

	*UPDATED TEST FOR ENDF FORMAT	FIXUP
	VERSION BASED ON RECENT FORMAT CHANGE	FIXUP
	*GENERAL IMPROVEMENTS BASED ON	FIXUP
	USER FEEDBACK	FIXUP
VERSION 99-2 (JUNE 1999)	*ASSUME ENDF-6, NOT 5, IF MISSING	FIXUP
	MF=1, MT-451.	FIXUP
	*FIXED CREATION OF SECTIONS	FIXUP
VERS. 2000-1 (FEBRUARY 2000)	*GENERAL IMPROVEMENTS BASED ON	FIXUP
	USER FEEDBACK	FIXUP
VERS. 2002-1 (MAY 2002)	*OPTIONAL INPUT PARAMETERS	FIXUP
	*SUMMATION RULES ARE DEFINED BASED	FIXUP
	ON CONTENTS OF TABLES.	FIXUP
VERS. 2004-1 (JAN. 2004)	*GENERAL UPDATE BASED ON USER FEEDBACK	FIXUP
	*INCREASED PAGE SIZE FROM 36000 TO	FIXUP
	60000 DATA POINTS.	FIXUP
VERS. 2005-1 (JAN. 2005)	*UPDATED MT CREATION TO ALLOW MAT =0	FIXUP
	INDICATING CREATE FOR ALL MATS.	FIXUP
VERS. 2007-1 (JAN. 2007)	*CHECKED AGAINST ALL ENDF/B-VII DATA	FIXUP
	*INCREASED PAGE SIZE FROM 60,000 TO	FIXUP
	600,000 DATA POINTS.	FIXUP
VERS. 2007-2 (OCT. 2007)	*ADDED MT=16 AS SUM MT=875 THRU 891	FIXUP
	*72 CHARACTER FILE NAMES	FIXUP
VERS. 2010-1 (Apr. 2010)	*Defining cross sections by summation	FIXUP
	to now mandatory - either build-in	FIXUP
	rules or by user input.	FIXUP
VERS. 2011-1 (March 2011)	*Added new MT # to allowed and	FIXUP
	summation rules.	FIXUP
VERS. 2012-1 (Aug. 2012)	*Corrected definition of MT=3 to avoid	FIXUP
	double counting of MT=18.	FIXUP
	*Extended incident particle list to	FIXUP
	include photon (ZA = 0).	FIXUP
	*Added CODENAME	FIXUP
	*32 and 64 bit Compatible	FIXUP
	*Added ERROR stops.	FIXUP
VERS. 2015-1 (Jan. 2015)	*Extended OUT9.	FIXUP
	*Replaced ALL 3 way IF Statements	FIXUP
		FIXUP
VERS. 2015-2 (Oct. 2015)	*Threshold Correction no longer	FIXUP
	allowed = TOO DANGEROUS!!!	FIXUP
VERS. 2017-1 (May 2017)	*Updated based on user feedback	FIXUP
	*Increased tables to 3,000,000.	FIXUP
	*All floating input parameters changed	FIXUP
	to character input + IN9 conversion.	FIXUP
	*Ignore attempts to "correct" reaction	FIXUP
	threshold = cannot be done for	FIXUP
	temperature dependent (MF=3) data.	FIXUP
VERS. 2017-2 (Oct. 2017)	*Updated to insure sharp edges for	FIXUP
	photon interaction cross sections	FIXUP
	MF=23.	FIXUP
	*Updated for ELECTRONS to create,	FIXUP
	MF/MT=23/501 = Total	FIXUP
	MF/MT=23/522 = Total ionization	FIXUP
	*Updated to define MF=26 and electron	FIXUP
	Cross Sections MT=526, 527, 528 as	FIXUP
	LEGAL MF/MT Combinations.	FIXUP
VERS. 2018-1 (Jan. 2018)	*Decreased PAGE size from 2,700,000	FIXUP
	to 1,800,000 - PAGE was too BIG for	FIXUP
	many computers - forcing the code	FIXUP
	to run VERY SLOWLY - smaller size	FIXUP
	improves running time.	FIXUP
	*Added on-line output for ALL ENDERROR	FIXUP
VERS. 2019-1 (June 2019)	*Additional Interpolation Law Tests	FIXUP
	*Print WARNING if ALL MTs in any	FIXUP
	evaluation DO NOT ALL EXTEND to the	FIXUP
	same Maximum Tabulated Energy =	FIXUP
	in this case data above the lowest	FIXUP
	common energy is identified as being	FIXUP
	UNRELIABLE.	FIXUP
VERS. 2019-2 (Oct. 2019)	*Corrected ERROR defining first point	FIXUP
	of each MT = first point was being	FIXUP
	ERRONEOUSLY skipped (due to an ERROR	FIXUP

