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===== Sixpak
PROGRAM SIXPAK                               Sixpak
===== Sixpak
VERSION 92-1 (JANUARY 1992)                   Sixpak
VERSION 92-2 (FEBRUARY 1992) *INCREASED CORE ALLOCATION TO        Sixpak
                                ACCOMMODATE JEF AND EFF EVALUATIONS. Sixpak
VERSION 92-3 (APRIL 1992) *ADDED ADDITIONAL DATA TESTS.        Sixpak
VERSION 92-4 (SEPT. 1992) *CORRECTED KALBACH-MANN CALCULATIONS. Sixpak
                                *FOR PHOTON PRODUCTION OUTPUT MF=12 Sixpak
                                (MULTIPLICITY), MF=14 (ISOTROPIC Sixpak
                                ANGULAR DISTRIBUTIONS) AND MF=15 Sixpak
                                (SPECTRA) - PREVIOUSLY ONLY MF=15. Sixpak
                                *FIRST ORDER CORRECTIONS TRANSFORMING Sixpak
                                CENTER-OF-MASS SPECTRA TO LAB SYSTEM Sixpak
                                FOR OUTPUT IN MF=5 Sixpak
                                *CORRECTED ISOTROPIC ANGULAR Sixpak
                                DISTRIBUTION FLAG (LI) Sixpak
VERSION 94-1 (JANUARY 1994) *VARIABLE ENDF/B INPUT DATA FILENAME Sixpak
                                TO ALLOW ACCESS TO FILE STRUCTURES Sixpak
                                (WARNING - INPUT PARAMETER FORMAT Sixpak
                                HAS BEEN CHANGED) Sixpak
                                *CLOSE ALL FILES BEFORE TERMINATING Sixpak
                                (SEE, SUBROUTINE ENDIT) Sixpak
                                *INCREASED MAXIMUM TABLE SIZE FROM Sixpak
                                2000 TO 6000. Sixpak
VERSION 96-1 (JANUARY 1996) *COMPLETE RE-WRITE Sixpak
                                *IMPROVED COMPUTER INDEPENDENCE Sixpak
                                *ALL DOUBLE PRECISION Sixpak
                                *ON SCREEN OUTPUT Sixpak
                                *UNIFORM TREATMENT OF ENDF/B I/O Sixpak
                                *IMPROVED OUTPUT PRECISION Sixpak
VERSION 99-1 (MARCH 1999) *CORRECTED CHARACTER TO FLOATING Sixpak
                                POINT READ FOR MORE DIGITS Sixpak
                                *UPDATED TEST FOR ENDF/B FORMAT Sixpak
                                VERSION BASED ON RECENT FORMAT CHANGE Sixpak
                                *GENERAL IMPROVEMENTS BASED ON Sixpak
                                USER FEEDBACK Sixpak
VERSION 99-2 (JUNE 1999) *ASSUME ENDF/B-VI, NOT V, IF MISSING Sixpak
                                MF=1, MT-451. Sixpak
VERS. 2000-1 (FEBRUARY 2000) *GENERAL IMPROVEMENTS BASED ON Sixpak
                                USER FEEDBACK Sixpak
VERS. 2002-1 (JANUARY 2002) *CORRECTED ANGULAR DISTRIBUTION (MF=4) Sixpak
                                OUTPUT TO INSURE USED FIELDS ARE 0 Sixpak
                                (MAY 2002) *OPTIONAL INPUT PARAMETERS Sixpak
                                (NOV. 2002) *EXTENDED TO ALLOW CHARGED PARTICLE Sixpak
                                ANGULAR DISTRIBUTION IN MF=4 - Sixpak
                                WARNING - STRICTLY SPEAKING THIS IS Sixpak
                                NOT LEGAL, SINCE MF=4 IS SUPPOSED TO Sixpak
                                BE USED ONLY FOR NEUTRON ANGULAR Sixpak
                                DISTRIBUTIONS - BUT WHERE MT MAKES Sixpak
                                IT OBVIOUS THAT THE OUTGOING PARTICLE Sixpak
                                IS NOT A NEUTRON HOPEFULLY IT WILL Sixpak
                                NOT CAUSE A PROBLEM IF MF=4 IS USED Sixpak
                                FOR CHARGED PARTICLES. Sixpak
VERS. 2004-1 (MARCH 2004) *ADDED INCLUDE FOR COMMON Sixpak
                                *INCREASED MAXIMUM TABLE SIZE FROM Sixpak
                                6,000 TO 12,000. Sixpak
                                *ADDED DUMMY A FOR ELEMENTS Sixpak
                                *CORRECTED OUTPUT INTERPOLATON LAWS Sixpak

OWNED, MAINTAINED AND DISTRIBUTED BY        Sixpak
----- Sixpak
THE NUCLEAR DATA SECTION                   Sixpak
INTERNATIONAL ATOMIC ENERGY AGENCY         Sixpak
P.O. BOX 100                               Sixpak
A-1400, VIENNA, AUSTRIA                    Sixpak
EUROPE                                       Sixpak

ORIGINALLY WRITTEN BY                       Sixpak
----- Sixpak

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COLLABORATION

DEVELOPED IN COLLABORATION WITH.

