					Sixp
PROGRAM					Sixp
======					Sixp
VERSION		-			Sixp
VERSION	92-2	(FEBRU	ARY 1992)	*INCREASED CORE ALLOCATION TO	Sixp
VERSION :	02-3	/ADDTT	10021	ACCOMMODATE JEF AND EFF EVALUATIONS. *ADDED ADDITIONAL DATA TESTS.	Sixp
VERSION :		-		*CORRECTED KALBACH-MANN CALCULATIONS.	Sixp
VERSION .	<i>J</i>	(SEFI.	1992)	*FOR PHOTON PRODUCTION OUTPUT MF=12	Sixp
				(MULTIPLICITY), MF=14 (ISOTROPIC	Sixp
				ANGULAR DISTRIBUTIONS) AND MF=15	Sixp
				(SPECTRA) - PREVIOUSLY ONLY MF=15.	Sixp
				*FIRST ORDER CORRECTIONS TRANSFORMING	Sixp
				CENTER-OF-MASS SPECTRA TO LAB SYSTEM	Sixp
				FOR OUTPUT IN MF=5	Sixp
				*CORRECTED ISOTROPIC ANGULAR	Sixp
				DISTRIBUTION FLAG (LI)	Sixp
VERSION	94-1	(JANUAI	RY 1994)	*VARIABLE ENDF/B INPUT DATA FILENAME	Sixp
				TO ALLOW ACCESS TO FILE STRUCTURES	Sixp
				(WARNING - INPUT PARAMETER FORMAT HAS BEEN CHANGED)	Sixp
				*CLOSE ALL FILES BEFORE TERMINATING	Sixp
				(SEE, SUBROUTINE ENDIT)	Sixp
				*INCREASED MAXIMUM TABLE SIZE FROM	Sixp
				2000 TO 6000.	Sixp
VERSION :	96-1	(JANUAF	RY 1996)	*COMPLETE RE-WRITE	Sixp
			•	*IMPROVED COMPUTER INDEPENDENCE	Sixp
				*ALL DOUBLE PRECISION	Sixp
				*ON SCREEN OUTPUT	Sixp
				*UNIFORM TREATMENT OF ENDF/B I/O	Sixp
	oo -	/a.a	1000:	*IMPROVED OUTPUT PRECISION	Sixp
VERSION	99-1	(MARCH	1999)	*CORRECTED CHARACTER TO FLOATING	Sixp
				POINT READ FOR MORE DIGITS	Sixp
				*UPDATED TEST FOR ENDF/B FORMAT VERSION BASED ON RECENT FORMAT CHANGE	Sixp
				*GENERAL IMPROVEMENTS BASED ON	Sixp
				"GENERAL IMPROVEMENTS BASED ON USER FEEDBACK	Sixp
VERSION :	99-2	(JUNE 1	L999)	*ASSUME ENDF/B-VI, NOT V, IF MISSING	Sixp
		-	•	MF=1, MT-451.	Sixp
VERS. 20	00-1	(FEBRUA	ARY 2000)	*GENERAL IMPROVEMENTS BASED ON	Sixp
				USER FEEDBACK	Sixp
VERS. 20	02-1	(JANUAF	RY 2002)	*CORRECTED ANGULAR DISTRIBUTION (MF=4)	_
				OUTPUT TO INSURE USED FIELDS ARE 0	Sixp
		(MAY 20	-	*OPTIONAL INPUT PARAMETERS	Sixp
		(NOV. 2	2002)	*EXTENDED TO ALLOW CHARGED PARTICLE	Sixp
				ANGULAR DISTRIBUTION IN MF=4 -	Sixp
				WARNING - STRICTLY SPEAKING THIS IS NOT LEGAL, SINCE MF=4 IS SUPPOSED TO	Sixp
				BE USED ONLY FOR NEUTRON ANGULAR	Sixp
				DISTRIBUTIONS - BUT WHERE MT MAKES	Sixp
				IT OBVIOUS THAT THE OUTGOING PARTICLE	
				IS NOT A NEUTRON HOPEFULLY IT WILL	Sixp
				NOT CAUSE A PROBLEM IF MF=4 IS USED	Sixp
				FOR CHARGED PARTICLES.	Sixp
VERS. 20	04-1	(MARCH	2004)	*ADDED INCLUDE FOR COMMON	Sixp
				*INCREASED MAXIMUM TABLE SIZE FROM	Sixp
				6,000 TO 12,000.	Sixp
				*ADDED DUMMY A FOR ELEMENTS	Sixp
				*CORRECTED OUTPUT INTERPOLATON LAWS	Sixp
VERS. 20	07-1	(JAN.	2007)	*CHECKED AGAINST ALL ENDF/B-VII.	Sixp
				*INCREASED MAXIMUM TABLE SIZE FROM	Sixp
TEDC 20	07-2	(DEC	20071	12,000 TO 120,000.	Sixp
VERS. 20		-	2007)	*72 CHARACTER FILE NAMES.	Sixp
VERS. 20: VERS. 20:		_	2010) 2011)	*General update based on user feedback *Added MF/MT=9/5 yield output starting	_
VERU. 20.	- T - T	/Hay	~011)	from MF/MT=6/5 distributions.	Sixp
				*Increased maximum Legendre order from	-
					_
				30 to 1,000 - WARNING - using more	Sixn
				30 to 1,000 - WARNING - using more than 30 results in NONSENSE = NOISE!!	Sixp