- (4) ALLOW REACTION (MF=3, ANY MT) DELETION. ALL SPECIFIED REACTIONS WILL BE DELETED WHEN THE DATA IS READ FROM THE INPUT ENDF DATA FILE AND WILL NOT BE IN THE OUTPUT ENDF DATA FILE. WARNING DELETED REACTIONS MAY NOT BE USED TO DEFINE ANY RECONSTRUCTED REACTIONS (I.E. REACTIONS DEFINED BY SUMMING OTHER REACTIONS). SINCE DELETED REACTIONS ARE DELETED DURING READING IT IS AS IF THEY NEVER EXISTED AND IF ANY DELETED REACTION IS REQUIRED LATER TO DEFINE ANY SUM AN ERROR WILL RESULT. THE USER MAY SPECIFY THAT THE DELETION RULES ARE TO READ FROM INPUT (IMOPS(4)=1) OR THAT THE BUILT IN SUMMATION RULES ARE TO BE USED (MOPS(4)=2). AT THE PRESENT TIME THE BUILT-IN DELETION RULES ARE THAT NO SECTIONS SHOULD BE DELETED (THE USER MAY OVERRIDE THIS CONVENTION BY INPUT).
- (5) ALLOW REACTION (MF=3, ANY MT) RECONSTRUCTION BY SUMMING OTHER REACTIONS. IN ORDER TO OPTIMIZE THE RUNNING TIME OF THIS PROGRAM CARE SHOULD BE EXERCISED TO MINIMIZE THE NUMBER OF TIMES THAT EACH CONTRIBUTING CROSS SECTION MUST BE USED. THE USER MAY SPECIFY THAT THE SUMMATION RULES ARE TO BE READ AS INPUT (IMOPS(5)=1) OR THAT THE BUILT IN SUMMATION RULES ARE TO BE USED (IMOPS(5)=2). THE BUILT IN SUMMATION RULES ARE DESIGNED TO USE ENDF CONVENTIONS AND TO MINIMIZE THE NUMBER OF TIMES THAT EACH CROSS SECTION IS USED.
- (6) INSURE THAT ALL CROSS SECTIONS ARE NON-NEGATIVE (I.E. ARE ZERO OR POSITIVE). DURING READING ALL NEGATIVE CROSS SECTIONS WILL BE SET EQUAL TO ZERO AND TREATED AS SUCH DURING ALL SUBSEQUENT SUMMATIONS AND ENDF OUTPUT.
NOTE...THIS OPTION SHOULD NEVER BE USED WITH DATA CONTAINING BACKGROUND CROSS SECTIONS WHICH MAY BE NEGATIVE. ONLY AFTER THE RESONANCE CONTRIBUTION HAS BEEN ADDED TO THE BACKGROUND TO DEFINE THE ACTUAL CROSS SECTION IS IT VALID TO ELIMINATE NEGATIVE CROSS SECTIONS.