*THE NATIONAL NUCLEAR DATA CENTER, BROOKHAVEN NATIONAL LAB

*THE NUCLEAR DATA SECTION, IAEA, VIENNA, AUSTRIA

*CENTRO TECNICO AEROSPACIAL, SAO JOSE DOS CAMPOS, BRAZIL

AS A PART OF AN INTERNATIONAL PROJECT ON THE EXCHANGE OF
NUCLEAR DATA

ACKNOWLEDGEMENT (VERSION 92-1)

THE AUTHOR THANKS SOL PEARLSTEIN (BROOKHAVEN NATIONAL LAB) FOR SIGNIFICANTLY CONTRIBUTING TOWARD IMPROVING THE ACCURACY AND COMPUTER INDEPENDENCE OF THIS CODE - THANKS, SOL

ACKNOWLEDGEMENT (VERSION 92-4)

THE AUTHOR THANKS BOB MACFARLANE (LOS ALAMOS) FOR SUGGESTING HOW TO PROPERLY OUTPUT THE PHOTON PRODUCTION DATA TO PUT IT INTO EXACTLY THE FORM NEEDED FOR USE IN PROCESSING CODES.

THE AUTHOR THANKS CHRIS DEAN (WINFRITH) FOR POINTING OUT ERRORS
IN THE EARLIER TREATMENT OF THE KALBACH-MANN FORMALISM AND IN
THE DEFINITION OF THE ISOTROPIC ANGULAR DISTRIBUTION FLAG (LI).

AUTHORS MESSAGE

THE COMMENTS BELOW SHOULD BE CONSIDERED THE LATEST DOCUMENTATION
INCLUDING ALL RECENT IMPROVEMENTS. PLEASE READ ALL OF THESE
COMMENTS BEFORE IMPLEMENTING AND USING THESE CODES.

AT THE PRESENT TIME WE ARE ATTEMPTING TO DEVELOP A SET OF COMPUTER INDEPENDENT PROGRAMS THAT CAN EASILY BE IMPLEMENTED ON ANY ONE OF A WIDE VARIETY OF COMPUTERS. IN ORDER TO ASSIST IN THIS PROJECT IT WOULD BE APPRECIATED IF YOU WOULD NOTIFY THE AUTHOR OF ANY COMPILER DIAGNOSTICS, OPERATING PROBLEMS OR SUGGESTIONS ON HOW TO IMPROVE THIS PROGRAM. HOPEFULLY, IN THIS WAY FUTURE VERSIONS OF THIS PROGRAM WILL BE COMPLETELY COMPATIBLE FOR USE ON YOUR COMPUTER.

PURPOSE

- 1) CHECK ALL DOUBLE-DIFFERENTIAL DATA (MF=6)
- 2) OUTPUT EQUIVALENT MF = 4, 5, 12, 14 AND 15 DATA.

DATA CHECKING

ALL OF THE ENDF/B-VI MF=6 DATA IS CHECKED - FOR DETAILS SEE BELOW.

THE MF=6 DATA IS NOT CORRECTED AND OUTPUT IN THE ENDF/B FORMAT.
IT IS MERELY CHECKED. IF ERRORS ARE FOUND IT IS UP TO THE USER
TO TAKE CORRECTIVE ACTION ON THE MF=6 DATA.

IN CONTRAST WHEN PROBLEMS ARE FOUND IN DATA WHICH WILL BE OUTPUT
IN THE ENDF/B FORMAT (MF=4, 5, 12, 14 AND 15), WHENEVER POSSIBLE

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CORRECTIVE ACTION WILL BE TAKEN.

=====
FURTHER CHECKS AND CORRECTIONS
=====
ONCE THE DATA HAS BEEN OUTPUT IN MF = 4, 5, 12, 14 AND 15 FORMATS
FURTHER CORRECTIVE ACTION CAN BE TAKEN AS FOLLOWS,

PROGRAM LEGEND
=====
CAN BE USED TO CORRECT ANGULAR DISTRIBUTIONS WHICH ARE NEGATIVE,
TO CONVERT FROM LEGENDRE COEFFICIENTS TO TABULATED ANGULAR
DISTRIBUTIONS AND GENERALLY PERFORM MORE EXTENSIVE TESTS OF
ALL MF=4 DATA.

PROGRAM EVALPLOT
=====
VERSION 92-1 AND LATER VERSIONS CAN PLOT ALL OF THE MF=4, 5 AND 15
DATA OUTPUT BY THIS CODE. EARLIER VERSIONS CAN PLOT MF=4 AND 5.
GRAPHICS IS AN EXCELLENT WAY TO CHECK THIS DATA.

PROGRAM PLOTTAB
=====
THIS IS A GENERAL PLOTTING PROGRAM AND THERE IS AN INTERFACE IN
THIS CODE TO PRODUCE OUTPUT FOR ANY MF=6 DATA IN THE PLOTTAB
INPUT FORMAT. THIS PROGRAM CAN BE USED TO CHECK ALL OF THE MF=6
DATA AS WELL AS THE EQUIVALENT MF=4, 5, 12, 14 AND 15 DATA - AS
WELL AS COMPARING THE ORIGINAL MF=6 AND EQUIVALENT DATA.

