### SOME EXAMPLES OF UNCERTAINTY ANALYSIS IN ACTIVATION MEASUREMENTS

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### Outlook

- Importance of the correct uncertainty estimation
- Uncertainty analysis of a neutron induced activation measurement at single energy point
- Energy-energy correlations
- Uncertainties of interfering reaction cross sections
   deduced from measurements using samples with different
   isotopic compositions
- Reaction-reaction correlations



#### Importance of the correct uncertainty estimation

"A primary benefit of improved nuclear data lies in the perspective of cost reductions in developing and operating nuclear reactors; with precise nuclear data, nuclear systems can be designed to reach high efficiencies whilst maintaining adequate safety standards in a cost-effective manner.

Nuclear data are an integral part of reactor codes. Such codes are used by many in the research, development and plant operations fields, often with little awareness of the impact of nuclear data on the final results. That impacts are large. It is manifestly demonstrated by sensitivity studies that relate the uncertainties of calculated system parameters to the uncertainties of the underlying nuclear data. "[1]

|       |                     | Energy Range      | Current<br>Accuracy<br>(%) | Target<br>Accuracy (%)                            |  |  |  |
|-------|---------------------|-------------------|----------------------------|---|--|--|--|
| U238  | σ <sub>inel</sub>   | 6.07 ÷ 0.498 MeV  | 10 ÷ 20                    | $2 \div 3$  |  |  |  |
|       | σ <sub>capt</sub>   | 24.8 ÷ 2.04 keV   | 3 ÷ 9                      | 1.5 ÷ 2   |  |  |  |
| Pu241 | $\sigma_{\rm fiss}$ | 1.35 MeV ÷ 454 eV | 8 ÷ 20                     | $2 \div 3$ (SFR,GFR,LFR)<br>$5 \div 8$ (ABTR,EFR) |  |  |  |

Table 2 . Summary of the SG26 Highest Priority Target Accuracies for Fast Reactors

[1] EUR 23977 EN : Nuclear data for sustainable nuclear energy Coordinated action on nuclear data for industrial development in Europe CANDIDE

## Uncertainty analysis of a neutron induced activation measurement at single energy point I

$$\sigma_{x} = \sigma_{st} \frac{A_{x}}{A_{st}} \frac{[I \epsilon F n \Phi_{0}]_{st}}{[I \epsilon F n \Phi_{0}]_{x}} \frac{C_{flux,x}}{C_{flux,st}} \frac{C_{low,x}}{C_{low,st}} \frac{C_{cions,x}}{C_{coins,st}}$$
  
Uncorrelated:  $\sigma_{st} A_{x} A_{st} A_{st} I_{x} n_{x}$ ,  $n_{st}$   
Correlated:  $F, \epsilon_{x}/\epsilon_{st}$ , some correction factors

$$F = \frac{\lambda}{exp(-\lambda t_c) (1 - exp(-\lambda t_e)) (1 - exp(-\lambda t_m))}$$
$$F = \frac{1}{F(T_{1/2}) F(T_{1/2}, t_{irr}) F(T_{1/2}, t_c) F(T_{1/2}, t_m)}$$

$$X = F(z_1(m), z_2(m), \dots, z_p(m))$$
  

$$S^2(x) = \sum_{i=1}^p \left[ \left( \frac{\partial F}{\partial z_k} \right) \Big|_{z_k} \right]^2 S^2(z_k) + 2 \sum_{k=1}^p \sum_{j < k}^p \left( \frac{\partial F}{\partial z_k} \right) \Big|_{z_k} \left( \frac{\partial F}{\partial z_j} \right) \Big|_{z_j} S(z_k, z_j)$$

$$z_{k,j} = f_{k,j}(m)$$

$$S(z_k, z_j) = \left(\frac{\partial z_k}{\partial m}\right)\Big|_m \left(\frac{\partial z_j}{m}\right)\Big|_m S^2(m)$$
EA

