

Prompt-neutron spectra from a general description of the fission process*

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Abstract

A new semi-empirical model of the fission process is described, which covers most of the properties of the fission fragments and the emitted neutrons and photons in a global and consistent way. The model is based on fragment shells that are deduced from measured fission-fragment mass distributions, assuming that the macroscopic contribution of the compound nucleus and the microscopic contributions of the nascent fragments in the potential-energy surface are separable. The distributions of the collective coordinates are attributed to the motion of the quantum oscillators in their respective potential pockets perpendicular to the fission path. Different contributions to the excitation energies of the final fragments and their division at scission are described with the help of statistical mechanics. Intrinsic excitation energies of the fragments at scission are consistently described together with the even-odd effect in fission-fragment Z distributions. Mass-dependent equilibrium deformations of the nascent fragments are adjusted to measured average prompt-neutron multiplicities. A unique set of parameters is found, which reproduces a large variety of measured data for all heavy fissioning systems with a good precision. In contrast to most available models, this approach is applicable to fissioning systems, for which no experimental data are available.

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1 Introduction

Global parametrisations and very elaborate models have been developed for calculating the energy spectra of prompt fission neutrons and their multiplicity distributions. Most of them are based on measured mass-TKE distributions of the fission fragments. With the help of the Q values for specific nuclear-charge and mass splits and by considering the initial excitation energy, the total excitation energy TXE of the fragments can directly be deduced. With an assumption on the division of the TXE between the fragments, which needs to be consistent with the observed mass-dependent neutron multiplicities, the initial conditions of both fragments for a statistical de-excitation code of the Weisskopf or Hauser-Feshbach type are determined. The task is appreciably more difficult when this experimental basis, the measured mass-TKE distribution, is not available. In this case, this information must be provided by a model calculation. The GEF code has been developed for this purpose. It is a semi-empirical model of the fission process, which covers most of the properties of the fission fragments and the emitted neutrons and photons in a global and consistent way. In addition to the mass-TKE distribution it also calculates the division of the TXE between the fragments and the angular momenta of the fragments. Moreover, the specific initial conditions of each individual fragment are given. This report gives an overview on the underlying physics ideas and the general technical features of the code and presents some results. More detailed information on the code can be found in the report JEF/DOC 1423 [1] and, if needed, in the source of the code [2]. The final aim of the present work is to provide predictions for the multiplicities and the energy spectra of the prompt fission neutrons.

2 Fission channels

2.1 Experimental systematics

Figure 1 gives an overview on the measured mass and nuclear-charge distributions of fission products from low-energy fission. Fission of target nuclei in the actinide region, mostly induced by neutrons, shows predominantly asymmetric mass splits. A transition to symmetric mass splits is seen around mass 258 in spontaneous fission of fusion residues. Electromagnetic-induced fission of relativistic secondary beams covers the transition from asymmetric to symmetric fission around mass 226 [3]. A pronounced fine structure close to symmetry appears in ^{201}Tl [4] and in ^{180}Hg [5]. It is difficult to

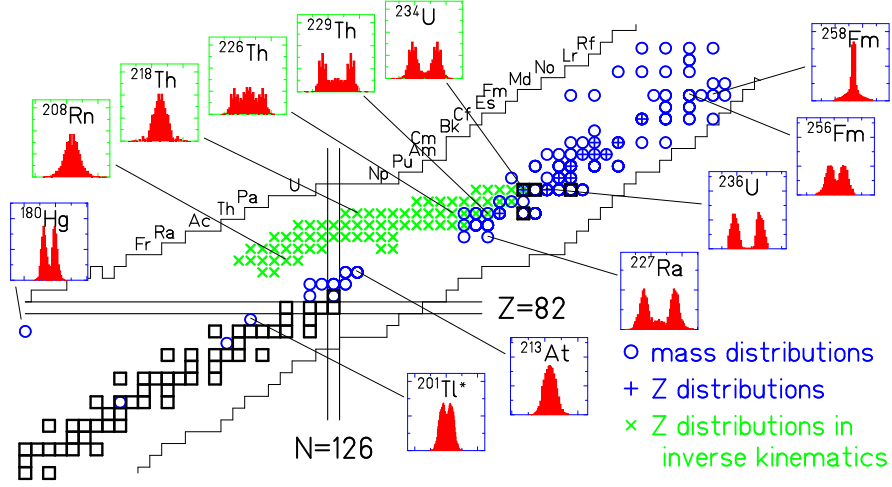


Figure 1: General view on the systems for which mass or nuclear-charge distributions have been measured. The distributions are shown for 12 selected systems. Blue circles (blue crosses): Mass (nuclear-charge) distributions, measured in conventional experiments [4, 5], and references given in [3]. Green crosses: Nuclear-charge distributions, measured in inverse kinematics [3].

observe low-energy fission in this mass range. Thus, ^{201}Tl could only be measured down to 7.3 MeV above the fission barrier due to its low fissility, which explains the filling of the minimum between the two peaks. Only ^{180}Hg was measured at energies close to the barrier after beta decay of ^{180}Tl . Considering the measured energy dependence of the structure for ^{201}Tl [4], the fission characteristics of these two nuclei are rather similar. Also other nuclei in this mass region show similar features, which have been attributed to the influence of fragment shells [6]. These shells are different from those governing the asymmetric fission of the actinides. They are not considered in the present model that concentrates on heavier nuclei with mass numbers $A > 200$, which are more important for technical applications.

2.2 Size of the heavy fragment in asymmetric fission

In the range where asymmetric fission prevails, e.g. from ^{227}Ra to ^{256}Fm , the light and the heavy fission-product components gradually approach each other, see figure 1. A quantitative analysis reveals that the mean mass of

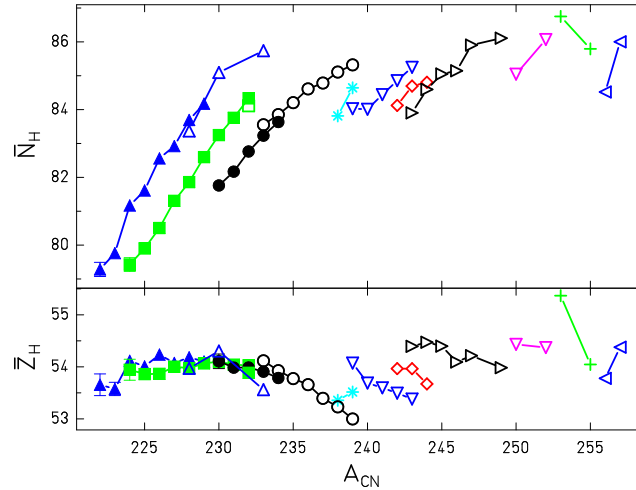


Figure 2: Mean neutron and proton number of the heavy component in asymmetric fission in the actinide region before the emission of prompt neutrons. The values were deduced from measured mass and nuclear-charge distributions using the semi-empirical GEF code [2] for the correction of charge polarization and prompt-neutron emission. Open symbols denote results from conventional experiments, full symbols refer to an experiment with relativistic projectile fragments of ^{238}U [3]. Data points for the same Z_{CN} are connected (See [2] for references of the underlying experimental data.)

the heavy component stays approximately constant [7] at about $A = 140$. This has been explained by the influence of a deformed ($\beta \approx 0.6$) fragment shell at $N = 88$ and the spherical shell at $N = 82$ [8], suggesting that the position of the heavy fragment is essentially constant in neutron number.

New data on Z distributions over long isotopic chains [3], however, reveal very clearly that the position in neutron number varies systematically over more than 7 units, while the position in proton number is approximately constant at $Z = 54$, see figure 2. The rather short isotopic sequences covered in former experiments did not show this feature clearly enough and gave the false impression of a constant position in mass. This finding represents a severe puzzle to theory, since shell-model calculations do not show any shell stabilization near $Z = 54$ at $\beta \approx 0.6$ [8, 9].

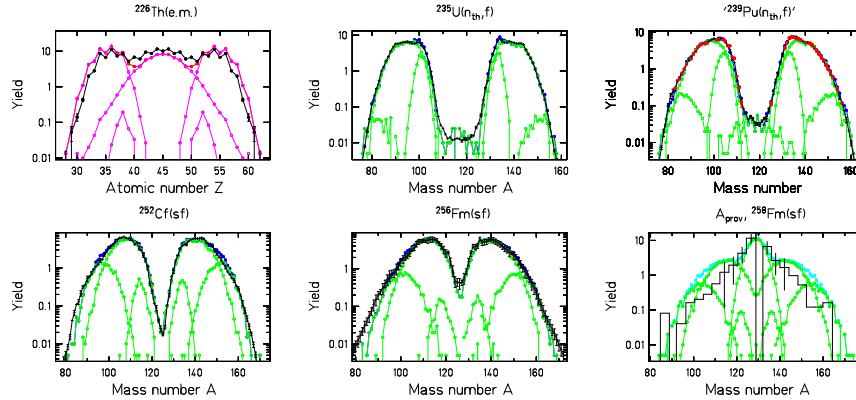


Figure 3: Nuclear-charge and post-neutron mass distributions of fission fragments. (For $^{258}\text{Fm}(\text{sf})$ the provisional mass A_{prov} is shown, which is directly deduced from the ratio of the kinetic energies of the fragments and thus not corrected for neutron emission.) Experimental or evaluated data (black lines, respectively histogram) for electromagnetic-induced (e.m.), thermal-neutron-induced (n_{th},f) and spontaneous fission (sf) are compared with predictions of the GEF code [2] (red and green lines). The red symbols for $^{239}\text{Pu}(n_{th},f)$ show the experimental data behind the evaluation. The contributions of different fission channels are shown. (See [2] for references of the data.)

