section · was · measured · using
    transmission method - sample-in data were divided by
     sample-out data collected during each individual
        experiment. All data were corrected for asssociated
       background.
CORRECTION Data were corrected for dead time
MONITOR (6-C-12(N, TOT), SIG)
. . . .
```



```
TITLE Tantalum, titanium, and zirconium neutron total
cross-section measurements from 0.4 to 25 MeV
AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
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lead, boron carbide, polyethylene, and iron
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····· (TRN) · Total · cross · section · was · measured · using
transmission method - sample-in data were divided by
·····sample-out data collected during each individual
experiment. All data were corrected for asssociated
background.
CORRECTION Data were corrected for dead time
MONITOR (6-C-12(N, TOT), SIG)
. . . .
```



The background is typically the most difficult parameter to accurately define in a transmission analysis. A constant background and a time-dependent portion are the two components that generally determine the background. The constant background is generated from natural radiation in the environment, such as earthen and building materials and cosmic radiation. This component was determined from the time recorded prior to the generation of neutrons and was consistent with recorded measurements taken without the linac in operation. The time-dependent portion was determined to originate primarily from neutron capture events in the EJ-301 detector volume. 6 As the neutrons enter the volume and interact with the hydrocarbon liquid scintillator and generate the luminescence that is used to detect the presence of the neutron, they also lose energy. When the neutrons slow to thermal energy they can be captured by the hydrogen and release gamma radiation. These gamma rays are then recorded by the system as a background. The spectrum quickly builds while neutrons are impinging on the detector and then exponentially decays in time. To determine this time-dependent portion of the background, simulations were run utilizing the Monte Carlo code MCNP (Ref. 12). The MCNP results were used to obtain the background shape and were then normalized to long TOFs, where essentially no primary neutrons originating from the neutron-producing target are detected. Figure 3 shows an example of the constant background, also known as room background, and the exponentially decaying portion of the neutron data and how the MCNP-simulated background connects the two regions under the peak of the primary neutron collection.

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cross-section measurements from 0.4 to 25 MeV
AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
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METHOD . . . (TOF) Electron linac was operated in pulse mode,
detectors were placed at a distance of 100m and 250m.
Neutrons migrated through evacuated neutron flight
···· tubes and gamma rays were filtered using depleted
···· uranium. Neutrons were collimated using series of
lead, boron carbide, polyethylene, and iron
collimators. Neutron energy was calculated using
relativistic kinematics.
(TRN) Total cross section was measured using
.... transmission method - sample-in data were divided by
sample-out data collected during each individual
experiment. All data were corrected for asssociated
background.
CORRECTION Data were corrected for dead time
        ·Data·were · corrected · for · constant · and · time-dependent
        background.
MONITOR \cdot \cdot \cdot \cdot \cdot (6-C-12(N, TOT), SIG)
```



The background is typically the most difficult parameter to accurately define in a transmission analysis. A constant background and a time-dependent portion are the two components that generally determine the background. The constant background is generated from natural radiation in the environment, such as earthen and building materials and cosmic radiation. This component was determined from the time recorded prior to the generation of neutrons and was consistent with recorded measurements taken without the linac in operation. The time-dependent portion was determined to originate primarily from neutron capture events in the EJ-301 detector volume.⁶ As the neutrons enter the volume and interact with the hydrocarbon liquid scintillator and generate the luminescence that is used to detect the presence of the neutron, they also lose energy. When the neutrons slow to thermal energy they can be captured by the hydrogen and release gamma radiation. These gamma rays are then recorded by the system as a background. The spectrum quickly builds while neutrons are impinging on the detector and then exponentially decays in time. To determine this time-dependent portion of the background, simulations were run utilizing the Monte Carlo code MCNP (Ref. 12). The MCNP results were used to obtain the background shape and were then normalized to long TOFs, where essentially no primary neutrons originating from the neutron-producing target are detected. Figure 3 shows an example of the constant background, also known as room background, and the exponentially decaying portion of the neutron data and how the MCNP-simulated background connects the two regions under the peak of the primary neutron collection.