DATA OUTPUT
=====
THE ENDF/B MF=4, 5, 12, 14 AND 15 FORMATS ONLY ALLOW FOR NEUTRONS
INCIDENTS

THE ENDF/B MF=4 AND 5 FORMATS ONLY ALLOW FOR NEUTRONS OUTGOING.

THE ENDF/B MF=12, 14 AND 15 ONLY ALLOWS FOR PHOTONS OUTGOING.

THESE ARE THE ONLY COMBINATIONS OF DATA OUTPUT BY THIS CODE.

ALL OTHER COMBINATIONS OF INCIDENT AND OUTGOING PARTICLES ARE
CHECKED, BUT THE RESULTS CANNOT BE OUTPUT IN THE ENDF/B FORMAT.
HOWEVER, USING THE PLOTTAB INTERFACE BUILT INTO THIS CODE THIS
DATA CAN, AND HAS BEEN, OUTPUT AND CHECKED.

THE NEUTRON DATA IN MF=4 CAN BE IN THE FORM OF EITHER TABULATED
ANGULAR DISTRIBUTIONS OR LEGENDRE COEFFICIENTS.

THE NEUTRON (MF=5) OR PHOTON (MF=15) SPECTRA ARE BOTH IN EXACTLY
THE SAME FORMAT = ARBITRARY TABULATED FUNCTIONS - ENDF/B OPTION
LF=1.

ENDF/B DATA OUTPUT ORDER
=====
ENDF/B DATA IS OUTPUT IN ASCENDING MAT, MF, MT ORDER. IN ORDER TO
ALLOW THIS PROGRAM TO PRODUCE ALL OUTPUT IN A SINGLE PASS THROUGH
THE MF=6 DATA, OUTPUT FOR EACH (MAT, MT) IS OUTPUT TO SEPERATE
FILES FOR MF=4, 5, 12, 14 AND 15.

FOR SUBSEQUENT USE THE ENDF/B FORMATTED DATA OUTPUT BY THIS CODE
CAN BE MERGED TOGETHER USING PROGRAM MERGER (CONTAIN THE AUTHOR
OF THIS CODE FOR A COPY OF MERGER), E.G., MERGE MF=12, 14 AND 15
DATA IN ORDER TO THEN CALCULATE PHOTON PRODUCTION DATA OR MF=4
AND 5 CAN BE MERGED TOGETHER TO CALCULATE NEUTRON TRANSFER - OR
ALL OF THEM CAN BE MERGED TOGETHER TO PERFORM NEUTRON AND PHOTON
CALCULATIONS.