2.3 Separability principle

The microscopic-macroscopic approach has proven to be very useful for calculating nuclear properties, in particular in applications to fission [10]. The early influence of fragment shells on the fission path, deduced from two-centre shell-model calculations [11], makes its application to fission even more powerful. It means that the microscopic properties of the fission observables are essentially determined by the shells of the fragments, and only the macroscopic properties are specific to the fissioning system [12]. This separability principle was exploited in the GEF code [2], which relies on an empirical description of the macroscopic stiffness parameters in the relevant normal modes and empirically deduced fragment shells, which are valid for all fissioning systems. Figure 3 demonstrates that the mass distributions over a large range of systems can be described very well with the same parameter set. For a more complete overview see ref. [1].

2.4 Fluctuations

Most fission observables form bell-shaped distributions around a mean value. This suggests treating the corresponding collective degree of freedom as an harmonic quantum oscillator coupled to a heat bath of temperature T . Especially for the charge-polarization degree of freedom there exists a long discussion about the importance of the zero-point motion [16, 17]. Nix estimated the level spacing in the oscillator corresponding to mass-asymmetric distortions at saddle with the liquid-drop model to 1-2 MeV in the actinide region [18]. According to the smaller widths of the corresponding components to the mass distribution, the level spacing for oscillations in the two asymmetric fission valleys (Standard 2 and Standard 1) is expected to be even larger. Also for oscillations in the charge-polarization degree of freedom, the level spacing is in the order of 10 MeV [16, 17]. This value is appreciably larger than the temperature values of actinides, which are about 0.5 MeV in the constant-temperature regime [19]. Thus, in a statistical approach the charge-polarization degrees of freedom is essentially not excited, and also the widths of the fission channels in mass asymmetry are expected to be strongly influenced by the zero-point motion in low-energy fission.

Also the angular-momentum distributions of the fragments have been explained by orientation pumping due to the uncertainty principle [20]. Experimental indications for thermal excitations of spherical fragments [21] have also been explained by the compensation of the orbital angular momentum, which itself is induced by the zero-point motion [22]. Here it is the operator of the orbital angular momentum which does not commute with the angle that characterizes the direction of particle motion. Thus, all fragment angular momenta measured in low-energy fission [23] are explained by the quantum-mechanical uncertainty principle. There is no room for excitations of the angular-momentum-bearing modes [24].

Due to the strong influence of quantum-mechanical effects it is mandatory to explicitly consider these effects, as it is e.g. done in the self-consistent microscopic approach of ref. [25]. Stochastic approaches with classical models [26] may miss certain aspects of the fission dynamics.

2.5 Comparison with previous ideas

Several descriptions of the fission observables with applications of the statistical model have been proposed in the past. The present approach is rather close to the outline of a scenario proposed by Jensen and Dossing [27], although the present model covers a larger variety of observables. More

importantly, it also tries to better exploit available empirical information.

Jensen and Dossing presented a statistical calculation of the mass distribution in fission with some ideas about the dynamics of the process. The most important modifications applied in the GEF code are: (i) The shell effects that were calculated from single-particle energy spectra in a Woods-Saxon potential with the Strutinsky method in ref. [27] are replaced by global fragment shells, which are adjusted to the measured mass distributions. The separability principle simplifies this task considerably, since the fragment shells are assumed to depend only on the fragment, and, thus, they are the same for all fissioning systems. (ii) The nuclear level density that was calculated from the same single-particle spectrum including pairing correlations using the BCS approximation in ref. [27] is replaced at low excitation energies by an empirical constant-temperature formula [19], which seems to be in good agreement with recent experimental results [28]. (iii) The influence of quantum-mechanics, in particular the zero-point motion, has been considered to model the distributions of collective coordinates. They are attributed to the motion of the quantum oscillators in their respective potential pockets perpendicular to the fission path. The parameters of these oscillators are deduced from experimental data. In addition, the shapes of the fragments at scission, the charge polarization, the angular momenta, and other properties of the fragments are calculated on the basis of similar ideas.

3 Prompt-neutron yields

3.1 Transformation of energy - the different contributions

In low-energy fission, the Q value of the reaction ends up either in the total kinetic energy (TKE) or the total excitation energy (TXE) of the fragments. The TKE is closely related to the distance of the centres of the two nascent fragments at scission, but, even if the pre-scission kinetic energy is neglected, the TKE cannot give information on the shapes of the individual fragments. The TXE, however, can be attributed to the individual fragments by a kinematical measurement of the prompt-neutrons. Still, there is no direct experimental information available on the processes, which are responsible for the transformation of part of the Q value into the excitation energies of the separated fragments. The situation is schematically illustrated in figure 4. Before scission, dissipation leads to intrinsic excitations, collective modes perpendicular to the fission direction (normal modes [18]) may be excited, and, finally, some energy is stored in deformation of the nascent fragments

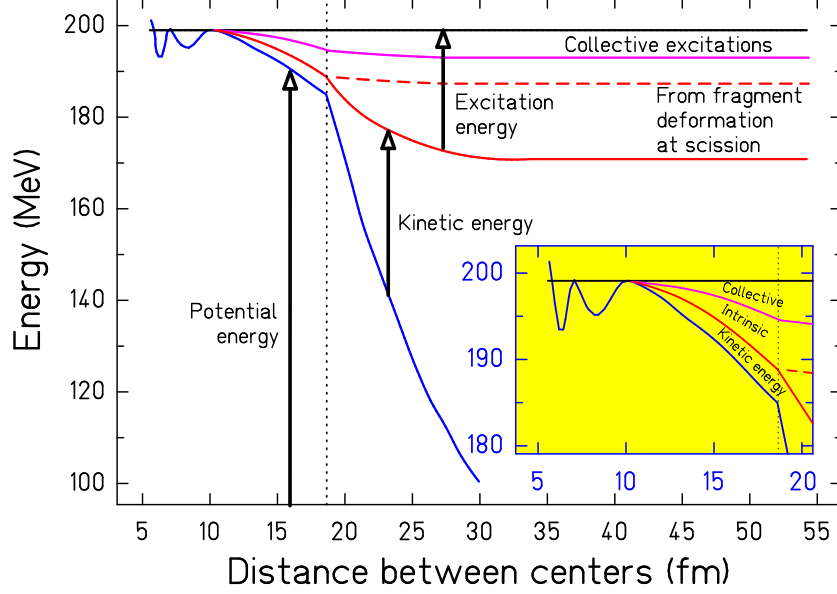


Figure 4: Schematic drawing of the transformation of energy during the fission process of an initial excitation energy equal to the height of the fission barrier.

that is induced by the Coulomb repulsion. The remaining part is found as pre-scission kinetic energy [29]. After scission, collective excitations and deformation energy are transformed and add up to the intrinsic excitations of the separated fragments.

The situation at scission is important for the understanding of fission dynamics, e.g. the magnitude of dissipation and the coupling between the different collective degrees of freedom, but without additional information, the repartition of the different contributions between the fragments remains ambiguous.

3.2 Origin of the saw-tooth shape

There is widespread agreement that the saw-tooth shape of the prompt-neutron yields, see figure 5, is caused by the deformation energies of the nascent fragments at scission. The scission-point model of ref. [8] attributes it to the influence of fragment shells, the random-neck-rupture model [30]

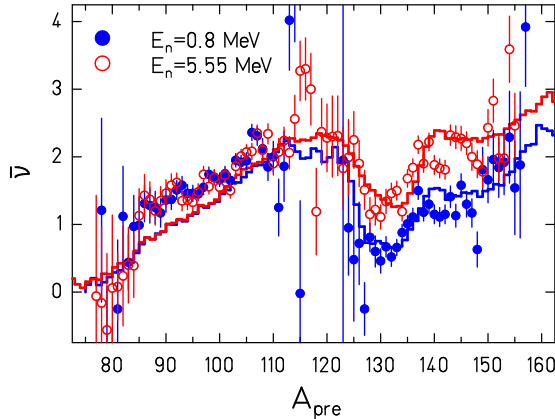


Figure 5: Measured prompt-neutron yield in $^{237}\text{Np}(n,f)$ as a function of pre-neutron mass at two different incident-neutron energies [32] (data points) in comparison with the result of the GEF code [2] (histograms).

links it to the location of the rupture, and also microscopic calculations predict large deformation energies of the fragments near scission [31]. Large even-odd effects in the fragment Z distributions indicate that the intrinsic excitation energy at scission is generally much too low to account for the variation of the prompt-neutron yield by several units over the different fragments.

3.3 Differential behaviour - energy sorting

Recent experimental results reveal that nuclei exhibit an essentially constant temperature, may be up to excitation energies of 20 MeV [28] with a temperature parameter that is grossly proportional to $A^{-2/3}$ [19]. This behaviour is explained by the breaking of pairs in the so-called superfluid regime [33]. This leads to a considerable increase of the heat capacity [34] and consequently to a slow variation of temperature as a function of excitation energy. Note that the BCS approximation severely underestimates the pairing condensation energy and consequently also the magnitude of the heat capacity in the so-called superfluid regime [35]. Thus, the assumption of a constant nuclear temperature becomes a good approximation. This implies that the intrinsic excitation energy of the two nascent fragments at scission is subject to energy sorting [36, 37, 38]: The hotter light fragment

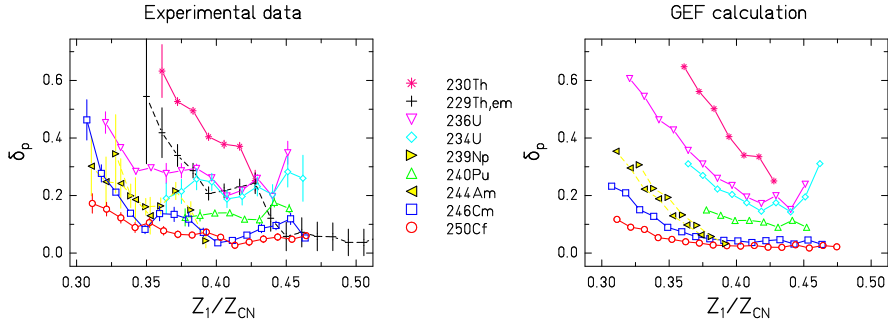


Figure 6: Measured (left) and calculated (right) local even-odd effect in fission-fragment Z distributions in (n_{th}, f) reactions. The fissioning nuclei are indicated. Data for fission of ^{229}Th , induced by electromagnetic excitations are included. See ref. [39] for references of the data.

transfers essentially all its intrinsic excitation energy to the colder heavy fragment. This energy sorting manifests itself in the mass-dependent neutron yields. Figure 5 shows data for neutron-induced fission of ^{237}Np with $E_n = 0.8$ MeV and $E_n = 5.55$ MeV as an example. The additional initial energy leads to an increased neutron yield from the heavy fragments, only. The behaviour is well reproduced by the GEF code, which includes a model for the process of energy sorting.