## Uncertainty analysis of a neutron induced activation measurement at single energy point II

Covariance matrix of detector efficiency  $\epsilon_i = aE_i^b$ 

### $<\delta\epsilon_i\delta\epsilon_j>=<\delta a\delta a>+b^2(lnE_i)(lnE_j)<\delta b\delta b>+(b(\ln(E_i)+b(\ln(E_j))<\delta a\delta b>$

|     | TT                | Table 3 Und             | certainties of            | interpolated     | value                             | es ir | ı det | tecti          | on e | fficie | ency |     |     | _   |     |
|-----|-------------------|-------------------------|---------------------------|------------------|-----------------------------------|-------|-------|----------------|------|--------|------|-----|-----|-----|-----|
| No. | Radio-<br>nuclide | E <sub>γ</sub><br>(MeV) | $\varepsilon(E_{\gamma})$ | Std. dev.<br>(%) | Correlation matrix $(\times 100)$ |       |       |                |      |        |      |     |     |     |     |
| 1   | <sup>51</sup> Ti  | 0.320                   | 9.45 E-2                  | 2.28             | 100                               |       |       |                |      |        |      |     |     |     |     |
| 2   | 115 <i>m</i> In   | 0.336                   | 8.96 $E-2$                | 2.23             | 100                               | 100   |       |                |      |        |      |     |     |     |     |
| 3   | <sup>196</sup> Au | 0.356                   | 8.43 $E$ -2               | 2.18             | 100                               | 100   | 100   |                |      |        |      |     |     |     |     |
| 4   | 113mIn            | 0.392                   | 7.61 $E$ -2               | 2.08             | 100                               | 100   | 100   | 100            |      |        |      |     |     |     |     |
| 5   | <sup>58</sup> Co  | 0.811                   | 3.51 E-2                  | 1.42             | 96                                | 96    | 96    | 97             | 100  |        |      |     |     |     |     |
| 6   | <sup>54</sup> Mn  | 0.835                   | 3.40 E-2                  | 1.40             | 95                                | 95    | 96    | <del>9</del> 6 | 100  | 100    |      |     |     |     |     |
| 7   | <sup>27</sup> Mg  | 0.844                   | 3.36 $E-2$                | 1.39             | 95                                | 95    | 96    | 96             | 100  | 100    | 100  |     |     |     |     |
| 8   | <sup>56</sup> Mn  | 0.847                   | 3.35 E-2                  | 1.39             | 95                                | 95    | 96    | 96             | 100  | 100    | 100  | 100 |     |     |     |
| 9   | <sup>64</sup> Cu  | 1.346                   | 2.04 $E-2$                | 1.07             | 80                                | 80    | 81    | 82             | 94   | 95     | 95   | 95  | 100 |     |     |
| 10  | <sup>24</sup> Na  | 1,369                   | 2.01 E-2                  | 1.06             | 79                                | 80    | 80    | 82             | 94   | 94     | 94   | 94  | 100 | 100 |     |
| 11  | <sup>32</sup> P   | β                       |                           | 2.0              | 0                                 | 0     | 0     | 0              | 0    | 0      | 0    | 0   | 0   | 0   | 100 |

W. Mannhart, "A Small Guide to Generating Covariance of Experimental Data", INDC(NDS)-0588



# Uncertainty analysis of a neutron induced activation measurement at single energy point III

#### Non-linear function

Nonlinear case should be carefully applied. It is applicable only at certain conditions. The interconnection of the expectation value of the function  $\langle f \rangle$  and the expectation values of the various variables  $\langle x_i \rangle$  requires the knowledge of the joint probability density function.

$$\langle x_i \rangle = \mu_i$$
  $f(x_i) \cong f(\mu_i) + \sum_i \frac{\partial f}{\partial x_i} \Big|_{\mu} (x_i - \mu_i)$ 

$$<\delta f\delta f>=\sum_{i}<\delta x_{i}\delta x_{i}>+\sum_{i}\sum_{j}b_{i}b_{j}<\delta x_{i}\delta x_{j}>$$

The criterion for applicability of the formula is:  $\langle dx_i dx_i \rangle \ll \mu_i^2$ 

If the explicit expression of the derivatives cannot be obtained in analytical form the approximate value of the derivatives can be calculated numerically using the expression:

$$\frac{\partial f}{\partial x_i} \approx \frac{f(x_i + \Delta x_i) - f(x_i - \Delta x_i)}{2\Delta x_i}$$