NOTE...THIS OPTION MAY BE USED TO DELETE NEGATIVE ELASTIC CROSS SECTIONS THAT MAY RESULT FROM RECONSTRUCTING CROSS SECTIONS FROM SINGLE LEVEL BREIT-WIGNER PARAMETERS. IF THE TOTAL CROSS SECTION IS THEN RECONSTRUCTED USING THE CORRECTED ELASTIC CROSS SECTION THE TOTAL WILL BE POSITIVE DUE TO THE CONTRIBUTIONS OF CAPTURE AND FISSION (THUS AVOIDING NUMERICAL INSTABILITY PROBLEMS DURING SELF-SHIELDING CALCULATIONS).
- (7) WITHIN EACH SECTION OF CROSS SECTIONS DELETE ENERGIES THAT ARE NOT IN ASCENDING ENERGY ORDER (ENERGY REPETITION IS O.K.)
- (8) WITHIN EACH SECTION OF CROSS SECTIONS ELIMINATE DUPLICATE POINTS (SUCCESSIVE POINTS WITH THE SAME ENERGY-CROSS SECTION).
- (9) TEST THAT ALL SECTIONS ARE IN ASCENDING MAT/MF/MT ORDER. IF NOT, NO CORRECTIVE ACTION WILL BE TAKEN, ONLY AN ERROR MESSAGE WILL BE OUTPUT.
- (10) CHECK MF/MT FOR EACH SECTION TO INSURE THAT THEY ARE DEFINED IN THE ENDF FORMAR MANUAL. IF THEY ARE NOT DEFINED AN ERROR MESSAGE IS PRINTED, BUT NO CORRECTIVE ACTION IS TAKEN.
- (11) ALLOW SECTIONS WHICH ARE NOT PRESENT IN THE ORIGINAL (INPUT) EVALUATION TO BE CREATED. NORMALLY THIS PROGRAM WILL ONLY RECONSTRUCT AND OUTPUT SECTIONS IF THE SECTION IS PRESENT IN THE ORIGINAL EVALUATION. THIS PROCEDURE IS FOLLOWED BECAUSE NORMALLY THE PROGRAM DOES NOT KNOW HOW TO DEFINE THE CONTENTS OF THE FIRST TWO LINES OF THE SECTION (E.G., Q-VALUE, TEMPERATURE, INITIAL AND FINAL STATES). THIS OPTION MAY BE USED TO ALLOW THE PROGRAM TO READ AND SAVE A TABLE DEFINING THE CONTENTS OF THE FIRST TWO LINES OF EACH SECTION TO BE CREATED.
NOTE...IF A SECTION IS PRESENT ANY COMMAND TO CREATE IT WILL BE IGNORED.
- (12) ALLOW ENERGY POINTS TO BE INSERTED. THE PROGRAM CAN READ UP TO 50, ENERGIES, MAT, MT AND USE LINEAR INTERPOLATION TO INSERT ENERGY POINTS INTO TABLES AS THEY ARE READ, E.G., INSERT AN ENERGY POINT AT THERMAL ENERGY (0.0253 EV). IF AN MAT AND/OR MT IS ZERO THIS IMPLIES = ALL - INSERT THE ENERGY IN ALL TABLES.
- (13) PUT ALLOW CROSS SECTIONS ON A UNIFORM ENERGY GRID = EACH SECTION (MT) OF CROSS SECTIONS WILL INCLUDE ALL ENERGIES WHICH APPEAR IN AT LEAST ONE SECTION OF DATA. PARAMETERS (MT=251 THROUGH 255) ARE NOT INCLUDED IN THE UNIFORM ENERGY GRID.