CORRELATED (MF=6) VS. UNCORRELATED (MF=4 AND 5) DATA
=====
THE ENDF/B DOUBLE DIFFERENTIAL = CORRELATED - DATA IN MF=6
REPRESENTS DATA IN THE FORM.

```

$F(E, EP, COS) = SIG(E) * Y(E) * G0(E, EP) * F(E, EP, COS)$ Sixpak
 SIG(E) = MF=3 CROSS SECTIONS Sixpak
 Y(E) = YIELD (MULTIPLICITY) Sixpak
 G0(E, EP) = ENERGY SPECTRUM Sixpak
 F(E, EP, COS) = ANGULAR DISTRIBUTION Sixpak
 IN A SITUATION WHERE YOU HAVE MONOENERGETIC AND MONODIRECTIONAL Sixpak
 NEUTRONS INCIDENT YOU WILL BE ABLE TO OBSERVE CORRELATION EFFECTS Sixpak
 IN THE NEUTRON SPECTRUM AND ANGULAR DISTRIBUTION. Sixpak
 EVEN IN SITUATIONS WHERE YOU HAVE A NARROW SPECTRUM OF NEUTRONS Sixpak
 THAT ARE HIGHLY DIRECTIONALLY ORIENTED YOU MAY BE ABLE TO OBSERVE Sixpak
 THESE CORRELATION EFFECTS, E.G., A NARROW 14 MEV FUSION SOURCE Sixpak
 INCIDENT ON THE FIRST WALL OF A CTR DEVICE. Sixpak
 FOR SUCH SITUATIONS USE OF THE CORRELATED (MF=6) DATA IS REQUIRED Sixpak
 IN CALCULATIONS. Sixpak
 HOWEVER, IN MANY APPLICATIONS WHERE THERE IS A BROAD SPECTRUM OF Sixpak
 NEUTRONS AND THE NEUTRON FLUX IS NOT HIGHLY DIRECTIONALLY Sixpak
 ORIENTED, THE NEUTRON MULTIPLICATION, SPECTRUM AND ORIENTATION Sixpak
 CAN BE FAIRLY ACCURATELY CALCULATED WITHOUT CONSIDERING Sixpak
 CORRELATION EFFECTS. Sixpak
 THE UNCORRELATED DATA PRODUCED BY THIS CODE REPLACES THE Sixpak
 CORRELATED DATA, Sixpak
 $F(E, EP, COS) = SIG(E) * Y(E) * G0(E, EP) * F(E, EP, COS)$ Sixpak
 BY THE UNCORRELATED DATA, Sixpak
 $F(E, EP, COS) = SIG(E) * Y(E) * G0(E, EP) * F0(E, COS)$ Sixpak
 BY INTEGRATING $G0(E, EP) * F(E, EP, COS)$ OVER SECONDARY ENERGY (EP) Sixpak
 TO DEFINE AN AVERAGE ANGULAR DISTRIBUTION, $F0(E, COS)$. Sixpak
 WHAT IS LOST IN THIS PROCESS IS THE CORRELATION BETWEEN EP AND COS Sixpak
 SO THAT IN A TRANSPORT CALCULATION ALL MOMENTS OF THE FLUX WILL Sixpak
 HAVE THE SAME SPECTRUM, $G0(E, EP)$ AND EACH WILL BE EFFECTED BY THE Sixpak
 AVERAGE ANGULAR DISTRIBUTION. Sixpak
 FOR APPLICATIONS TO HIGH ENERGY FUSION APPLICATIONS CORRELATED Sixpak
 DATA SHOULD BE USED. HOWEVER, FOR LOWER ENERGY APPLICATIONS, Sixpak
 SUCH AS FISSION REACTORS, IT SHOULD BE ADEQUATE TO USE THE Sixpak
 UNCORRELATED DATA - IN THIS CASE THE MOST IMPORTANT EFFECT Sixpak
 WILL BE THE OVERALL NEUTRON MULTIPLICATION AND SPECTRUM. Sixpak
 AN IMPORTANT CONSIDERATION IN DESIGNING THIS PROGRAM IS THAT Sixpak
 MANY COMPUTER CODES - DATA PROCESSING AND TRANSPORT CODES - Sixpak
 CANNOT USE THE CORRELATED (MF=6) DATA - NOR ARE THEY INTENDED Sixpak
 FOR HIGH ENERGY USE. FOR THESE CODES THE UNCORRELATED DATA Sixpak
 PRODUCED BY THIS CODE SHOULD BE ADEQUATE TO MEET THEIR NEEDS. Sixpak
 WARNING - IT CANNOT BE STRESSED ENOUGH THAT THE OUTPUT OF THIS Sixpak
 CODE SHOULD ONLY BE USED FOR LOW ENERGY APPLICATIONS - FAILURE Sixpak
 TO HEED THIS WARNING CAN LEAD TO COMPLETELY UNRELIABLE RESULTS. Sixpak
 ENDF/B FORMAT Sixpak
 ===== Sixpak
 THIS PROGRAM ONLY USES THE ENDF/B BCD OR CARD IMAGE FORMAT (AS Sixpak
 OPPOSED TO THE BINARY FORMAT) AND CAN HANDLE DATA IN ANY VERSION Sixpak
 OF THE ENDF/B FORMAT (I.E., ENDF/B-I, II, III, IV, V OR VI FORMAT). Sixpak
 IT IS ASSUMED THAT THE DATA IS CORRECTLY CODED IN THE ENDF/B Sixpak
 FORMAT AND NO ERROR CHECKING IS PERFORMED. IN PARTICULAR IT IS Sixpak
 ASSUMED THAT THE MAT, MF AND MT ON EACH LINE IS CORRECT. SEQUENCE Sixpak
 NUMBERS (COLUMNS 76-80) ARE IGNORED ON INPUT, BUT WILL BE Sixpak
 CORRECTLY OUTPUT ON ALL LINES. THE FORMAT OF SECTION MF=1, MT=451 Sixpak
 AND ALL SECTIONS OF MF=6 MUST BE CORRECT. THE PROGRAM SKIPS ALL Sixpak