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AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
T.H.Trumbull, Y.Danon)
METHOD · · · · (TOF) · Electron · linac · was · operated · in · pulse · mode,
detectors were placed at a distance of 100m and 250m.
Neutrons migrated through evacuated neutron flight
···· tubes and gamma rays were filtered using depleted
····· uranium. Neutrons were collimated using series of
lead, boron carbide, polyethylene, and iron
collimators. Neutron energy was calculated using
relativistic kinematics.
(TRN) Total cross section was measured using
transmission method - sample-in data were divided by
sample-out data collected during each individual
experiment. All data were corrected for asssociated
background.
CORRECTION Data were corrected for dead time
         Data were corrected for constant and time-dependent
         background.
         The constant background, due to natural
        radiation, was determined by a measurement at TOF
        before neutron generation.
MONITOR \cdot \cdot \cdot \cdot \cdot (6-C-12(N, TOT), SIG)
```



The background is typically the most difficult parameter to accurately define in a transmission analysis. A constant background and a time-dependent portion are the two components that generally determine the background. The constant background is generated from natural radiation in the environment, such as earthen and building materials and cosmic radiation. This component was determined from the time recorded prior to the generation of neutrons and was consistent with recorded measurements taken without the linac in operation. The time-dependent portion was determined to originate primarily from neutron capture events in the EJ-301 detector volume.⁶ As the neutrons enter the volume and interact with the hydrocarbon liquid scintillator and generate the luminescence that is used to detect the presence of the neutron, they also lose energy. When the neutrons slow to thermal energy they can be captured by the hydrogen and release gamma radiation. These gamma rays are then recorded by the system as a background. The spectrum quickly builds while neutrons are impinging on the detector and then exponentially decays in time. To determine this time-dependent portion of the background, simulations were run utilizing the Monte Carlo code MCNP (Ref. 12). The MCNP results were used to obtain the background shape and were then normalized to long TOFs, where essentially no primary neutrons originating from the neutron-producing target are detected. Figure 3 shows an example of the constant background, also known as room background, and the exponentially decaying portion of the neutron data and how the MCNP-simulated background connects the two regions under the peak of the primary neutron collection.

ENTRY · · · · · · · · · · 14576 · · · 20191005 · · · 20200218 · · · 20200206 · · · · · · · · 1460
SUBENT · · · · · · · · 14576001 · · · 20191005 · · · 20200218 · · · 20200206 · · · · · · · · 1460
BIB · · · · · · · · · · · · · · · · · ·
TITLE · · · · · Tantalum, · titanium, · and · zirconium · neutron · total
·····cross-section measurements from 0.4 to 25 MeV
AUTHOR (M.J.Rapp, D.P.Barry, G.Leinweber, R.C.Block, B.E.Epping,
T.H.Trumbull, Y.Danon)
• • • •
METHOD · · · · · (TOF) · Electron · linac · was · operated · in · pulse · mode,
detectors were placed at a distance of 100m and 250m.
Neutrons migrated through evacuated neutron flight
tubes and gamma rays were filtered using depleted
uranium. Neutrons were collimated using series of
·····lead, boron carbide, polyethylene, and iron
····· collimators. Neutron energy was calculated using
·····relativistic kinematics.
······································
····· transmission method - sample-in data were divided by
·····sample-out data collected during each individual
····· experiment. All data were corrected for asssociated
····background.
CORRECTION Data were corrected for dead time
·······················Data·were·corrected·for·constant·and·time-dependent
··················background.·
······································
······································
before neutron generation.
The time-dependent background, due primarily to
neutron capture in the EJ-301 detector volume,
was simulated in MCNP and normalized to long TOF data.
MONITOR (6-C-12(N, TOT),, SIG)



The background is typically the most difficult parameter to accurately define in a transmission analysis. A constant background and a time-dependent portion are the two components that generally determine the background. The constant background is generated from natural radiation in the environment, such as earthen and building materials and cosmic radiation. This component was determined from the time recorded prior to the generation of neutrons and was consistent with recorded measurements taken without the linac in operation. The time-dependent portion was determined to originate primarily from neutron capture events in the EJ-301 detector volume. As the neutrons enter the volume and interact with the hydrocarbon liquid scintillator and generate the luminescence that is used to detect the presence of the neutron, they also lose energy. When the neutrons slow to thermal energy they can be captured by the hydrogen and release gamma radiation. These gamma rays are then recorded by the system as a background. The spectrum quickly builds while neutrons are impinging on the detector and then exponentially decays in time. To determine this time-dependent portion of the background, simulations were run utilizing the Monte Carlo code MCNP (Ref. 12). The MCNP results were used to obtain the background shape and were then normalized to long TOFs, where essentially no primary neutrons originating from the neutron-producing target are detected. Figure 3 shows an example of the constant background, also known as room background, and the exponentially decaying portion of the neutron data and how the MCNP-simulated background connects the two regions under the peak of the primary neutron collection.

Conclusions

- Templates of expected measurement uncertainties will soon be submitted for many neutron-induced observable measurements
 - They could be used by compilers to check if the authors have provided pertinent metadata and uncertainty sources needed for nuclear data evaluations.
 - They could help to render the entries more consistent across EXFOR
- WPEC SG50 is planning to develop a database with a stringent and parsable format that will be able to store "subjective" corrections
 - We are currently developing the requirements document that will list the metadata and numerical values we will have keywords for
 - We will periodically present the format to the NRDC to see if any of the developments are of interest to the community