THE ORIGINAL EVALUATION (I.E. ENDF/B DATA READ) ONE SECTION OF DATA WILL BE OUTPUT, UNLESS THE SECTION HAS BEEN DELETED. THIS INCLUDES ANY SECTIONS WHICH ARE NOT PRESENT IN THE ORIGINAL EVALUATION, BUT THE USER INDICATES (BY INPUT) SHOULD BE CREATED.

THE PROGRAM WILL NOT OUTPUT ANY SECTION RECONSTRUCTED BY SUMMATION UNLESS THE CORRESPONDING SECTION (MT NUMBER) IS PRESENT IN THE ORIGINAL EVALUATION OR USER INPUT INDICATES SHOULD BE CREATED AND OUTPUT. THIS IS (A) BECAUSE THE PROGRAM CANNOT DEFINE THE PARAMETERS TO APPEAR ON THE FIRST TWO LINES OF THE SECTION, (B) TO AVOID OUTPUTTING TOO MUCH DATA WHICH THE USER MAY NOT BE INTERESTED IN.

(2) FOR ANY SECTIONS THAT DO NOT APPEAR IN THE ORIGINAL DATA THE USER MAY SPECIFY THAT THEY BE DEFINED BY SUMMATION. ANY SUCH SECTION MAY BE USED TO DEFINE SUBSEQUENT SUMS, BUT THE SECTION ITSELF WILL NOT BE OUTPUT (E.G. GENERALLY MT=27 AND 101 ARE NOT PRESENT IN EVALUATIONS. HOWEVER, THE BUILT-IN SUMMATION RULES OF THIS PROGRAM USES THE ENDF SUMMATION RULES TO DEFINE MT=27 AND 101, WHICH IN TURN ARE USED TO DEFINE THE NON-ELASTIC CROSS SECTION, MT=3. SECTIONS MT=27 AND 101 ARE NOT OUTPUT).

(3) ALL DATA IN FILE 3 AND 23 MUST BE LINEARLY INTERPOLABLE. IF THE DATA IS NOT LINEARLY INTERPOLABLE THIS PROGRAM WILL TERMINATE.

PROGRAM OPERATION

ALL MAT NUMBER ON AN ENDF TAPE ARE PROCESSED. EACH MAT IS TREATED SEPARATELY. WITHIN EACH MAT, EACH SECTION BEFORE MF=3 IS READ, CHECKED/CORRECTED (BASED ON INPUT OPTIONS) AND OUTPUT. WHEN MF=3 IS LOCATED ALL CROSS SECTIONS ARE READ, SECTIONS TO BE DELETED ARE DELETED, SECTIONS WHICH ARE NOT PRESENTED AND USER INPUT INDICATES SHOULD BE CREATED ARE CREATED, SECTIONS TO BE KEPT ARE CHECKED/CORRECTED (BASED ON INPUT OPTIONS) AND WRITTEN TO A SCRATCH FILE. NEXT, IF THE USER SPECIFIES THAT THEY SHOULD, SECTIONS ARE RECONSTRUCTED. FINALLY ALL CROSS SECTIONS (OLD AND NEW) ARE OUTPUT. WITHIN THE SAME MAT, EACH SECTION AFTER MF=3 IS READ, CHECKED/CORRECTED (BASED ON INPUT OPTIONS) AND OUTPUT.

MF=3

THE TREATMENT OF THE CROSS SECTIONS REQUIRES UP TO 4 PASSES FOR CROSS SECTIONS. IN THE PROGRAM THEY CORRESPOND TO SUBROUTINES PASS1, PASS2, PASS3 AND PASS4. THE ORIGINAL AND FINAL ENDF DATA FILES, 5 SCRATCH FILES AND 3 IN CORE ARRAYS ARE USED. OPERATIONS PERFORMED DURING EACH PASS ARE,

PASS1

READ ALL CROSS SECTIONS FROM ITAPE. DELETED ANY SECTIONS. CREATE ANY SECTIONS. CHECK/CORRECT THEM AND WRITE THEM TO SCRATCH FILE. DATA IS READ INTO ARRAY A, TRANSFERRED TO ARRAY C (AFTER EDITING) AND OUTPUT TO ISCR FROM ARRAY C.
ITAPE - UNIT ORIGINAL ENDF DATA IS READ FROM.
ISCR - SCRATCH UNIT THAT EDITED DATA IS WRITTEN ON.
TAB A - ARRAY INTO WHICH ORIGINAL DATA IS READ.
TAB C - ARRAY INTO WHICH EDITED DATA IS TRANSFERRED TO AND FROM WHICH IT IS WRITTEN TO ISCR.