 OTHER SECTIONS OF DATA AND AS SUCH IS INSENSITIVE TO THE FORMAT Sixpak

OF ALL OTHER SECTIONS.

Sixpak

Sixpak

CONTENTS OF OUTPUT

Sixpak

=====

Sixpak

5 ENDF/B FORMATTED OUTPUT FILES ARE PRODUCED FOR NEUTRON INCIDENT DATA,

Sixpak

Sixpak

1) ENDFB.MF4 - ANGULAR DISTRIBUTIONS AND LEGENDRE COEFFICIENTS FOR NEUTRONS

Sixpak

Sixpak

2) ENDFB.MF5 - TABULATED NEUTRON ENERGY SPECTRA

Sixpak

Sixpak

3) ENDFB.M12 - PHOTON EMISSION MULTIPLICITY

Sixpak

Sixpak

4) ENDFB.M14 - PHOTON EMISSION ANGULAR DISTRIBUTIONS (ALWAYS ISOTROPIC)

Sixpak

Sixpak

5) ENDFB.M15 - TABULATED PHOTON EMISSION SPECTRA

Sixpak

Sixpak

EMITTED PARTICLE YIELD

Sixpak

=====

Sixpak

NEUTRONS

Sixpak

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Sixpak

IN MF=6 THE YIELD FOR EACH REACTION IS THE ACTUAL MULTIPLICITY OF THE REACTION, E.G., (N,2N) = 2. IN USING MF=4 AND 5 DATA THE ENDF/B CONVENTION IS THAT THE MULTIPLICITY IS IMPLIED BY THE MT NUMBER, E.G., MT=16 = (N,2N) = 2.

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THE ONLY EXCEPT IN ENDF/B-VI IS MT=201 = TOTAL NEUTRON PRODUCTION WHERE AN ACTUAL ENERGY DEPENDENT YIELD IS INCLUDED IN MF=6. HOWEVER, IN THIS CASE THE MF=3 CROSS SECTION INCLUDES THE MULTIPLICITY (S. PEARLSTEIN, PRIVATE COMMUNICATION, JAN. 1992), $SIG(MT=201) = 2 * SIG(N,2N) + 3 * SIG(N,3N) \dots \text{ETC.}$

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Sixpak

Sixpak

SO THAT FOR ALL ENDF/B-VI DATA AS OF JANUARY 1992 THE MF=4 AND 5 DATA OUTPUT BY THIS CODE CAN BE USED IN CONJUNCTION WITH THE MF=3 CROSS SECTIONS - WITHOUT ANY REFERENCE TO THE MF=6 YIELD.

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Sixpak

Sixpak

Sixpak

PHOTONS

Sixpak

=====

Sixpak

UNLIKE THE NEUTRONS WHERE WITH ONLY ONE EXCEPTION (MT=201) THE MF=6 YIELD IS ENERGY INDEPENDENT, IN THE CASE OF PHOTON EMISSION ALMOST ALL OF THE PHOTONS HAVE AN ENERGY DEPENDENT YIELD.

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Sixpak

THIS PROGRAM WILL OUTPUT THE PHOTON MULTIPLICITY IN MF=12 AND INDICATE THAT THERE IS A NORMALIZED DISTRIBUTION IN MF=15 (LF=1 IN MF=12).

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Sixpak

THIS PROGRAM WILL OUTPUT THE NORMALIZED PHOTON SPECTRA IN MF=15. CONTINUOUS ENERGY SPECTRA AND DISCRETE PHOTONS WILL ALL BE OUTPUT AS NORMALIZED SPECTRA.

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Sixpak

THIS PROGRAM WILL ALSO OUTPUT MF=14 PHOTON ANGULAR DISTRIBUTION DATA, ALWAYS USING THE ISOTROPIC FLAG TO MINIMIZE OUTPUT.

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Sixpak

Sixpak

WARNING OF ENERGY DEPENDENT YIELD

Sixpak

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Sixpak

THIS PROGRAM WILL PRINT A WARNING MESSAGE IF A SECTION OF DATA BEING OUTPUT IN THE ENDF/B FORMAT HAS AN ENERGY DEPENDENT MF=6 YIELD AND THE EMITTED PARTICLE IS A NEUTRON - SINCE THE ENDF/B CONVENTION IS THAT FOR EACH MT NUMBER THE MULTIPLICITY IS IMPLIED WE DO NOT EXPECT AN ENERGY DEPENDENT MULTIPLICITY FOR NEUTRON EMISSION.

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Sixpak

USING THE OUTPUT

Sixpak

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Sixpak

NOTE, THAT IN USING THIS DATA, STARTING FROM THE RELATIONSHIP,

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Sixpak

$F(E,EP,COS) = SIG(E) * Y(E) * G0(E,EP) * F0(E,COS)$

Sixpak

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Sixpak

Sixpak

Sixpak

Sixpak