## Uncertainty analysis of a neutron induced activation measurement at single energy point V

$$\sigma_{x} = \sigma_{st} \frac{S_{x}}{S_{st}} \frac{[I \in Fn\Phi_{0}]_{st}}{[I \in Fn\Phi_{0}]_{x}} \frac{C_{flux,x}}{C_{flux,st}} \frac{C_{low,x}}{C_{low,st}} \frac{C_{cions,x}}{C_{coins,st}}$$

| 1                                       |                      |      |       |      | 5     |      |       | 0     |       |  |
|---|----------------------|------|-------|------|-------|------|-------|-------|-------|--|
|   | Neutron energy (MeV) |      |       |      |       |      |       |       |       |  |
|   | 8.34                 | 9.15 | 13.33 | 16.1 | 17.16 | 17.9 | 19.36 | 19.95 | 20.61 |  |
| $\sigma_{ m Al}$                        | 1.9                  | 1.9  | 1.6   | 2    | 2     | 2.2  | 3.1   | 4.1   | 5.4   |  |
| $S_{ m Am}$                             | 5.0                  | 4.0  | 2.5   | 2.1  | 1.5   | 1.3  | 6.3   | 1.4   | 5.7   |  |
| $S_{\rm Al}$                            | 1.0                  | 1.0  | 1.0   | 1.0  | 1.0   | 0.7  | 2.0   | 1.0   | 1.6   |  |
| $I_{ m Am}$                             | 1.2                  | 1.2  | 1.2   | 1.2  | 1.2   | 1.2  | 1.2   | 1.2   | 1.2   |  |
| $n_{\rm Al}$                            | 0.1                  | 0.1  | 0.1   | 0.1  | 0.1   | 0.1  | 0.1   | 0.1   | 0.1   |  |
| $n_{\rm Am}$                            | 0.3                  | 0.3  | 0.3   | 0.3  | 0.3   | 0.3  | 0.3   | 0.3   | 0.3   |  |
| $\epsilon_{ m Al}/\epsilon_{ m Am}$     | 3.0                  | 3.0  | 3.0   | 3.0  | 3.0   | 3.0  | 3.0   | 3.0   | 3.0   |  |
| $(f_{\Sigma}f_r)_{\rm Am}$              | 0.9                  | 0.6  | 0.4   | 0.6  | 0.6   | 0.7  | 0.6   | 0.6   | 0.6   |  |
| $\frac{C_{\rm low,Am}}{C_{\rm low,Al}}$ |                      |      | 0.3   | 0.3  | 0.3   | 0.3  | 1.3   | 1.4   | 1.4   |  |

Table 4.2: Principal sources of uncertainty and their estimated magnitudes in %.

0.6



#### **Energy-energy correlations I**

$$\sigma_{x} = \sigma_{st} \frac{S_{x}}{S_{st}} \frac{[I \epsilon F n \Phi_{0}]_{st}}{[I \epsilon F n \Phi_{0}]_{x}} \frac{C_{flux,x}}{C_{flux,st}} \frac{C_{low,x}}{C_{low,st}} \frac{C_{cions,x}}{C_{coins,st}}$$

Fully correlated:  $I_x$ ,  $F_x$ ,  $\varepsilon_x/\varepsilon_{st}$ ,  $I_{st}$ ,  $F_{st}$ , Partially correlated terms:  $\sigma_{st}$ ,  $(n_x, n_{st})$ Uncorrelated  $S_x$ ,  $S_{st}$  (only diagonal elements)

If we use functional standard deviations ( $\Delta x/\langle x \rangle$ ) and functional covariance ( $V_{xy}/(\langle x \rangle \langle y \rangle)$ ) the covariance matrix can be presented as C= $\Sigma A_i$  with  $A_i$  relative covariance matrices corresponding to different terms needed to calculate the cross sections.