PASS2

IF A UNIFORM ENERGY GRID IS REQUESTED IT IS CREATED DURING THIS PASS. FIRST ALL OF THE CROSS SECTIONS FROM PASS1 ARE READ AND A UNIFORM ENERGY GRID IS CREATED = ALL ENERGIES THAT ARE INCLUDED IN AT LEAST ONE SECTION (MT) OF CROSS SECTIONS.
ISCR A - SCRATCH UNIT CONTAINING UNIFORM ENERGY GRID.
ISCR B - SCRATCH UNIT CONTAINING UNIFORM ENERGY GRID.
ISCR C - SCRATCH UNIT THAT EDITED DATA IS READ FROM.

TABA	- ARRAY CONTAINING UNIFORM ENERGY GRID.	FIXUP
TABB	- ARRAY CONTAINING UNIFORM ENERGY GRID.	FIXUP
TABC	- ARRAY CONTAINING EDITED DATA.	FIXUP

THE UNIFORM ENERGY GRID ENDS UP ON ISCRB. NEXT EACH SECTION OF CROSS SECTIONS FROM PASS1 IS READ FROM ISCRB, INTERPOLATED TO THE UNIFORM ENERGY GRID AND OUTPUT TO ISCRB. FINALLY ISCRB AND ISCRB ARE SWITCH, SO THAT AT THE END OF THIS PASS THE DATA WILL AGAIN BE ON ISCRB (EXACTLY AS AT THE END OF PASS1), WITH UPDATED POINT COUNTS.

ISCRB	- SCRATCH UNIT THAT UNIFORM ENERGY GRID DATA IS WRITTEN ON.	FIXUP
ISCRB	- SCRATCH UNIT CONTAINING UNIFORM ENERGY GRID.	FIXUP
ISCRB	- SCRATCH UNIT THAT EDITED DATA IS READ FROM.	FIXUP
TABA	- ARRAY CONTAINING UNIFORM ENERGY GRID DATA.	FIXUP
TABB	- ARRAY CONTAINING UNIFORM ENERGY GRID.	FIXUP
TABC	- ARRAY CONTAINING EDITED DATA.	FIXUP

PASS3

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SUMMATION CROSS SECTIONS ARE DEFINED BY READING DATA FROM ISCRB AND MERGING THEM ONTO ISCRB. THE FIRST SECTION THAT CONTRIBUTES TO A SUM IS MERELY COPIED FROM C TO A. IF MORE SECTIONS WILL CONTRIBUTE TO THE SUM THE DATA IN A IS TRANSFERRED TO B, A SECTION OF DATA FROM C IS ADDED TO THE DATA IN B AND STORED IN A. THE CYCLE OF ADDED C AND B TO A, FOLLOWED BY MOVING A TO B IS CONTINUED UNTIL ALL CONTRIBUTING SECTIONS HAVE BEEN ADDED. THE SUM IS THEN COPIED FROM A TO D. IF NEWLY CONSTRUCTED SECTION IS REQUIRED FOR ANY LATER SUMMATIONS IT IS ALSO COPIED TO E. THE CYCLE OF ADDED SECTIONS FROM C AND B TO A IS REPEATED FOR EACH REQUIRED SUMMATION REACTION. IN ADDITION TO SECTIONS FROM C, AFTER THE FIRST SUMMATION SECTIONS MAY ALSO BE ADDED TO A FROM E (THE CONTRIBUTION OF NEW RECONSTRUCTED CROSS SECTIONS). WHEN ALL REQUIRED SECTIONS HAVE BEEN RECONSTRUCTED THE NEW SECTIONS WILL BE ON E AND THE ORIGINAL SECTIONS ON C.