	*Added CODENAME	Sixpak
	*32 and 64 bit Compatible	Sixpak
	*Added ERROR stop	Sixpak
	*For photons, combine discrete and	Sixpak
	<pre>continuum into tabulated increasing energy order.</pre>	Sixpak Sixpak
	*Check energy output order increasing.	_
	Print WARNING if not increasing - do	Sixpak
	not STOP- stopping would prevent ALL	Sixpak
	output - the user may not be at all interested in the BAD data, but may	Sixpak Sixpak
	be interested in other output data	Sixpak
	that is o.k.	Sixpak
VERS. 2015-1 (Jan. 2015)	*Extended OUT9.	Sixpak
	*Replaced ALL 3 way IF Statements.	Sixpak
VERS. 2017-1 (May 2017)	*Deleted unused coding. *Increased max. point to 600,000	Sixpak Sixpak
·=···· = (,	*Updated based on user feedback	Sixpak
VERS. 2017-2 (Oct. 2017)	*Updated for new P(nu) formats =	Sixpak
	Recognized and ignored = no MF=5	Sixpak
VERS. 2018-1 (Jan. 2018)	<pre>equivalent. *Updated to skip Nu-Bar Data = there</pre>	Sixpak Sixpak
12101 2020 2 (00111 2020)	is no double-differential data to	Sixpak
	process.	Sixpak
	*On-linr report for ALL ENDERROR	Sixpak
VERS. 2019-1 (June 2019)	*Additional Interpolation Law Tests *Checked Maximum Tabulated Energy to	Sixpak Sixpak
	insure it is the same for all MTs -	Sixpak
	if not, print WARNING messages.	Sixpak
	*WARNING MT=5 - not allowed in MF=4/5	Sixpak
	see ENDF102 - but will translate here to allow diagnostic use ONLY.	Sixpak Sixpak
	*Corrected END Histogram - guarantee	Sixpak
	it ends with zero cross section, e.g.,	_
	(E,Y) only defines upper energy of	Sixpak
	the last group - Y has no meaning,	Sixpak
	by ENDF convention it should be $Y = 0$	Sixpak
OWNED, MAINTAINED AND DISTR	IBUTED BY	Sixpak
		Sixpak
THE NUCLEAR DATA SECTION	ACENOV	Sixpak
INTERNATIONAL ATOMIC ENERGY P.O. BOX 100	AGENCI	Sixpak Sixpak
A-1400, VIENNA, AUSTRIA		Sixpak
EUROPE		Sixpak
ORIGINALLY WRITTEN BY		Sixpak
ORIGINALLI WRITTEN BI		Sixpak Sixpak
Dermott E. Cullen		Sixpak
		Sixpak
PRESENT CONTACT INFORMATION		Sixpak
Dermott E. Cullen		Sixpak Sixpak
1466 Hudson Way		Sixpak
Livermore, CA 94550		Sixpak
U.S.A.		Sixpak
Telephone 925-443-1911 E. Mail RedCullen1@Comca:		Sixpak
	st.net	Sixpak
Website RedCullen1.net/Ho		Sixpak Sixpak
Website RedCullen1.net/H		Sixpak Sixpak
Website RedCullen1.net/HGCOLLABORATION	OMEPAGE. NEW	Sixpak Sixpak Sixpak
Website RedCullen1.net/HG COLLABORATION	OMEPAGE.NEW	Sixpak Sixpak Sixpak Sixpak
Website RedCullen1.net/HGCOLLABORATION	OMEPAGE.NEW	Sixpak Sixpak Sixpak
Website RedCullen1.net/HC COLLABORATION ====================================	OMEPAGE.NEW	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
Website RedCullen1.net/HC COLLABORATION ====================================	OMEPAGE.NEW WITH, CENTER, BROOKHAVEN NATIONAL LAB	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
Website RedCullen1.net/HC COLLABORATION ====================================	OMEPAGE.NEW WITH, CENTER, BROOKHAVEN NATIONAL LAB	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
Website RedCullen1.net/HG COLLABORATION DEVELOPED IN COLLABORATION WATHE NATIONAL NUCLEAR DATA GATHE NUCLEAR DATA SECTION, SECTION	OMEPAGE.NEW WITH, CENTER, BROOKHAVEN NATIONAL LAB	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
Website RedCullen1.net/HG COLLABORATION DEVELOPED IN COLLABORATION WITHE NATIONAL NUCLEAR DATA GOVERNMENT OF THE NUCLEAR DATA SECTION, SECT	OMEPAGE.NEW WITH, CENTER, BROOKHAVEN NATIONAL LAB LAEA, VIENNA, AUSTRIA	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak

NUCLEAR DATA	Sixpak
ACKNOWLEDGEMENT (VERSION 92-1)	Sixpak Sixpak
THE AUTHOR THANKS SOL PEARLSTEIN (BROOKHAVEN NATIONAL LAB) FOR SIGNIFICANTLY CONTRIBUTING TOWARD IMPROVING THE ACCURACY AND COMPUTER INDEPENDENCE OF THIS CODE - THANKS, SOL	Sixpak Sixpak Sixpak Sixpak
ACKNOWLEDGEMENT (VERSION 92-4)	Sixpak Sixpak
THE AUTHOR THANKS BOB MACFARLANE (LOS ALAMOS) FOR SUGGESTING HOW TO PROPERLY OUTPUT THE PHOTON PRODUCTION DATA TO PUT IT INTO EXACTLY THE FORM NEEDED FOR USE IN PROCESSING CODES.	Sixpak Sixpak Sixpak Sixpak Sixpak
THE AUTHOR THANKS CHRIS DEAN (WINFRITH) FOR POINTING OUT ERRORS IN THE EARLIER TREATMENT OF THE KALBACH-MANN FORMALISM AND IN THE DEFINITION OF THE ISOTROPIC ANGULAR DISTRIBUTION FLAG (LI).	Sixpak Sixpak Sixpak Sixpak
AUTHORS MESSAGE	Sixpak
THE COMMENTS BELOW SHOULD BE CONSIDERED THE LATEST DOCUMENTATION INCLUDING ALL RECENT IMPROVEMENTS. PLEASE READ ALL OF THESE COMMENTS BEFORE IMPLEMENTING AND USING THESE CODES.	Sixpak Sixpak Sixpak Sixpak Sixpak
AT THE PRESENT TIME WE ARE ATTEMPTING TO DEVELOP A SET OF COMPUTER INDEPENDENT PROGRAMS THAT CAN EASILY BE IMPLEMENTED ON ANY ONE OF A WIDE VARIETY OF COMPUTERS. IN ORDER TO ASSIST IN THIS PROJECT IT WOULD BE APPECIATED IF YOU WOULD NOTIFY THE AUTHOR OF ANY COMPILER DIAGNOSTICS, OPERATING PROBLEMS OR SUGGESTIONS ON HOW TO IMPROVE THIS PROGRAM. HOPEFULLY, IN THIS WAY FUTURE VERSIONS OF THIS PROGRAM WILL BE COMPLETELY COMPATIBLE FOR USE ON YOUR COMPUTER.	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
PURPOSE	Sixpak
1) CHECK ALL DOUBLE-DIFFERENTIAL DATA (MF=6)	Sixpak Sixpak
2) OUTPUT EQUIVALENT MF = 4, 5, 12, 14 AND 15 DATA.	Sixpak Sixpak
DATA CHECKING	Sixpak Sixpak
ALL OF THE ENDF/B-VI MF=6 DATA IS CHECKED - FOR DETAILS SEE BELOW.	Sixpak Sixpak Sixpak
THE MF=6 DATA IS NOT CORRECTED AND OUTPUT IN THE ENDF/B FORMAT. IT IS MERELY CHECKED. IF ERRORS ARE FOUND IT IS UP TO THE USER TO TAKE CORRECTIVE ACTION ON THE MF=6 DATA.	Sixpak Sixpak Sixpak
IN CONTRAST WHEN PROBLEMS ARE FOUND IN DATA WHICH WILL BE OUTPUT IN THE ENDF/B FORMAT (MF=4, 5, 12, 14 AND 15), WHENEVER POSSIBLE CORRECTIVE ACTION WILL BE TAKEN.	Sixpak Sixpak Sixpak Sixpak
FURTHER CHECKS AND CORRECTIONS	Sixpak Sixpak Sixpak
ONCE THE DATA HAS BEEN OUTPUT IN MF = 4, 5, 12, 14 AND 15 FORMATS FURTHER CORRECTIVE ACTION CAN BE TAKEN AS FOLLOWS,	Sixpak Sixpak Sixpak
PROGRAM LEGEND	Sixpak Sixpak
CAN BE USED TO CORRECT ANGULAR DISTRIBUTIONS WHICH ARE NEGATIVE, TO CONVERT FROM LEGENDRE COEFFICIENTS TO TABULATED ANGULAR DISTRIBUTIONS AND GENERALLY PERFORM MORE EXTENSIVE TESTS OF ALL MF=4 DATA.	Sixpak Sixpak Sixpak Sixpak Sixpak
PROGRAM EVALPLOT	Sixpak Sixpak
VERSION 92-1 AND LATER VERSIONS CAN PLOT ALL OF THE MF=4, 5 AND 15 DATA OUTPUT BY THIS CODE. EARLIER VERSIONS CAN PLOT MF=4 AND 5. GRAPHICS IS AN EXCELLENT WAY TO CHECK THIS DATA.	Sixpak Sixpak
PROGRAM PLOTTAB	Sixpak Sixpak Sixpak