USING THE ENDF/B CONVENTION THAT THE MULTIPLICITY IS EITHER IMPLIED BY THE MT NUMBER (E.G., MT=16 = N,2N - MULTIPLICITY = 2) OR INCLUDED IN THE CROSS SECTION (E.G., MT=201 = TOTAL NEUTRON PRODUCTION) ALL THE INFORMATION REQUIRED FOR A CALCULATION IS AVAILABLE IN,

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Sixpak

MF=3 - SIG(E) Sixpak
 MF=4 - F0(E,COS) - FOR OUTGOING NEUTRONS Sixpak
 MF=5 - G0(E,EP) - FOR OUTGOING NEUTRONS Sixpak
 MF=12 - Y(E) - FOR OUTGOING PHOTONS Sixpak
 MF=14 - F0(E,COS) - FOR OUTGOING PHOTONS (ALWAYS ISOTROPIC) Sixpak
 MF=15 - G0(E,EP) - FOR OUTGOING PHOTONS Sixpak
 Sixpak

DOCUMENTATION Sixpak
 ===== Sixpak
 ONLY SECTIONS OF MF=4, 5, 12, 14, 15 ARE OUTPUT ON A ENDF/B FILE. Sixpak
 THE ONLY DOCUMENTATION IS THE ENDF/B TAPE LABEL (FIRST RECORD OF Sixpak
 EACH FILE) WHICH IDENTIFIES THE DATA AS SIXPAK OUTPUT. Sixpak
 Sixpak

REACTION INDEX Sixpak
 ===== Sixpak
 THIS PROGRAM DOES NOT USE THE REACTION INDEX WHICH IS GIVEN IN Sixpak
 SECTION MF=1, MT=451 OF EACH EVALUATION. Sixpak
 Sixpak

SECTION SIZE Sixpak
 ===== Sixpak
 ALL OF THE DATA IN ENDF/B-VI, MF=6 ARE QUITE SMALL TABLES. AS SUCH Sixpak
 THIS PROGRAM ONLY ALLOWS TABLES OF UP TO 12000 POINTS (12,000 X, Sixpak
 Y VALUES). THIS SIZE IS MORE THAN ADEQUATE TO HANDLE ALL OF THE Sixpak
 CURRENT ENDF/B-VI DATA, AND IT CAN BE EASILY INCREASED TO HANDLE Sixpak
 ANY NEWER DATA AS IT BECOMES AVAILABLE. Sixpak
 Sixpak

PLEASE CONTACT THE AUTHOR IF YOU HAVE AN EVALUATION WHICH EXCEEDS Sixpak
 THIS LIMIT. Sixpak
 Sixpak

SELECTION OF DATA Sixpak
 ===== Sixpak
 THE PROGRAM SELECTS DATA TO BE PROCESSED BASED ON MAT/MT RANGES Sixpak
 (MF=6 ASSUMED). THIS PROGRAM ALLOWS UP TO 100 MAT/MT RANGES TO BE Sixpak
 SPECIFIED BY INPUT PARAMETERS. THE PROGRAM WILL ASSUME THAT THE Sixpak
 ENDF/B TAPE IS IN MAT ORDER. THE PROGRAM WILL TERMINATE EXECUTION Sixpak
 WHEN A MAT IS FOUND THAT IS ABOVE ALL REQUESTED MAT RANGES. Sixpak
 Sixpak

PROGRAM OPERATION Sixpak
 ===== Sixpak
 EACH SECTION (MT) OF MF=6 DATA IS SUBDIVIDED INTO SUBSECTIONS - Sixpak
 ONE SUBSECTION FOR EACH EMITTED PARTICLE. Sixpak
 Sixpak

EACH SUBSECTION OF DATA IS CONSIDERED SEPARATELY. EACH SUBSECTION Sixpak
 OF ENDF/B MF=6 DATA TO PROCESS IS IN THE FORM, Sixpak
 Sixpak

$$F(E,EP,COS) = SIG(E)*Y(E)*G0(E,EP)*F(E,EP,COS)$$
 Sixpak
 Sixpak

SIG(E) = MF=3 CROSS SECTIONS Sixpak
 Y(E) = YIELD (MULTIPLICITY) Sixpak
 G0(E,EP) = ENERGY SPECTRUM Sixpak
 F(E,EP,COS) = ANGULAR DISTRIBUTION Sixpak
 Sixpak

G0(E,EP) = 1 WHEN INTEGRATED OVER EP (SECONDARY ENERGY) Sixpak
 G0(E,EP)*F(E,EP,COS) = 1 WHEN INTEGRATED OVER EP AND COS Sixpak
 Sixpak

THIS PROGRAM WILL DEFINE THE ZEROth ORDER MOMENTS OF THE Sixpak
 ENERGY AND ANGULAR DISTRIBUTIONS, Sixpak
 Sixpak

G0(E,EP) = G0(E,EP)*F(E,EP,COS) INTEGRATED OVER COS Sixpak
 F0(E,COS) = G0(E,EP)*F(E,EP,COS) INTEGRATED OVER EP Sixpak
 Sixpak

FOR NEUTRON INDUCED REACTIONS THE ENDF/B FORMATTED OUTPUT WILL BE Sixpak
 Sixpak

F0(E,COS)- IN ENDFB.MF4 FOR NEUTRONS OUT OF A REACTION Sixpak
 G0(E,EP) - IN ENDFB.MF5 FOR NEUTRONS OUT OF A REACTION Sixpak
 - IN ENDFB.M15 FOR PHOTONS OUT OF A REACTION Sixpak
 Sixpak

FOR NEUTRONS INCIDENT AND NEUTRONS EMITTED THIS DATA WILL BE Sixpak
 OUTPUT IN MF=4 AND 5 FORMATS. Sixpak
 Sixpak

ENDF/B-VI ALLOWS NEW INTERPOLATION LAWS FOR CORRESPONDING POINT AND UNIT BASE TRANSFORMATION INTERPOLATION. NONE OF THESE NEW INTERPOLATION LAWS ARE USED IN THE ENDF/B-VI LIBRARY AS OF JANUARY 1992 TO INTERPOLATE IN SECONDARY ENERGY OR COSINE. THEREFORE THIS PROGRAM CAN PERFORM ALL OF THE REQUIRED INTEGRALS OVER SECONDARY ENERGY AND/OR COSINE USING ONLY THE OLDER INTERPOLATION CODES. THIS PROGRAM ONLY PERFORMS INTEGRALS FOR EACH INCIDENT ENERGY, SO THAT INTERPOLATION IN INCIDENT ENERGY IS NOT PERFORMED BY THIS PROGRAM.