4 Even-odd effect in Z yields

4.1 Experimental systematics

A systematic view on the local even-odd effect in fission-fragment Z distributions [39] reveals a regular pattern and a general dependence on the fissioning system, see figure 6. The magnitude of the even-odd effect is small at symmetry, and it increases strongly with increasing asymmetry. At the same time, the even-odd effect generally decreases for heavier systems. The even-odd effect in the light fragment group of nearby even- Z and odd- Z systems is essentially identical, except at symmetry, where the even-odd effect in odd- Z systems is exactly zero. Electromagnetic excitations lead to slightly higher excitation energies, thus reducing the magnitude of the even-odd effect. The large number of systems investigated revealed that the appearance of a large even-odd effect at large asymmetry is a general phenomenon, also in odd- Z fissioning systems [40]. In any case, there is

an enhancement of even- Z fragments in the light fragment group, indicating that it is the enhanced production of even- Z light fragments in their ground state, which is at the origin of the large even-odd effect at extreme asymmetry.

4.2 Final stage of energy sorting

It seems straightforward to attribute the enhanced production of even- Z light fragments to the energy-sorting mechanism [41] that explained already the differential behaviour of the prompt-neutron yields. If the time until scission is sufficient for the energy sorting to be accomplished, the system can still gain an additional amount of entropy by predominantly producing even-even light fragments. Compared to the production of odd-odd light fragments, the excitation energy of the heavy fragment increases by two times the pairing gap, and its entropy increases due to the increasing number of available states in the heavy fragment. The right part of figure 6 shows a calculation with the GEF code, where this idea is included in a schematic way. The basic features are: (i) The excitation energy induced by dissipation grows with the Coulomb parameter $Z^2/A^{1/3}$, and the time needed for complete energy sorting is correspondingly increased. Also the energy gain from saddle to scission increases, and, thus the energy exceeds the constant-temperature domain. This explains the observed reduction of the even-odd effect for heavier systems. (ii) The temperature difference of the nascent fragments grows with increasing asymmetry, which enhances the energy-sorting process. This explains the strong increase of the even-odd effect at large asymmetry.

5 Charge polarization

5.1 Experimental information

Most experimental information on charge polarization at scission is indirect, because only the fragment masses after the emission of prompt neutrons can be measured with good resolution. Thus, the influence of prompt-neutron emission has to be corrected. This correction introduces some uncertainties, because most data on mass-dependent prompt-neutron multiplicities are not very precise, and for many systems such data are not available.

Figure 7 shows the measured deviation of the mean nuclear charge from the UCD (unchanged charge distribution) value for a fixed post-neutron mass and the standard deviation of the corresponding nuclear-charge distri-

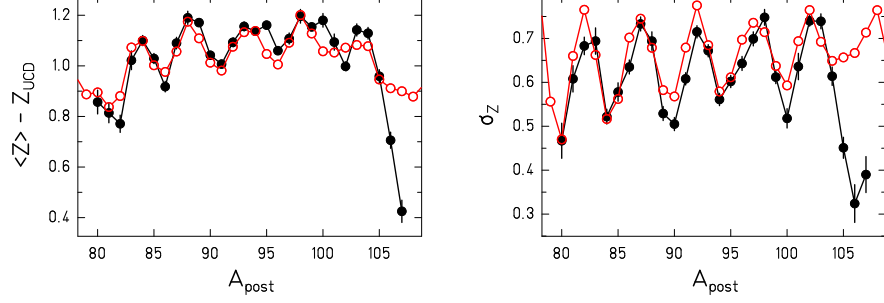


Figure 7: Indirect information on the charge polarization in $^{235}\text{U}(n_{th}, f)$. Left part: Deviation of the mean nuclear charge from the UCD (unchanged charge distribution) value for a fixed post-neutron mass A_{post} . Experimental data [42] (full points) are compared with the result of the GEF code [2] (open points). Right part: Standard deviation of the nuclear-charge distribution for a fixed post-neutron mass A_{post} . Experimental data [42] (full points) are compared with the result of the GEF code [2] (open points).

bution for the thermal-neutron-induced fission of ^{235}U [42]. The influence of the even-odd staggering of the Z yields is clearly visible in both quantities.

5.2 Simulation

The simulation of the nuclear-charge distributions for fixed post-neutron mass starts from the calculated pre-neutron nuclide distribution and the excitation energy of each individual fragment. The emission of prompt neutrons must be considered, which is constrained by measured mass-dependent prompt-neutron multiplicity distributions. The good agreement with post-neutron fragment distributions shown in figure 7 was obtained by minimizing the potential energy of the scission configuration, approximated by quadrupole-deformed fragments with a tip distance of 1 fm with respect to their N/Z ratios. However, for the asymmetric fission channels, the value of $\langle Z \rangle - Z_{UCD}$ had to be increased (decreased) by 0.3 units in the light (heavy) fragment. This additional charge polarization may be attributed to the influence of the shell effects. The mean deformation of the fragments at scission is linked to the mean prompt-neutron multiplicity, considering the amount of intrinsic excitation energy at scission, which is consistent with the description of the even-odd effect in the Z distributions.

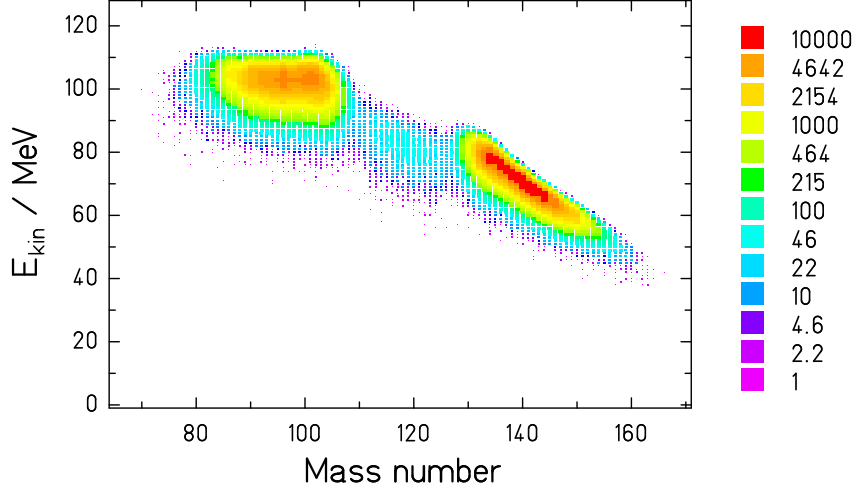


Figure 8: Two-dimensional distribution of kinetic energies and fission-fragment masses before emission of prompt neutrons for $^{235}\text{U}(n_{th},f)$. The colour scale refers to the counts of the Monte-Carlo calculation with the GEF code.

6 Fragment kinetic energies

In the GEF code, the total kinetic energy of the fission fragments is given by subtracting the total excitation energy of the separated fragments from the sum of the initial excitation energy of the fissioning nucleus and the Q value of the fission process. The resulting distribution for $^{235}\text{U}(n_{th},f)$ is shown in figure 8. The overall behaviour is in agreement with expectations from systematics. In the model, the shape of the energy distribution for a fixed mass is mainly defined by the distribution of fragment deformations at scission, which is taken as a Gaussian distribution with a maximum in the respective potential minimum. These shapes are assumed to be decisive for the amount of deformation energy of the separated fragments with respect to their respective ground state, which finally adds up to their intrinsic excitation energy. This explains the skewness of the distributions, which seem to be slightly larger than found in experiment.

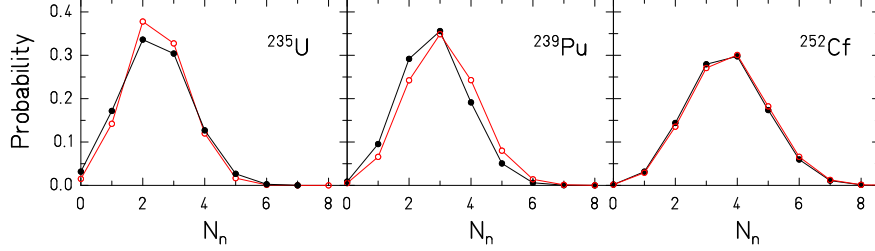


Figure 9: Measured prompt-neutron multiplicity distributions [43, 45] for $^{235}\text{U}(n_{th},f)$ (left part), $^{239}\text{Pu}(n_{th},f)$ (middle part) and $^{252}\text{Cf}(sf)$ (right part) are compared to the results of the GEF code (open symbols).

7 Neutron multiplicities

Besides the mass-dependent mean prompt-neutron yields, see figure 5, there exist two other experimental results, which have been determined with high accuracy: The mass-integrated neutron-multiplicity distribution and the mean number of prompt fission neutrons. The measured mean number of prompt-fission neutron yields is compared in table 1 with the values given by the GEF code for some selected systems. The same parameter set was used for all systems.