THIS IS A GENERAL PLOTTING PROGRAM AND THERE IS AN INTERFACE IN THIS CODE TO PRODUCE OUTPUT FOR ANY MF=6 DATA IN THE PLOTTAB INPUT FORMAT. THIS PROGRAM CAN BE USED TO CHECK ALL OF THE MF=6 DATA AS WELL AS THE EQUIVALENT MF=4, 5, 12, 14 AND 15 DATA - AS WELL AS COMPARING THE ORIGINAL MF=6 AND EQUIVALENT DATA.

DATA OUTPUT

THE ENDF/B MF=4, 5, 12, 14 AND 15 FORMATS ONLY ALLOW FOR NEUTRONS INCIDENTS

THE ENDF/B MF=4 AND 5 FORMATS ONLY ALLOW FOR NEUTRONS OUTGOING.

THE ENDF/B MF=12, 14 AND 15 ONLY ALLOWS FOR PHOTONS OUTGOING.

THESE ARE THE ONLY COMBINATIONS OF DATA OUTPUT BY THIS CODE.

ALL OTHER COMBINATIONS OF INCIDENT AND OUTGOING PARTICLES ARE CHECKED, BUT THE RESULTS CANNOT BE OUTPUT IN THE ENDF/B FORMAT. HOWEVER, USING THE PLOTTAB INTERFACE BUILT INTO THIS CODE THIS DATA CAN, AND HAS BEEN, OUTPUT AND CHECKED.

THE NEUTRON DATA IN MF=4 CAN BE IN THE FORM OF EITHER TABULATED ANGULAR DISTRIBUTIONS OR LEGENDRE COEFFICIENTS.

THE NEUTRON (MF=5) OR PHOTON (MF=15) SPECTRA ARE BOTH IN EXACTLY THE SAME FORMAT = ARBITRARY TABULATED FUNCTIONS - ENDF/B OPTION LF=1.

ENDF/B DATA OUTPUT ORDER

ENDF/B DATA IS OUTPUT IN ASCENDING MAT, MF, MT ORDER. IN ORDER TO ALLOW THIS PROGRAM TO PRODUCE ALL OUTPUT IN A SINGLE PASS THROUGH THE MF=6 DATA, OUTPUT FOR EACH (MAT, MT) IS OUTPUT TO SEPERATE FILES FOR MF=4, 5, 12, 14 AND 15.

FOR SUBSEQUENT USE THE ENDF/B FORMATTED DATA OUTPUT BY THIS CODE CAN BE MERGED TOGETHER USING PROGRAM MERGER (CONTAIN THE AUTHOR OF THIS CODE FOR A COPY OF MERGER), E.G., MERGE MF=12, 14 AND 15 DATA IN ORDER TO THEN CALCULATE PHOTON PRODUCTION DATA OR MF=4 AND 5 CAN BE MERGED TOGETHER TO CALCULATE NEUTRON TRANSFER - OR ALL OF THEM CAN BE MERGED TOGETHER TO PERFORM NEUTRON AND PHOTON CALCULATIONS.