ISCRB	- SCRATCH FILE FROM WHICH ORIGINAL DATA IS READ.	FIXUP
ISCRB	- SCRATCH FILE ONTO WHICH SUM FOR ONE SECTION IS WRITTEN.	FIXUP
ISCRD	- SCRATCH FILE ONTO WHICH ALL SUM CROSS SECTIONS ARE WRITTEN.	FIXUP
ISCRE	- SCRATCH FILE ONTO WHICH ALL SUM CROSS SECTIONS WHICH ARE REQUIRED FOR LATER SUMS ARE WRITTEN.	FIXUP
ISCRB	- UTILITY SCRATCH FILE USED TO CREATE SUM CROSS SECTIONS.	FIXUP
TABA	- ARRAY INTO WHICH SUMS ARE WRITTEN.	FIXUP
TABB	- ARRAY INTO WHICH PARTIAL SUMS ARE WRITTEN.	FIXUP
TABC	- ARRAY INTO WHICH ORIGINAL DATA IS READ.	FIXUP

PASS4

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CROSS SECTIONS ARE READ FROM ISCRB (ORIGINAL) AND ISCRD (NEW) AND ARE WRITTEN IN THE ENDF FORMAT ON OTAPE. THE BEGINNING OF EACH SECTION OF ORIGINAL DATA IS READ FROM ISCRB (TO DEFINE SECTION HEADER INFORMATION). IF THIS MT HAS NOT BEEN RECONSTRUCTED ON ISCRD THE ORIGINAL SECTION IS OUTPUT. IF THE SECTION HAS BEEN RECONSTRUCTED THE ORIGINAL SECTION IS SKIPPED AND THE NEW SECTION IS OUTPUT.

OTAPE	- OUTPUT DATA IN THE ENDF FORMAT.	FIXUP
ISCRB	- SCRATCH FILE FROM WHICH ORIGINAL DATA IS READ.	FIXUP
ISCRD	- SCRATCH FILE FROM WHICH NEW DATA IS READ.	FIXUP
TABC	- ARRAY INTO WHICH CROSS SECTIONS ARE READ FROM SCRATCH AND WRITTEN TO OTAPE	FIXUP

I/O FILE DEFINITIONS

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UNIT	DESCRIPTION	
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2	INPUT PARAMETERS.	FIXUP
3	OUTPUT REPORT.	FIXUP
10	ORIGINAL DATA IN THE ENDF FORMAT.	FIXUP
11	FINAL DATA IN THE ENDF FORMAT.	FIXUP
12	SCRATCH FILE	FIXUP
14	SCRATCH FILE	FIXUP
15	SCRATCH FILE	FIXUP

	34-44	I11	L2 OF SECTION TO BE CREATED	FIXUP
	45-48	I4	MAT OF SECTION TO BE CREATED	FIXUP
	49-51	I3	MT OF SECTION TO BE CREATED	FIXUP
SECOND	1-11	E11.4	C1 OF SECTION TO BE CREATED	FIXUP
LINE	12-22	E11.4	C2 OF SECTION TO BE CREATED	FIXUP
	23-33	I11	L1 OF SECTION TO BE CREATED	FIXUP
	34-44	I11	L2 OF SECTION TO BE CREATED	FIXUP
			*PAIRS OF LINES MAY BE IN ANY MAT/MT ORDER (E.G., THEY NEED NOT BE IN ASCENDING MAT/MT ORDER).	FIXUP FIXUP
			*UP TO 50 PAIRS OF LINES MAY BE USED TO DEFINE SECTIONS TO BE CREATED. THE LIST IS TERMINATED WHEN THE FIRST LINE OF A PAIR CONTAINS A ZERO (OR BLANK) MAT AND/OR MT.	FIXUP FIXUP FIXUP FIXUP
M-N			IF THE USER SPECIFIES THAT ENERGIES WHICH ARE NOT PRESENT IN THE ORIGINAL EVALUATION MAY BE INSERTED, ONE LINE MUST BE INPUT FOR EACH ENERGY TO BE INSERTED.	FIXUP FIXUP FIXUP
	1-11	E11.4	ENERGY TO BE INSERTED	FIXUP
	12-15	I4	MAT IN WHICH TO INSERT ENERGY = 0 = ALL	FIXUP
	16-18	I3	MT IN WHICH TO INSERT ENERGY = 0 = ALL	FIXUP
			*UP TO 50 (ENERGY, MAT, MT) LINES MAY BE USED. THE LIST IS TERMINATED BY A BLANK LINE.	FIXUP FIXUP
			*INPUT MAY BE IN ANY (ENERGY, MAT, MT) ORDER.	FIXUP FIXUP
			*ENERGY POINTS CAN ONLY BE INSERTED WITHIN THE ORIGINAL ENERGY RANGE OF A SECTION - THIS OPTION CANNOT BE USED TO EXTEND THE CROSS SECTION EITHER BELOW OR ABOVE THE ORIGINAL TABULATED ENERGY RANGE.	FIXUP FIXUP FIXUP FIXUP FIXUP