FOR CONSISTENCY WITH EARLIER VERSIONS OF ENDF/B IN CREATING THE ENDF/B OUTPUT, IF ANY INPUT INTERPOLATION LAW IS NOT IN THE RANGE 1-5, IT WILL FIRST BE TESTED TO SEE IF MOD(10) IT IS IN THIS RANGE, FINALLY IF EVEN THIS DOESN'T WORK IT IS SET EQUAL TO 2 (LINEARLY INTERPOLATION). THIS METHOD WILL EFFECTIVELY REPLACE CORRESPONDING POINT AND UNIT BASE TRANSFORMATION BY THE CLOSEST RELATED INTERPOLATION LAW 1 THROUGH 5 - AGAIN NOTE, AS OF JANUARY 1992 NONE OF THESE NEW LAWS ARE USED IN ENDF/B-VI. IF THIS MUST BE DONE FOR INTERPOLATION IN SECONDARY ENERGY OR COSINE AN ERROR MESSAGE WILL BE PRINTED - SINCE THIS WOULD EFFECT THE ACCURACY OF THE INTEGRALS PERFORMED BY THIS PROGRAM. IF THIS MUST BE DONE FOR INCIDENT ENERGY NO MESSAGE IS PRINTED - SINCE THIS WILL NOT EFFECT THE ACCURACY OF THE INTEGRALS PERFORMED BY THIS PROGRAM.

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LEGENDRE COEFFICIENTS

=====

LEGENDRE COEFFICIENTS IN NORMAL FORM ARE CHECKED TO INSURE
THEY ARE IN THE RANGE -1 TO +1 = THE LEGENDRE EXPANSION OF A
DELTA FUNCTION AT COS=+1 OR -1 - COEFFICIENTS SHOULD NOT
EXCEED WHAT YOU GET FROM A DELTA FUNCTION.

CREATING ENDF/B OUTPUT

=====

THIS PROGRAM HAS NOT BEEN TESTED ON OTHER DATA LIBRARIES, E.G.,
JEF, JENDL, ETC.

ISOTROPIC PHOTON EMISSION

EITHER TABULATED OR LEGENDRE COEFFICIENTS

IN ORDER TO OBTAIN CORRECT ENDF/B OUTPUT THE REPRESENTATION
MUST BE THE SAME FOR ALL INCIDENT ENERGIES = MF=4 DATA CAN ONLY
BE TABULATED OR LEGENDRE OVER THE ENTIRE ENERGY RANGE.

THE YIELD INCLUDED WITH EACH SECTION OF DATA IS NOT USED FOR
OUTPUT FOR NEUTRONS, BUT IS INCLUDED IN THE OUTPUT FOR PHOTONS.
IN ALL CASES THE ANGULAR DISTRIBUTIONS AND SPECTRA OUTPUT ARE
NORMALIZED TO UNITY.

NO OUTPUT - INCIDENT NEUTRON - EMITTED PHOTON OR NEUTRON
REACTIONS ARE NOT EXPECTED.

FOR EACH INCIDENT, ENERGY DISCRETE AND CONTINUOUS EMISSION SPECTRA CANNOT BE MIXED TOGETHER - THEY MUST BE ALL EITHER DISCRETE OR CONTINUOUS. IF DISCRETE EMISSION IS GIVEN ONLY 1 SECONDARY ENERGY (NEP=1) MAY BE GIVEN = A NORMALIZED DISTRIBUTION FOR A SINGLE DISCRETE EMISSION ENERGY. ALL OF THE ENDF/B-VI DATA AS OF JANUARY 1992 CONFORM TO THESE LIMITATIONS.

DISCRETE PHOTONS ARE OUTPUT IN MF=15 AS 3 POINT DISTRIBUTIONS
WITH SECONDARY ENERGY POINTS AT EP-DEP, EP, EP+DEP, WHERE
DEP=0.001*EP. THE VALUES AT EP-DEP AND EP+DEP ARE 0.0, AND
AT EP THE VALUE IS 1000.0/EP TO NORMALIZE THE DISTRIBUTION.

NO LIMITATION ON REPRESENTATIONS.

NO LIMITATION ON REPRESENTATIONS.

NO OUTPUT - INCIDENT NEUTRON - EMITTED PHOTON OR NEUTRON
REACTIONS ARE NOT EXPECTED.

NO OUTPUT - INCIDENT NEUTRON - EMITTED PHOTON OR NEUTRON
REACTIONS ARE NOT EXPECTED.

PREPRO 2004

NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA
 FOR (N,D) 2N,P.

LAW=7
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 FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER,

- 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY ENERGIES.
- 2) LINEAR = FOR EACH INCIDENT COSINE THE INTERPOLATION LAW BETWEEN SECONDARY ENERGIES MUST BE LINEAR.

THESE 2 PRESENTATIONS ARE THE ONLY ONES PRESENTED IN ENDF/B-VI AS OF JANUARY 1992 - SO THIS PROGRAM CAN TRANSLATED ALL LAW=7 DATA FOR ENDF/B-VI.

LABORATORY VS. CENTER-OF-MASS SYSTEM