System	E_n	Exp	GEF
$^{235}\text{U}(n,f)$	thermal	2.41 [43]	2.47
$^{235}\text{U}(n,f)$	0.5 MeV	2.46 [44]	2.58
$^{235}\text{U}(n,f)$	5.55 MeV	3.19 [44]	3.38
$^{237}\text{Np}(n,f)$	0.8 MeV	2.73 [32]	2.63
$^{237}\text{Np}(n,f)$	5.55 MeV	3.46 [32]	3.43
$^{239}\text{Pu}(n,f)$	thermal	2.88 [43]	3.06
$^{252}\text{Cf}(sf)$	—	3.77 [45]	3.72

Table 1: Selected values of mean prompt-neutron multiplicities. The measured values are compared with the result of the GEF code.

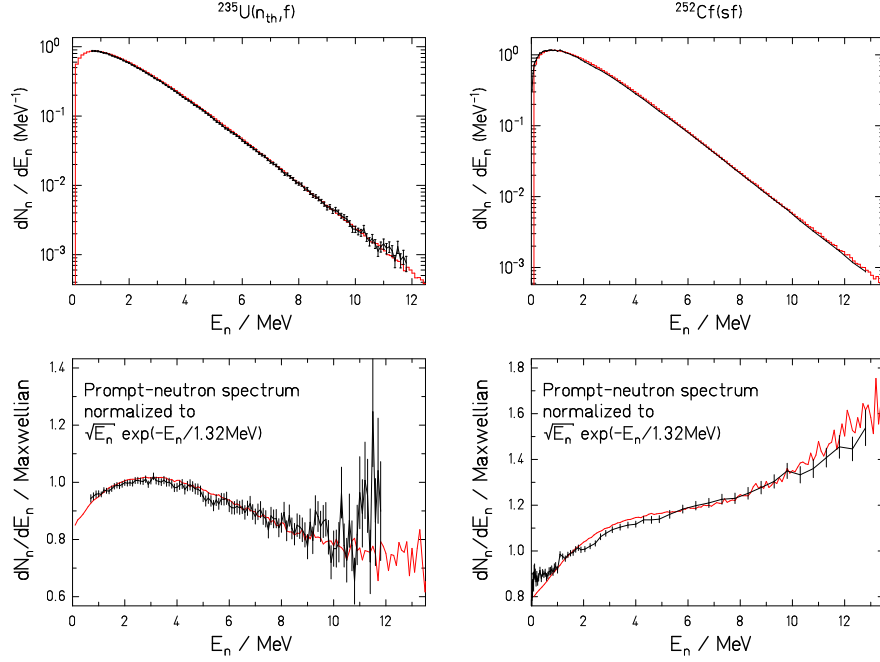


Figure 10: Experimental prompt-fission-neutron spectra (black lines and error bars) for $^{235}\text{U}(n_{th},f)$ [46] (left panels) and $^{252}\text{Cf}(sf)$ [47] (right panels) in comparison with the result of the GEF code (red lines) in logarithmic scale. In the lowest panels, all spectra have been normalized to a Maxwellian with $T = 1.32$ MeV.

8 Prompt-neutron spectrum

The experimental prompt-fission-neutron spectra for the systems $^{235}\text{U}(n_{th},f)$ [46] and $^{252}\text{Cf}(sf)$ [47] are compared with results of the GEF code in figure 10. In order to better visualize the deviations, the lower panels show a reduced presentation with the spectra normalized to a Maxwellian distribution with the parameter $T = 1.32$ MeV.

In this calculation, the de-excitation of the separated fragments has been obtained within the statistical model. It is assumed that both the emission of neutrons and the emission of E1 gammas does not change the angular momentum on the average, which seems to be a good approximation in the relevant angular-momentum range [48]. When the yrast line is reached, the angular momentum is carried away by a cascade of E2 gammas. The inverse neutron absorption cross section has been described by the parameterisation

from ref. [49]. Since the fast-neutron spectrum in fission is composed of the contributions from many emitting fragments, the use of this global description that is computed very quickly is probably not too critical. Gamma competition at energies above the neutron separation energy was considered. The gamma strength of the giant dipole resonance (GDR) following the description proposed in ref. [50] was applied. The nuclear level density was modelled by the constant-temperature description of v. Egidy and Bucurescu [52] at low energies. The level density was smoothly joined at higher energies with the modified Fermi-gas description of Ignatyuk et al. [53, 54] for the nuclear-state density:

$$\omega \propto \frac{\sqrt{\pi}}{12\tilde{a}^{1/4}U^{5/4}} \exp(2\sqrt{\tilde{a}U}) \quad (1)$$

with $U = E + E_{cond} + \delta U(1 - \exp(-\gamma E))$, $\gamma = 0.55$ and the asymptotic level-density parameter $\tilde{a} = 0.078A + 0.115A^{2/3}$. The shift parameter $E_{cond} = 2 \text{ MeV} - n\Delta_0$, $\Delta_0 = 12/\sqrt{A}$ with $n = 0, 1, 2$, for odd-odd, odd- A and even-even nuclei, respectively, as proposed in ref. [55]. δU is the ground-state shell correction. A constant spin-cutoff parameter was used. The matching energy is determined from the matching condition (continuous level-density values and derivatives of the constant-temperature and the Fermi-gas part). Values slightly below 10 MeV are obtained. The matching condition also determines a scaling factor for the Fermi-gas part. It is related with the collective enhancement of the level density. The corresponding results are shown by the red full lines. The transformation of the neutron-energies into the laboratory frame was performed considering the acceleration phase [56, 57] after scission by a numerical trajectory calculation. The mean pre-scission total kinetic energy was assumed to be 40% of the potential-energy gain from saddle to scission derived by Asghar and Hasse [58] as

$$< TKE >_{pre} = 0.032(Z^2/A^{1/3} - 1527) \text{ MeV} \quad (2)$$

with a standard deviation of the same amount. The distribution was truncated at negative values.

The good reproduction of the measured neutron spectra, especially for the lighter system $^{235}\text{U}(\text{n}_{th}, \text{f})$, does not give indication for neutron emission at scission [59, 60, 61, 62], although it is difficult to draw a definite conclusion due to the uncertainties in the level densities.

The emission during the acceleration phase is stronger for the system $^{252}\text{Cf}(\text{sf})$, since higher excitation energies and, thus, shorter emission times

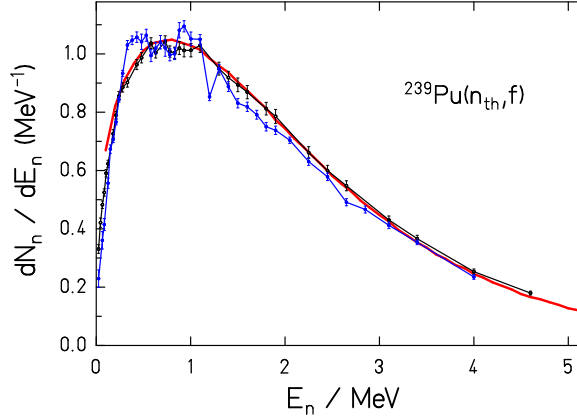


Figure 11: Experimental prompt-fission-neutron spectrum for the system $^{239}\text{Pu}(n_{th},f)$ from ref. [63] (black open symbols) and from ref. [64] (blue full symbols) in comparison with the result of the GEF code (red thick full line). The calculated spectrum was normalized to the measured total neutron multiplicity ($\bar{\nu} = 2.88$ [43]). The measured spectra are slightly scaled for minimizing the overall deviations from the calculated spectrum in order to better compare the spectral shapes.

are involved in this system. Neutron emission during fragment acceleration reduces especially the laboratory energies of the first neutrons emitted at short times from the most highly excited fragments in $^{252}\text{Cf}(sf)$ and allows for a decently consistent description of the two systems with the GEF code, using the same parameter set. Experimental prompt-fission neutron spectra of the systems $^{239}\text{Pu}(n_{th},f)$ and $^{240}\text{Pu}(sf)$ are compared with the result of the GEF code in figures 11 and 12, again using the same model parameters. Obviously, the data are very well reproduced.

In general, the GEF code reproduces the available experimental fission-prompt-neutron spectra rather well. This qualifies the GEF code for estimating prompt-neutron spectra in cases where experimental data do not exist. The results of the GEF code for thermal-neutron-induced fission of several target nuclei are listed in the appendix I. More data can be generated by downloading the code [2] and by performing the calculations for the appropriate fissioning system. The code also seems to be a suitable tool for improving evaluations.

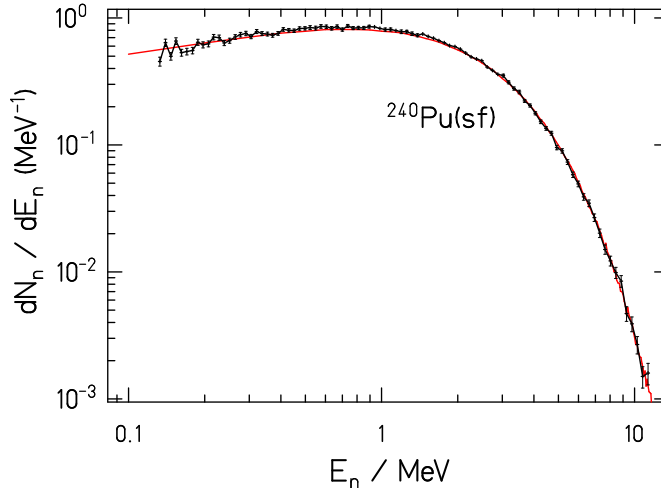


Figure 12: Experimental prompt-fission-neutron spectrum for the system $^{240}\text{Pu}(\text{sf})$ from ref. [65] (black symbols) in comparison with the result of the GEF code (red line). The measured data were scaled to the height of the calculated spectrum. Since the experiment covers especially well the lower-energy range, a double-logarithmic presentation was chosen.