EXAMPLE INPUT NO. 1

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	(1) USE OPTIONS 1-11 (ALL OPTIONS, EXCEPT INSERT ENERGY POINTS)	FIXUP
	(2) DELETE MT=900 (FOR EXAMPLE PURPOSES ONLY)	FIXUP
	(3) DEFINE THE FOLLOWING MT NUMBERS TO BE RECONSTRUCTED,	FIXUP
	(MT= 4) = THE SUM OF MT= 51 THROUGH 91	FIXUP
	(MT=103) = THE SUM OF MT=700 THROUGH 718 (NOT 719)	FIXUP
	(MT=104) = THE SUM OF MT=720 THROUGH 738 (NOT 739)	FIXUP
	(MT=105) = THE SUM OF MT=740 THROUGH 758 (NOT 759)	FIXUP
	(MT=106) = THE SUM OF MT=760 THROUGH 778 (NOT 779)	FIXUP
	(MT=107) = THE SUM OF MT=780 THROUGH 798 (NOT 799)	FIXUP
NEW	(MT= 16) = THE SUM OF MT=875 THROUGH 891	FIXUP
	(MT=101) = THE SUM OF MT=102 THROUGH 114	FIXUP
	(MT= 18) = (MT=19) + (MT=20 AND 21) + (MT=38)	FIXUP
	(IF TOTAL FISSION, MT=18, IS NOT PRESENT, DEFINE IT BY SUMMING FIRST, SECOND, ETC. CHANCE - NOTE THAT THIS MUST BE DONE IN THIS ORDER, SINCE THE NEXT SUM INVOLVES USING MT=18.	FIXUP FIXUP FIXUP
	(MT= 27) = THE SUM OF MT= 18 AND 101	FIXUP
	(MT=101 RECONSTRUCTED ABOVE USED IN SUM).	FIXUP
	(MT= 3) = THE SUM OF (MT=4)+(MT=6-9)+(MT=16-17)+(MT=22-37)+ (MT=41-45)	FIXUP FIXUP
	(MT=4 AND 27 RECONSTRUCTED ABOVE USED IN SUM).	FIXUP
	(MT= 19) = (MT=18) - (MT=20 AND 21) - (MT=38)	FIXUP
	(DEFINE FIRST CHANGE FISSION BY SUBTRACTION TO ALLOW RESONANCE CONTRIBUTION FROM MT=18 TO BE INCLUDED IN MT=19).	FIXUP FIXUP
	(MT= 1) = THE SUM OF MT=2 AND 3	FIXUP
	(MT=3 RECONSTRUCTED ABOVE USED IN SUM).	FIXUP
	(4) THRESHOLD ENERGIES OF THE FOLLOWING MT NUMBERS WILL NOT BE TESTED OR CORRECTED.	FIXUP FIXUP
	MT=1, 4, 18, 19, 91, 103 THROUGH 114.	FIXUP
	(5) DEFINE MT=254 TO BE THE CAPTURE TO FISSION RATIO (MT=102/18)	FIXUP
	(6) CREATE MAT=1300/MT=254 - NOTE, THIS IS NECESSARY IN ORDER TO HAVE THE CAPTURE TO FISSION RATIO OUTPUT IN THE ENDF FORMAT	FIXUP FIXUP
	NOTE, ON THE FOLLOWING INPUT LINES THE CHARACTERS = () + , HAVE BEEN USED ONLY TO MAKE THE INPUT MORE READABLE - THESE CHARACTERS	FIXUP FIXUP