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 IN MANY CASES PEOPLE ASSUME THAT FOR HEAVY (HIGH ATOMIC WEIGHT) MATERIALS THE CENTER-OF-MASS AND LAB SYSTEMS ARE ALMOST IDENTICAL, SINCE IN THIS CASE THE CENTER-OF-MASS ENERGY WILL BE MUCH SMALLER THAN THE INCIDENT ENERGY. FOR A PROCESS SUCH AS ELASTIC SCATTERING WHERE FOR HEAVY MATERIALS THE SECONDARY ENERGY, EP, WILL ALWAYS BE A LARGE FRACTION OF THE INCIDENT ENERGY, THIS ASSUMPTION IS VALID. HOWEVER, FOR THE TYPICAL REACTIONS INCLUDED IN MF=6 THIS IS NOT ALWAYS TRUE - IN MANY OF THESE CASES THE SECONDARY ENERGY CAN EXTEND ALL THE WAY DOWN TO ZERO, AND IN PARTICULAR IT CAN BE SMALL COMPARED TO THE CENTER-OF-MASS ENERGY - WHICH MAKES THE TRANSFORMATION FROM CENTER-OF-MASS TO LAB IMPORTANT. THEREFORE GENERALLY TO TREAT MF=6 DATA WE MUST CONSIDER THIS TRANSFORMATION.

THE FOLLOWING DISCUSSING ONLY APPLIES TO SPECTRA THAT MAY BE OUTPUT IN MF=5 = ONLY DATA FOR NEUTRONS INCIDENT AND EMITTED - IN PARTICULAR THE FOLLOWING DEFINITIONS ARE NOT GENERAL - THEY ARE ONLY VALID FOR INCIDENT AND EMITTED NEUTRONS.

DOUBLE DIFFERENTIAL DATA IN MF=6 MAY BE GIVEN IN EITHER THE LAB OR C.M. SYSTEM. SIMILARLY ANGULAR DISTRIBUTIONS IN MF=4 MAY BE GIVEN IN EITHER THE LAB OR C.M. SYSTEM. IN CONTRAST ENERGY SPECTRA IN MF=5 CAN ONLY BE GIVEN IN THE LABORATORY SYSTEM.

THE ANGULAR DISTRIBUTIONS OUTPUT BY THIS CODE IN MF=4 ARE IN THE SAME SYSTEM IN WHICH THEY ARE GIVEN IN MF=6 - EITHER LAB OR CENTER-OF-MASS SYSTEM.

THE ENERGY SPECTRA OUTPUT BY THIS CODE IN MF=5 MUST BE IN THE LAB SYSTEM - THIS IS THE ONLY ALLOWED FORM FOR MF=5 DATA.

FOR MF=6 SPECTRA GIVEN IN THE LAB SYSTEM THIS MERELY REQUIRES COPYING THE GIVEN SPECTRA TO MF=5 OUTPUT.

FOR MF=6 SPECTRA GIVEN IN THE CENTER-OF-MASS SYSTEM ONLY FIRST ORDER CORRECTIONS IN THE SPECTRA AND USED AND THEY ARE THEN OUTPUT IN MF=5 AS IN THE LAB SYSTEM - THE FIRST ORDER CORRECTIONS ARE DESCRIBED BELOW.

DEFINING,
 MM = CENTER OF MASS MOTION
 CM = OUTGOING (EMITTED) PARTICLE IN CENTER OF MASS
 LAB = OUTGOING (EMITTED) PARTICLE IN LAB
 THETA = CM SCATTERING ANGLE RELATIVE TO INCIDENT DIRECTION
 COS(CM) = COSINE OF THE CM SCATTERING ANGLE

FOR NEUTRONS INCIDENT WITH AN ENERGY, E, AND THEREFORE A SPEED,

$$VN(E) = 2 * \sqrt{E} / \text{MASS(IN)}$$

THE CENTER-OF-MASS SPEED IS GIVEN BY,

$$V(MM) = VN(E) / (1 + A)$$

$$\begin{aligned} E(\text{MM}) &= 1/2 * \text{MASS}(\text{IN}) * V(\text{MM})^2 \\ &= 1/2 * \text{MASS}(\text{IN}) * V_N(E)^2 / (1 + A)^2 \\ &= E / (1 + A)^2 \end{aligned}$$
$$\begin{aligned} V(\text{LAB}) * \cos(\text{LAB}) &= V(\text{MM}) + V(\text{CM}) * \cos(\text{CM}) \\ V(\text{LAB}) * \sin(\text{LAB}) &= V(\text{CM}) * \sin(\text{CM}) \end{aligned}$$

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EP(LAB) = 0.5*MASS(OUT)*V(LAB)**2

= E(MM) + EP(CM) + 2*COS(CM)*SORT(E(MM)*EP(CM))
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$$\begin{aligned} V(CM) * \cos(CM) &= V(LAB) * \cos(LAB) - V(MM) \\ V(CM) * \sin(CM) &= V(LAB) * \sin(LAB) \end{aligned}$$

$$EP(CM) = 0.5 \cdot MASS(OUT) \cdot V(CM)^2$$

$$= E(MM) + EP(LAB) - 2 \cdot \cos(LAB) \cdot \sqrt{E(MM) \cdot EP(LAB)}$$

$$V(\underline{\text{LAB}}) * \text{COS}(\text{LAB}) = V(\underline{\text{MM}}) + V(\underline{\text{CM}}) * \text{COS}(\text{CM})$$

OR $\cos(CM)$ FROM THE RELATIONSHIP,

$$\text{COS (CM)} = [\text{V (LAB)} * \text{COS (LAB)} - \text{V (MM)}] / \text{V (CM)}$$

THE JACOBIAN CAN BE DEFINED FROM,

$$J = D[\text{COS}(\text{CM})]/D[\text{COS}(\text{LAB})] = V(\text{LAB})/V(\text{CM}) \\ = \text{SORT}[\text{EP}(\text{LAB})/\text{EP}(\text{CM})]$$

$$F(E, EP(LAB), \cos(LAB)) * D(\cos(LAB)) = F(E, EP(CM), \cos(CM)) * D(\cos(CM))$$

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