9 Correlations

Since the prompt-neutron spectra measured in the laboratory frame are the result of a convolution due to the emission under different angles from the moving fragments, they are not very sensitive to the yield of neutrons with very low energies in the frame of the fragments. Therefore, one may look for other experimental signatures that are more sensitive to specific features of the neutron emission. One of this signatures is the variation of the neutron multiplicity as a function of the angle between the directions of the emitted neutrons and the light fission fragment. Figure 13 shows the experimental data of ref. [66] in comparison with the result of the GEF code. The measured data are well reproduced over almost the complete angular range. The code underestimates the yield only very close to the direction of the light fragments. The two right-most points of the distribution correspond to angles of 5.7 and 9.9 degrees, corresponding to neutron energies in the fragment frame of 30 and 10 keV, respectively. Thus, these deviations can be explained by a slight underestimation of the neutron-absorption

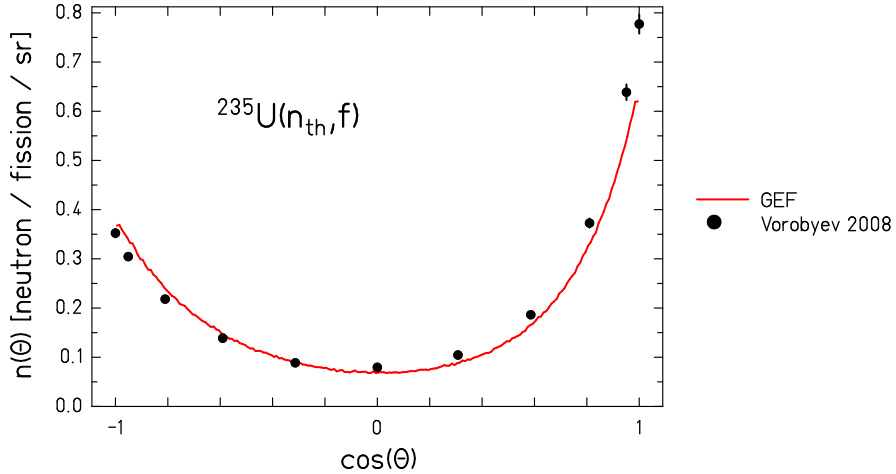


Figure 13: Variation of the prompt-neutron multiplicities versus the neutron direction relative to the light fission fragment. The result of the GEF code is compared with experimental data from ref. [66]. The nominal threshold in the experiment was 0.15 to 0.2 MeV.

cross sections in the very restricted low-energy regime below 50 keV. In the prompt-neutron spectrum, figures 10 to 12, these events appear at laboratory energies around 1 MeV due to the velocity of the emitting fragment. Here, no indication for this deviation can be seen. It seems that the description by the GEF code is very well suited for estimating the prompt-neutron spectra in the laboratory frame of heavy fissioning systems, which are most important for technical applications. The slight deviations in the angular distributions, figure 13, have practically no influence on the energy distribution of the prompt neutrons in the laboratory frame.

In the following we investigate the prompt-neutron yield as a function of the fission-fragment total kinetic energy. Figure 14 shows a comparison of the result of the GEF code with experimental data [67, 68] and a previous calculation of Kornilov [69]. The GEF calculation has been performed using Thomas-Fermi masses of Myers and Swiatecki [70] with recommended shell corrections and schematic even-odd fluctuations. The variation of the prompt-neutron yields from the light and the heavy fragment are assumed to be uncorrelated for a given split in Z and N .

The GEF calculation, in particular the slope, is rather close to the ex-

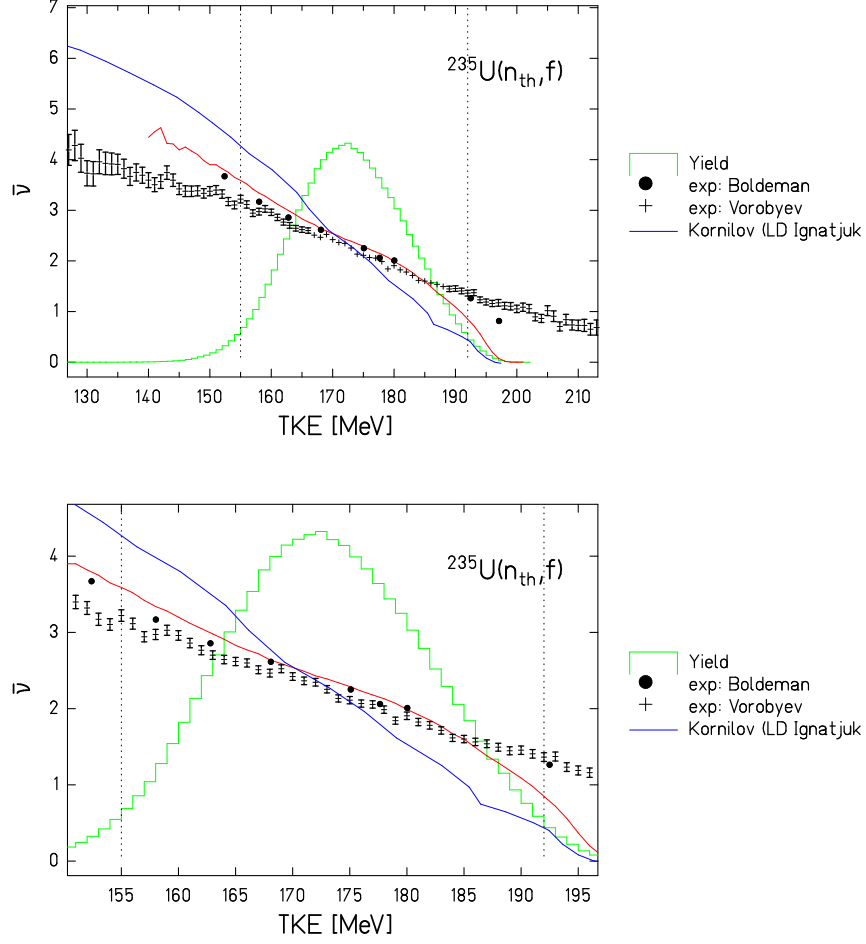


Figure 14: Mean prompt-neutron yield as a function of fission-fragment total kinetic energy for the system $^{235}\text{U}(n_{\text{th}},f)$. The experimental data of Boldeman et al. [67] and Vorobyev et al. [68] are compared with a calculation of Kornilov [69] (labeled as LD Ignatyuk) and the result of the GEF code (red line). The lower part shows a zoom on the central part of the TKE distribution. The green histogram shows the calculated pre-neutron TKE distribution in an arbitrary scale. The dotted vertical lines denote the region that contains 95 % of the fission events.

perimental data in the region between 155 MeV and 185 MeV. Also the

low-energy point of Boldeman et al. is well reproduced. For energies higher than 185 MeV all calculations, also the calculation of Kornilov, are appreciably below the experimental data. The cut-off of the neutron multiplicity slightly below 200 MeV is probably realistic, because even for the splits with the highest Q values the excitation energies of the fragments fall below the corresponding neutron separation energy for these high TKE values.

One should not forget that scattering phenomena can considerably disturb experimental data in regions of low yield as e.g. demonstrated in ref. [71]. Such processes would tend to flatten the variation of the measured prompt-neutron yield as a function of TKE. In this context it is interesting to note that the data of Boldeman et al. have a steeper slope than the data of Vorobyev et al., especially in the wings of the TKE distribution. The data of Vorobyev et al. even extend to TKE values, where there is hardly any yield expected, and neutrons are still seen above $TKE = 200$ MeV, where neutron emission is suppressed in the GEF code due to the Q -value limit. This puts also doubts on the data of Vorobyev et al. for total kinetic energies below 150 MeV, where the yield is low, and scattering phenomena may have an important influence.

The GEF code reproduces also well the measured mean prompt-neutron yields as a function of the total fission-fragment total kinetic energy for spontaneous fission of ^{252}Cf of ref. [72], see figure 15. The deviations at high TKE appear in a region of extremely low yield. They may be explained by a background of events with lower TKE due to random coincidences of fragment and prompt-neutron signals. Also the deviations at low TKE appear in a region with low yield. They may be caused at least to a part by incompletely measured TKE values due to scattering phenomena in the experiment.

One may speculate that the transport of a multitude of correlations along the fission process in the GEF code without any intermediate averaging has an important influence on correlations between different fission observables. These correlations might not have been fully considered in other models. The calculations with the GEF code do not give strong hints for additional phenomena like scission neutrons; the data of figures 14 and 15 can rather well be reproduced with the assumption of prompt-neutron emission from the fragments after scission, only.

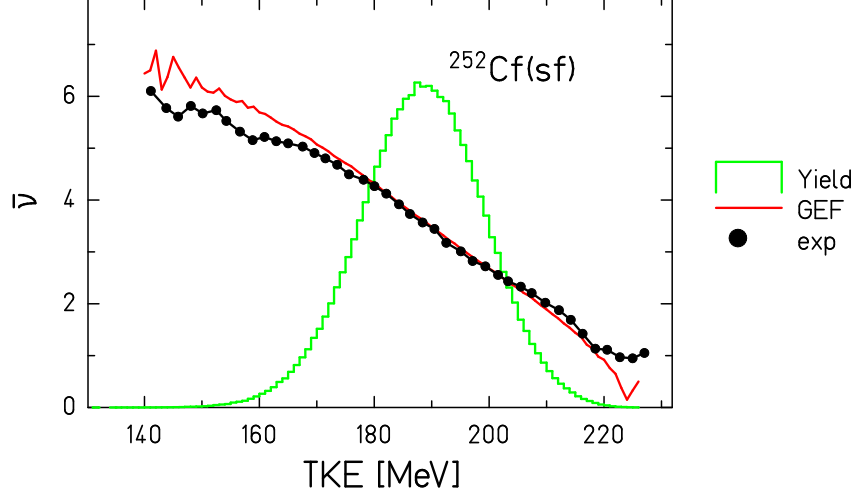


Figure 15: Mean prompt-neutron yield as a function of fission-fragment total kinetic energy for the system $^{252}\text{Cf}(\text{sf})$. The experimental data of Budtz-Jorgensen et al. [72] are compared with the result of the GEF code. The green histogram shows the calculated pre-neutron TKE distribution in an arbitrary scale.