### **Energy-energy correlations II**

|   | 13.8 | 53.5449 |         |         |         |       |
|---|------|---------|---------|---------|---------|-------|
|   | 15.6 | 26.282  | 41.2949 |         |         |       |
| _ | 17.6 | 26.3829 | 27.77   | 96.2949 |         |       |
|   | 18   | 24.1868 | 25.418  | 26.65   | 37.6949 |       |
|   | 19.3 | 24.0456 | 25.172  | 26.97   | 24.99   | 42.29 |
|   |      | _       |         |         |         |       |

|      | Cross   | Rel.     |     | Corre  | lation |     |     |
|------|---------|----------|-----|--------|--------|-----|-----|
| En   | section | Std.Dev. |     | matrix | x 100  |     |     |
| 13.8 | XXX     | 7.32     | 100 |        |        |     |     |
| 15.6 | XXX     | 6.43     | 56  | 100    |        |     |     |
| 17.6 | xxx     | 9.81     | 37  | 44     | 100    |     |     |
| 18   | xxx     | 6.14     | 54  | 64     | 44     | 100 |     |
| 19.3 | XXX     | 6.50     | 51  | 60     | 42     | 63  | 100 |



## Uncertainties of interfering reaction cross sections deduced from measurements using samples with different isotopic compositions



$$RR_{183W_np} = \frac{N_{184,es}RR_{ns} - N_{184,ns}RR_{es}}{N_{183,ns}N_{184es} - N_{184,ns}N_{183,es}} \qquad RR_{184W_nx} = \frac{N_{183,ns}RR_{es} - N_{183,es}RR_{ns}}{N_{183,ns}N_{184es} - N_{184,ns}N_{183,es}} RR_{183W_np} = a_1RR_{ns} + a_2RR_{es} \qquad RR_{184W_nx} = a_3RR_{es} + a_4RR_{ns} P_1 = (x_1, x_2), P_2 = (x_1, x_2) \begin{bmatrix} u_{p_1}^2 \\ cov(p_1, p_2) & u_{p_2}^2 \end{bmatrix} = \begin{bmatrix} \frac{\partial p_1}{\partial x_1} & \frac{\partial p_1}{\partial x_2} \\ \frac{\partial p_2}{\partial x_1} & \frac{\partial p_2}{\partial x_2} \end{bmatrix} \begin{bmatrix} u_{x1}^2 \\ cov(x1, x2) & u_{x2}^2 \end{bmatrix} \begin{bmatrix} \frac{\partial p_1}{\partial x_1} & \frac{\partial p_2}{\partial x_2} \\ \frac{\partial p_1}{\partial x_2} & \frac{\partial p_2}{\partial x_2} \end{bmatrix}$$



#### **Reaction-reaction correlations**

 $\sigma_2^{[183}W(n,p)^{183}Ta] / \sigma_1^{[27}Al(n,\alpha)]$ 

 $\sigma_3[^{184}W(n,x)]^{183}Ta / \sigma_1[^{27}Al(n,\alpha)]$ 

$$\sigma_i / \sigma_j = P_i / P_j$$
$$P_i = \frac{A_i}{\epsilon_i} \frac{1}{N_i} \sum_{l} k_i$$

 $R_{12} = P_2/P_1$  and  $R_{13} = P_3/P_1$ 

 $< \delta R_{12} \delta R_{12} > = < \delta P_2 \delta P_2 > + < \delta P_1 \delta P_1 > -2 < \delta P_1 \delta P_2 >$ 

 $<\!\!\delta R_{13} \delta R_{13}\!\!> = <\!\!\delta P_3 \delta P_3\!\!> + <\!\!\delta P_1 \delta P_1\!\!> - 2 <\!\!\delta P_1 \delta P_3\!\!>$ 

 $<\!\!\delta R_{12} \delta R_{13} > = <\!\!\delta P_1 \delta P_1 > + <\!\!\delta P_2 \delta P_3 > - <\!\!\delta P_1 \delta P_3 > - <\!\!\delta P_2 \delta P_1 > \\<\!\!\delta R_{13} \delta R_{12} > = <\!\!\delta P_1 \delta P_1 > + <\!\!\delta P_3 \delta P_2 > - <\!\!\delta P_3 \delta P_1 > - <\!\!\delta P_1 \delta P_2 >$ 

$$\begin{bmatrix} < \delta R_{12} \delta R_{12} > & < \delta R_{12} \delta R_{13} > \\ < \delta R_{13} \delta R_{12} > & < \delta R_{13} \delta R_{13} > \end{bmatrix}$$
  
$$<\delta P_i \delta P_j > = <\delta A \delta A_j > + <\delta \varepsilon_i \delta \varepsilon_j > + <\delta N_i \delta N_j > + \Sigma <\delta k_i \delta k_j >$$