10 Conclusion

The semi-empirical fission model, implemented in the GEF code, reproduces a large variety of observables with a good precision in a consistent way without further adjustment to specific fissioning systems with a unique parameter set. With this global approach one is able to predict several characteristic quantities of the fission process, e.g. the energy and multiplicity distribution of prompt-fission neutrons, without the need for specific experimental information of the respective system, e.g. measured mass-TKE distributions. All properties of the fission fragments that are considered in the code (e.g. nuclear charge, mass, excitation energy, angular momentum) are sampled in the corresponding multi-dimensional parameter space by a Monte-Carlo technique. Thus, all respective correlations are preserved. Moreover, correlations between all observables considered in the code are provided on an event-by-event basis. The measured prompt-neutron spectra in fission induced by thermal neutrons are very well reproduced by the GEF code

without any specific adjustment of the model for all systems that were investigated. It is to be expected that this agreement is preserved for fission induced by neutrons of higher energies. There are no systematic deviations which suggest the presence of scission neutrons in these cases.

Acknowledgment

This work has been supported by the NEA of the OECD (<http://www.oecd-neo.org/>) and by the EFNUDAT (<http://www.efnudat.eu/>) and by the ERINDA (<http://www.erinda.org/>) projects of the European Union.

Appendix 1:

Table of prompt-neutron energy spectra for selected target nuclei in thermal-neutron-induced fission from a GEF calculation with 10^7 events. (ZT , AT : Z and A of target nucleus, ν : mean prompt-neutron multiplicity. The energy values mark the lower limits of the energy bins: 0.1 denotes the energy bin from 0.1 to 0.2 MeV, 0.2 denotes the energy bin from 0.2 to 0.3 MeV, and so on.)

ZT	90	90	90	90	90	91	91	91	91	91	91	92	92	92	92
AT	227	228	229	230	231	232	229	230	231	232	233	234	232	233	234
ν	1.842	1.756	1.867	1.821	1.967	1.870	1.936	2.209	1.986	2.218	2.061	2.252	2.176	2.448	2.256
E/MeV															
0.1	444324	426755	455220	448311	487077	469515	450668	510774	463826	520045	488860	535803	492915	550934	513683
0.2	540910	521727	555930	544412	591977	570546	548624	621672	567519	634241	594689	651279	599111	669414	627286
0.3	600267	577175	614775	604618	655925	631488	608866	689381	629654	700157	659776	723810	667622	745040	696467
0.4	635802	612719	652253	640245	694446	669081	646644	734374	669485	746106	702405	767374	709157	792160	738806
0.5	659341	634956	676617	664393	718746	692557	672936	762269	693522	774141	727414	794743	741150	822472	771112
0.6	673702	646558	689549	676544	732917	703066	686543	780784	710271	791272	744939	813082	752561	842894	788794
0.7	676112	649784	690993	679054	732768	702428	693770	785388	715114	797290	747641	816319	763204	849100	795500
0.8	672919	646173	686161	672600	725381	696252	689380	784909	712130	793597	741792	811106	760599	849372	792824
0.9	660105	631088	674421	659831	713009	680649	681468	775242	701643	783980	731746	798866	753524	841255	781419
1	645182	616719	656482	642900	696090	661464	666680	760468	684786	767809	714149	782732	739238	826403	767858
1.1	629319	602070	639392	623678	676654	641723	649853	741819	669210	749427	696350	762281	722347	807211	751805
1.2	610401	582587	618527	604362	654938	620418	632528	722938	651451	726576	676875	742507	704170	787465	731380
1.3	589076	563093	598268	583150	632604	599334	613345	700590	628819	704927	654471	717928	684444	766429	709157
1.4	567253	541595	575499	560559	608049	574790	592337	678718	608426	681771	631113	692758	606090	741407	688676
1.5	546887	520886	551850	538889	582061	551113	571075	653935	584720	655510	607113	667494	638593	716569	662026
1.6	523011	497654	528351	515469	558757	525625	549515	627177	561238	630263	582926	637862	612781	689917	635789
1.7	498572	474209	504928	490734	530694	501785	524969	602052	537337	603195	558098	612166	589799	662736	609574
1.8	476913	451711	480124	466698	506112	477460	501867	574538	513548	574959	531859	582324	563236	633038	583389
1.9	453759	429864	457193	443692	481149	453797	477916	547969	489683	548390	505976	554898	538968	605886	557273
2	427855	408297	433375	420231	456132	428874	455102	521968	464604	522281	481226	527018	513031	579297	531711
2.1	407843	388017	411004	398536	430767	405517	432289	497148	441726	496636	456854	502144	489016	552360	505508
2.2	385612	366665	389169	378182	408444	384851	411240	470595	419162	471387	433134	476595	464701	525264	480229
2.3	365733	346379	367122	356310	386455	364047	388826	446256	397319	445829	411670	450343	441239	498566	457445
2.4	344691	328171	347735	337586	363944	342532	368678	422701	376290	423975	388363	426441	420353	473453	432824
2.5	325756	308408	327671	318602	345581	323066	348628	400468	356824	399502	367315	401802	389013	448233	409195
2.6	307542	291688	309353	300196	324849	305275	329114	378324	336668	377302	347024	381197	375543	424219	387381
2.7	289628	274644	292215	282725	306010	287433	310704	357471	317579	356185	328119	358047	354872	401758	367538
2.8	272846	258629	274494	265731	288193	270512	294374	337460	300667	335613	308417	339112	335193	379724	346705
2.9	256204	243659	258221	251234	271485	255158	277167	316908	283308	316726	290893	318093	316636	356341	327314
3	241316	227687	242642	236378	253732	238951	260169	299488	265886	298841	274376	298894	298488	337935	307456
3.1	227394	215177	228423	220398	236920	224700	244931	282396	250286	280827	258560	281106	281513	319552	289779
3.2	213811	201381	213865	207434	223804	211867	230238	264492	263862	242505	264642	264237	300097	273617	299814
3.3	199949	190022	200650	194480	209924	198217	216527	249297	221145	246993	227899	249295	249314	282885	257108
3.4	187699	177687	188261	183039	196886	185776	203623	234132	207426	232218	214688	233348	235002	266346	242457
3.5	175416	166261	177160	170699	184906	174723	192099	218534	194666	218150	200713	218598	220118	249998	227115
3.6	165459	156368	164654	160601	171918	163032	179905	206302	183777	203839	188394	204347	207243	235275	213445
3.7	154742	146463	155204	150643	161519	152475	167471	193032	171480	191901	176265	191895	194158	221424	200255
3.8	144811	136706	145482	141172	152030	143106	157579	180839	160832	179686	165665	179985	182604	206592	187745
3.9	135217	128167	135212	131434	141508	134117	147936	169924	150726	168246	155203	167997	170695	194570	176485
4	126043	119452	126812	123013	131657	124830	138456	159212	140578	157873	144524	158217	159976	182020	165139
4.1	117955	111390	118999	114546	123652	117143	129186	148252	132329	147198	136140	146962	150668	170574	154790
4.2	110108	104547	110953	107568	115764	109363	120763	138880	123515	137702	126831	137148	140276	160562	144345
4.3	103520	97239	103650	100752	107403	101988	113118	128938	115035	127814	118438	128459	131556	149099	135347
4.4	96531	90990	96727	93547	100525	95551	105802	121249	107918	119223	110540	119763	122684	140196	126578
4.5	89384	85291	90285	87308	94213	89036	98924	112273	100475	111297	103343	111422	115135	130992	118244
4.6	84235	79608	84222	81424	87592	83244	91423	105547	93475	104351	96343	103774	107419	122832	110624
4.7	78284	74380	78729	76355	81305	77274	85914	98142	88001	96567	90217	97070	100282	114015	102763
4.8	72729	68890	72907	70965	75748	72279	80792	91618	82048	89990	83870	90402	93140	106609	96993
4.9	67482	64196	68041	66273	70667	67381	74994	84879	76398	84559	78052	84371	87473	99653	90011
5	63097	60033	63403	61844	65867	62240	69439	78962	70937	78491	73029	78910	81755	92761	84114
5.1	58652	55695	59211	57795	61126	58408	65052	74216	66653	73533	67914	73516	75999	87033	78164
5.2	54655	52417	55510	53345	56501	54461	60676	68496	62357	68572	63418	68104	70844	80756	72692
5.3	51062	48340	50820	49336	52798	50614	56473	64312	57569	63527	58936	63485	66235	75610	67785
5.4	47620	45008	47753	46069	49034	46909	52794	59612	53829	59129	55219	58883	61607	70370	63335
5.5	44585	41728	44230	42579	46002	43970	49137	55326	49707	55164	51027	54492	57465	65866	58825
5.6	41044	38969	41377	40476	42641	40772	45842	51560	46202	51154	47775	51071	53832	61514	55129
5.7	38200	36254	38434	37253	39456	37496	42217	47963	42834	47476	44463	47491	49868	57172	50946
5.8	35817	33770	35530	34466	36793	35128	39449	44764	40028	44250	40958	43928	46158	52958	47570
5.9	32976	31573	33053	32221	34327	32577	36268	41483	37334	41062	38449	40783	43168	49339	44427

6	30666	28702	31090	30016	31730	30535	34169	38636	34954	37817	35386	38007	40239	45820	41003	45245
6.1	28602	26807	28695	27937	29240	28326	31802	35822	32228	35142	33285	35477	37285	42355	38395	42121
6.2	26560	25028	26605	26016	27416	26566	29740	33346	30075	33039	30799	32780	34893	39696	35495	39615
6.3	24457	23346	24544	24009	25580	24425	27370	30798	27881	30314	28634	30464	32418	36984	33382	36697
6.4	23136	21296	22838	22543	23430	22610	25262	28692	26130	28139	26527	28367	30103	34184	31189	33857
6.5	20955	20218	21275	20717	21657	21071	23668	26359	23779	26121	24921	26275	27713	31951	28849	31213
6.6	19568	18884	19984	19363	20266	19733	22097	24488	22354	24281	23175	24435	26016	29865	26647	29029
6.7	18110	17091	18051	17914	18823	18047	20246	22707	20788	22149	21271	22470	24072	27749	24808	27400
6.8	16629	15841	17093	16188	17282	16828	18872	21031	19167	20881	19848	20914	22263	25727	23211	25231
6.9	15457	14859	15686	15149	16288	15596	17414	19592	17982	19543	18192	19228	20745	24098	21170	23597
7	14547	13557	14645	14204	14791	14325	16396	18175	16499	17951	16761	17700	19200	22138	19733	21643
7.1	13515	12936	13359	13133	13793	13347	15208	16636	15461	16707	15582	16654	17901	20515	18465	20226
7.2	12577	11782	12361	12306	12802	12430	13853	15347	14227	15230	14848	15732	16719	19135	17075	18664
7.3	11490	10832	11580	11259	11729	11596	12888	14450	13310	14305	13829	14321	15642	17798	15719	17434
7.4	10774	10085	10716	10411	10911	10686	12125	13130	12364	13136	12461	13308	14487	16468	15036	16177
7.5	9780	9431	10061	9633	10234	9915	11337	12198	11349	12266	11770	12230	13300	15147	13600	14849
7.6	9167	8827	9163	8977	9455	9245	10328	11461	10647	11330	10880	11398	12611	14284	12684	14168
7.7	8432	7936	8602	8289	8666	8507	9690	10605	9845	10412	9826	10421	11523	13217	11836	13021
7.8	7854	7188	7808	7647	8028	7948	8810	9787	9021	9647	9252	9665	10522	12084	10977	12129
7.9	7333	6724	7368	7047	7447	7311	8202	8982	8531	8831	8656	9010	9751	11371	10081	11135
8	6826	6375	6644	6689	6900	6860	7615	8398	7785	8359	7927	8106	9177	10544	9306	10440
8.1	6323	6021	6312	5906	6282	6171	7227	7504	7189	7655	7429	7647	8321	9661	8467	9606
8.2	5721	5517	5837	5648	5838	5854	6577	7397	6720	7277	6706	7335	7929	8901	8093	8939
8.3	5379	5148	5240	5194	5474	5307	6132	6525	6073	6514	6206	6599	7297	8243	7405	8231
8.4	4953	4819	4964	4771	5036	4928	5668	6227	5720	5961	5822	6177	6607	7691	6925	7670
8.5	4643	4199	4607	4442	4684	4594	5179	5649	5305	5750	5507	5665	6253	7242	6462	6999
8.6	4340	3936	4214	3947	4351	4271	4748	5207	4911	5282	4947	5217	5733	6584	5995	6575
8.7	3891	3706	3831	3752	3947	3867	4348	4872	4442	4836	4684	4725	5350	6286	5560	6115
8.8	3566	3457	3615	3621	3654	3621	4156	4610	4102	4589	4382	4454	4900	5656	5217	5612
8.9	3263	3205	3275	3335	3340	3365	3814	4255	3977	4054	4042	4197	4533	5356	4751	5316
9	3093	2924	3096	3020	3171	3093	3504	3838	3571	3993	3638	3745	4190	4956	4413	4778
9.1	2905	2716	2763	2840	2974	2834	3285	3537	3335	3593	3385	3609	4027	4697	4121	4538
9.2	2596	2509	2656	2504	2578	2638	3041	3218	3106	3260	3215	3297	3544	4238	3783	4334
9.3	2405	2302	2494	2424	2386	2465	2799	2975	2873	3080	2922	3115	3405	3924	3429	3822
9.4	2168	2057	2274	2245	2297	2292	2569	2637	2551	2872	2682	2831	3158	3707	3285	3618
9.5	2062	1916	2051	2057	2179	2116	2351	2602	2510	2469	2513	2606	2933	3372	2978	3329
9.6	1955	1787	1923	1839	1936	1898	2272	2316	2219	2387	2325	2387	2722	3161	2862	3036
9.7	1765	1700	1789	1638	1803	1844	1989	2168	2120	2238	2183	2172	2502	2935	2571	2868
9.8	1593	1537	1670	1609	1761	1677	2008	2023	1882	2033	2041	2069	2301	2796	2371	2608
9.9	1535	1450	1459	1525	1546	1526	1770	1889	1813	1889	1875	1831	2120	2573	2219	2390
10	1382	1276	1374	1416	1397	1418	1669	1745	1672	1737	1719	1711	1942	2334	2043	2322
10.1	1265	1203	1306	1251	1293	1233	1511	1580	1529	1632	1602	1592	1852	2101	1874	2224
10.2	1218	1111	1249	1154	1228	1205	1405	1457	1508	1446	1531	1513	1734	1978	1783	1958
10.3	1076	1044	1104	1103	1148	1140	1271	1364	1322	1410	1373	1354	1559	1870	1598	1813
10.4	1023	985	999	942	1080	977	1207	1297	1168	1245	1263	1290	1465	1700	1522	1720
10.5	862	884	952	976	994	904	1106	1181	1164	1127	1136	1213	1363	1674	1324	1557
10.6	956	849	833	848	889	897	991	1076	1052	1101	1070	1124	1219	1624	1285	1492
10.7	828	761	785	769	765	827	971	1010	984	1026	1005	1069	1156	1386	1212	1361
10.8	754	732	776	696	725	752	882	888	861	926	892	939	1060	1306	1122	1280
10.9	694	607	684	641	695	727	857	814	784	853	848	906	966	1143	1046	1135
11	602	559	677	634	676	596	735	802	821	780	794	820	888	1087	887	1050
11.1	559	568	593	556	594	585	689	686	684	731	719	756	859	1010	902	999
11.2	497	526	581	509	505	600	681	708	689	679	651	722	772	948	831	954
11.3	487	468	539	448	526	529	557	643	581	608	667	648	705	810	795	846
11.4	453	411	453	423	506	482	585	586	568	549	543	567	660	804	678	753
11.5	404	373	439	424	455	425	525	533	517	563	532	529	656	674	672	710
11.6	393	370	383	392	435	384	456	459	493	510	476	514	610	683	612	687
11.7	321	323	345	343	382	329	427	465	414	482	481	484	535	637	565	620
11.8	333	329	329	303	330	344	389	403	369	432	458	450	516	577	558	577
11.9	348	292	289	302	306	302	388	391	356	424	354	379	510	533	504	544
12	281	281	298	282	310	289	344	355	361	395	339	378	453	506	440	499
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12.2	219	224	248	278	277	254	285	279	280	310	310	333	390	438	399	445
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12.4	207	201	203	238	209	210	247	235	274	244	279	277	324	332	356	351
12.5	188	171	193	186	199	188	237	263	233	233	239	206	286	359	328	358
12.6	171	169	182	169	175	182	245	206	204	245	233	240	254	322	296	321
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12.9	125	116	135	127	136	148	157	156	179	170	180	180	227	214	250	243
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13.6	69	68	90	79	78	82	69	111	102	103	102	123	113	141	118	142
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19.9	0	1	1	2	0	0	0	1	0	0	0	0	2	0	2	2
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7.3	16415	17793	17234	18396	20986	18747	21140	19386	21620	20323	22992	24853	23073	25270	23814	26475
7.4	15222	16339	15817	17034	19750	17479	19172	18051	19845	18877	21114	23620	21569	23384	21912	24791
7.5	13973	15254	14572	15691	18064	16392	18210	16948	18655	17618	19885	22015	20190	21808	20608	23363
7.6	13009	14153	13725	14642	17102	15335	16865	15407	17653	16182	18421	20328	18871	20222	19160	21352
7.7	12135	13186	12653	13523	15828	14110	15560	14325	16062	14955	17499	18928	17407	19105	17655	20008
7.8	11111	12054	11783	12812	14721	13341	14226	13420	14934	13973	15852	17490	16358	17497	16756	18616
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8	9735	10550	10379	10826	12423	11506	12478	11629	12952	12200	13781	15168	14066	15091	14481	16154
8.1	9170	9754	9450	10137	11665	10437	11771	11039	11808	11344	12882	14254	12981	14247	13432	14962
8.2	8427	9193	8855	9529	10814	9682	10661	10053	10965	10413	12126	13172	12022	13274	12518	13884
8.3	7692	8442	8218	8796	10050	9144	9996	9229	10266	9825	11053	12293	11278	12193	11671	13110
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8.5	6664	7242	7076	7516	8667	7872	8537	8032	8848	8565	9793	10525	9766	10681	10234	11304
8.6	6204	6586	6544	7003	8022	7207	7826	7495	8323	7849	8907	9980	8994	10057	9449	10450
8.7	5764	6207	6039	6489	7419	6856	7407	6741	7696	7318	8316	9171	8436	9326	8807	9867
8.8	5282	5759	5532	6033	6952	6291	6926	6517	7199	6907	7793	8442	7940	8397	8000	9140
8.9	4946	5258	5178	5483	6504	5924	6342	5954	6599	6374	7319	7906	7413	7989	7651	8577
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9.1	4203	4570	4440	4752	5490	4994	5334	5184	5767	5361	6378	6767	6188	6825	6535	7355
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19.1	4	4	5	1	3	1	1	4	3	5	2	4	4	4	6	7
19.2	2	0	3	1	4	3	2	4	0	8	5	3	4	2	6	6
19.3	7	2	1	2	2	1	6	1	1	4	3	5	0	6	3	4
19.4	0	0	3	2	1	3	2	2	1	5	3	5	5	3	3	6
19.5	1	2	5	2	1	0	2	2	5	3	0	3	3	3	5	3
19.6	0	3	3	1	5	2	2	1	2	2	6	3	4	2	1	4
19.7	1	2	1	2	4	2	5	0	1	4	2	1	1	6	1	0
19.8	2	2	1	3	0	5	0	3	3	0	3	3	2	4	2	3
19.9	0	1	1	1	2	3	1	4	5	1	1	3	5	2	0	6
20	2	1	1	2	1	2	0	1	2	2	1	3	2	3	0	1

7.2	26016	28616	28908	31427	29252	32023	29782	31426	33446	36069	33128	35991	32770	35486	31998	34550
7.3	24362	26407	27429	29527	27145	29586	27443	29584	31302	33571	31065	33594	30674	33176	29944	32486
7.4	22623	24838	25368	27480	25298	27474	25586	27498	29238	31550	28985	31505	28796	30903	27794	30166
7.5	21033	23131	23626	25191	23560	25744	23811	25415	27425	29401	26476	29120	26632	28418	26054	28015
7.6	19898	21484	22036	23514	22202	23614	22416	23745	25874	26914	25004	27367	24778	26694	24274	25992
7.7	18409	20077	20324	22176	20503	22312	20737	22307	23672	25619	23629	25580	23317	25031	22462	24283
7.8	16967	18751	18877	20719	19153	20621	19292	20906	22210	23810	21959	23762	21357	23187	20916	22775
7.9	15831	17482	17700	19536	17787	19335	17939	19242	20621	22182	20374	21929	20186	21951	19696	20934
8	14728	16180	16400	17957	16718	17950	16816	17867	19574	20629	19107	20703	18726	20230	18252	19641
8.1	13712	14971	15295	16514	15527	16552	15877	17015	18167	19477	17612	18974	17386	18692	17004	18344
8.2	12787	14122	14529	15627	14405	15506	14619	15387	17012	18064	16376	17732	16153	17816	15899	17108
8.3	11850	13090	13505	14321	13602	14535	13375	14405	15745	16795	15396	16814	15044	16420	15017	15893
8.4	10998	12169	12328	13550	12630	13453	12463	13545	14656	15716	14549	15553	14132	15316	13832	14585
8.5	10484	11282	11719	12556	11637	12295	11840	12586	13760	14656	13403	14363	13130	14254	12699	13676
8.6	9495	10455	10734	11671	10649	11911	10929	11727	12767	13754	12394	13424	12282	13352	12005	12783
8.7	9090	9713	10009	10822	10118	10956	10193	10747	11724	12784	11580	12431	11179	12269	11194	11895
8.8	8265	9069	9288	10071	9499	10324	9556	10068	11289	11769	10760	11588	10608	11360	10337	11366
8.9	7626	8357	8876	9447	8749	9490	8980	9426	10174	11074	10151	10919	9860	10694	9575	10113
9	7237	7860	8243	8800	8258	8863	8356	8900	9762	10139	9567	10098	9244	9997	9109	9595
9.1	6626	7344	7519	8296	7811	8256	7823	8118	8851	9764	8730	9481	8617	9336	8419	9103
9.2	6319	6919	7136	7624	7193	7632	7285	7619	8121	9143	8167	8803	8063	8725	7972	8413
9.3	5791	6326	6565	7064	6762	7096	6677	7132	7857	8206	7635	8119	7330	7899	7317	7850
9.4	5514	6027	6270	6684	6229	6673	6215	6601	7246	7788	7172	7566	6953	7354	6920	7401
9.5	5136	5596	5696	6222	5839	6115	5774	6206	6694	7201	6688	7028	6574	6973	6303	6666
9.6	4643	5090	5502	5782	5516	5694	5480	5549	6411	6686	6157	6444	6132	6541	5844	6230
9.7	4428	4814	5076	5260	4905	5346	5029	5364	5930	6286	5883	6253	5692	5913	5456	5846
9.8	4145	4440	4601	5032	4625	5041	4664	4885	5492	5808	5387	5804	5338	5678	5149	5364
9.9	3882	4253	4334	4608	4248	4622	4334	4532	5122	5407	5118	5317	4893	5221	4654	4911
10	3657	3729	4048	4289	3989	4359	3982	4301	4733	5078	4652	4964	4556	4845	4476	4636
10.1	3249	3532	3670	4083	3763	4076	3892	3958	4346	4720	4379	4713	4306	4485	4191	4341
10.2	3025	3343	3465	3776	3477	3709	3561	3678	4153	4367	4024	4325	3967	4156	3850	4068
10.3	2927	3177	3287	3412	3279	3545	3272	3421	3964	4095	3731	4159	3662	3959	3668	3736
10.4	2633	2835	3017	3253	3010	3191	3041	3187	3607	3713	3534	3763	3429	3654	3371	3645
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10.6	2309	2472	2587	2834	2633	2834	2658	2759	3179	3308	3071	3303	2981	3114	3003	3020
10.7	2175	2321	2478	2722	2459	2523	2578	2527	3035	3155	2917	3068	2815	2920	2804	2845
10.8	1951	2166	2358	2500	2275	2379	2260	2348	2721	2924	2720	2863	2564	2677	2557	2757
10.9	1945	2029	2156	2204	2231	2255	2089	2271	2599	2707	2464	2660	2291	2534	2452	2408
11	1736	1863	2058	2155	2028	2049	2021	2129	2354	2534	2317	2418	2319	2362	2211	2346
11.1	1558	1794	1892	1960	1903	1942	1909	1883	2157	2306	2136	2328	2099	2258	2037	2131
11.2	1496	1614	1727	1800	1795	1781	1698	1765	2117	2174	1952	2064	1906	2056	1891	1968
11.3	1420	1478	1572	1699	1631	1738	1630	1670	1919	2090	1856	1947	1938	1991	1827	1837
11.4	1305	1388	1488	1593	1470	1622	1561	1597	1778	1923	1784	1877	1723	1791	1630	1712
11.5	1169	1315	1378	1466	1380	1464	1344	1532	1707	1809	1605	1765	1615	1727	1519	1637
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11.7	1049	1142	1280	1292	1229	1303	1213	1274	1491	1545	1409	1566	1388	1404	1346	1454
11.8	969	1082	1124	1188	1142	1201	1258	1158	1292	1470	1268	1325	1336	1355	1246	1302
11.9	905	1007	1034	1173	1100	1113	1097	1090	1312	1362	1266	1220	1262	1246	1148	1229
12	839	972	1012	1037	968	1015	991	1013	1192	1211	1177	1207	1132	1163	1077	1109
12.1	774	856	879	1015	978	929	922	1016	1112	1165	1110	1131	1075	1089	1047	1052
12.2	724	755	851	919	866	892	886	833	1018	1048	1034	1093	952	1061	904	942
12.3	691	760	760	861	810	832	782	836	983	1027	979	979	892	907	891	950
12.4	660	700	757	791	750	803	671	782	946	901	892	969	839	876	870	892
12.5	582	654	728	777	678	772	763	733	856	907	841	890	780	835	797	865
12.6	572	618	635	705	638	662	658	633	829	826	750	807	729	773	760	735
12.7	530	532	546	658	584	618	702	605	709	776	676	708	680	689	684	689
12.8	497	500	565	591	553	586	551	633	724	736	619	678	658	709	618	670
12.9	403	518	541	525	512	573	501	512	614	710	621	642	619	626	544	575
13	402	457	516	522	492	498	459	498	585	631	606	609	528	583	549	537
13.1	392	416	461	431	440	493	475	454	598	574	554	491	530	547	484	526
13.2	372	400	410	460	432	452	438	420	555	563	458	492	460	519	492	507
13.3	358	325	387	384	424	396	394	368	500	487	460	498	472	462	430	421
13.4	309	333	344	341	373	360	371	373	459	466	467	472	430	450	385	387
13.5	286	324	308	366	312	313	317	348	443	473	411	412	387	426	376	397
13.6	251	294	293	333	321	318	307	344	411	442	436	411	390	399	348	376
13.7	244	305	306	310	299	317	279	287	383	356	380	338	362	381	335	357
13.8	247	283	280	328	275	300	286	307	355	333	323	307	292	345	305	314
13.9	228	249	249	271	294	288	259	266	263	340	281	320	261	319	315	286
14	199	196	259	249	237	250	244	227	304	279	325	300	290	277	289	267
14.1	187	224	202	212	223	186	215	220	289	234	260	266	278	268	249	236
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14.3	172	175	193	206	165	211	192	181	219	273	204	224	201	229	212	217
14.4	165	145	177	194	166	185	162	191	232	197	235	217	216	236	192	186
14.5	144	166	190	190	174	183	180	180	248	245	202	214	218	181	180	213
14.6	138	153	131	170	153	168	171	133	208	197	181	216	182	159	188	185
14.7	126	142	150	163	152	156	146	132	159	180	158	193	175	177	166	166

14.8	128	126	124	129	137	143	158	141	182	199	165	168	174	152	165	149
14.9	101	134	131	129	122	124	131	122	162	176	142	145	142	159	133	135
15	90	120	91	128	122	118	113	139	141	158	153	150	125	136	125	136
15.1	77	87	109	104	121	113	101	101	151	122	132	152	119	125	124	126
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15.3	87	82	88	90	99	108	97	106	131	133	126	117	123	121	111	128
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15.5	69	58	70	96	102	75	87	89	99	101	105	106	97	87	87	93
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16.4	39	44	52	48	51	51	45	41	61	53	63	49	55	62	47	47
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16.9	31	23	26	37	24	35	28	23	37	28	32	37	37	35	32	39
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19.9	1	4	1	3	0	3	3	3	3	2	4	6	1	5	2	5
20	2	3	0	5	6	5	5	1	3	4	2	5	3	7	2	6

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