CORRELATED (MF=6) VS. UNCORRELATED (MF=4 AND 5) DATA

THE ENDF/B DOUBLE DIFFERENTAL = CORRELATED - DATA IN MF=6 REPRESENTS DATA IN THE FORM,

F(E,EP,COS) = SIG(E)*Y(E)*GO(E,EP)*F(E,EP,COS)

SIG(E) = MF=3 CROSS SECTIONS Y(E) = YIELD (MULTIPLICITY) GO(E,EP) = ENERGY SPECTRUM F(E,EP,COS) = ANGULAR DISTRIBUTION

IN A SITUATION WHERE YOU HAVE MONOENERGETIC AND MONODIRECTIONAL NEUTRONS INCIDENT YOU WILL BE ABLE TO OBSERVE CORRELATION EFFECTS IN THE NEUTRON SPECTRUM AND ANGULAR DISTRIBUTION.

EVEN IN SITUATIONS WHERE YOU HAVE A NARROW SPECTRUM OF NEUTRONS THAT ARE HIGHLY DIRECTIONALLY ORIENTED YOU MAY BE ABLE TO OBSERVE THESE CORRELATION EFFECTS, E.G., A NARROW 14 MEV FUSION SOURCE INCIDENT ON THE FIRST WALL OF A CTR DEVICE.

FOR SUCH SITUATIONS USE OF THE CORRELATED (MF=6) DATA IS REQUIRED IN CALCULATIONS.

HOWEVER, IN MANY APPLICATIONS WHERE THERE IS A BROAD SPECTRUM OF NEUTRONS AND THE NEUTRON FLUX IS NOT HIGHLY DIRECTIONALLY ORIENTED, THE NEUTRON MULTIPLICATION, SPECTRUM AND ORIENTATION

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CAN BE FAIRLY ACCURATELY CALCULATED WITHOUT CONSIDERING CORRELATION EFFECTS.	Sixpak Sixpak
THE UNCORRELATED DATA PRODUCED BY THIS CODE REPLACES THE CORRELATED DATA,	Sixpak Sixpak Sixpak
F(E,EP,COS) = SIG(E)*Y(E)*GO(E,EP)*F(E,EP,COS)	Sixpak Sixpak
BY THE UNCORRELATED DATA,	Sixpak Sixpak
F(E,EP,COS) = SIG(E)*Y(E)*G0(E,EP)*F0(E,COS)	Sixpak Sixpak
BY INTEGRATING G0 (E,EP)*F(E,EP,COS) OVER SECONDARY ENERGY (EP) TO DEFINE AN AVERAGE ANGULAR DISTRIBUTION, F0 (E,COS).	Sixpak Sixpak Sixpak
WHAT IS LOST IN THIS PROCESS IS THE CORRELATION BETWEEN EP AND COS SO THAT IN A TRANSPORT CALCULATION ALL MOMENTS OF THE FLUX WILL HAVE THE SAME SPECTRUM, GO(E,EP) AND EACH WILL BE EFFECTED BY THE AVERAGE ANGULAR DISTRIBUTION.	Sixpak Sixpak Sixpak
FOR APPLICATIONS TO HIGH ENERGY FUSION APPLICATIONS CORRELATED DATA SHOULD BE USED. HOWEVER, FOR LOWER ENERGY APPLICATIONS, SUCH AS FISSION REACTORS, IT SHOULD BE ADEQUATE TO USE THE UNCORRELATED DATA - IN THIS CASE THE MOST IMPORTANT EFFECT WILL BE THE OVERALL NEUTRON MULTIPLICATION AND SPECTRUM.	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
AN IMPORTANT CONSIDERATION IN DESIGNING THIS PROGRAM IS THAT MANY COMPUTER CODES - DATA PROCESSING AND TRANSPORT CODES - CANNOT USE THE CORRELATED (MF=6) DATA - NOR ARE THEY INTENDED FOR HIGH ENERGY USE. FOR THESE CODES THE UNCORRELATED DATA PRODUCED BY THIS CODE SHOULD BE ADEQUATE TO MEET THEIR NEEDS.	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
WARNING - IT CANNOT BE STRESSED ENOUGH THAT THE OUTPUT OF THIS CODE SHOULD ONLY BE USED FOR LOW ENERGY APPLICATIONS - FAILURE TO HEED THIS WARNING CAN LEAD TO COMPLETELY UNRELIABLE RESULTS.	Sixpak Sixpak Sixpak
ENDF/B FORMAT	Sixpak Sixpak
ENDF/B FORMAT THIS PROGRAM ONLY USES THE ENDF/B BCD OR CARD IMAGE FORMAT (AS OPPOSED TO THE BINARY FORMAT) AND CAN HANDLE DATA IN ANY VERSION OF THE ENDF/B FORMAT (I.E., ENDF/B-I, II,III, IV, V OR VI FORMAT).	Sixpak Sixpak Sixpak Sixpak Sixpak
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THIS PROGRAM ONLY USES THE ENDF/B BCD OR CARD IMAGE FORMAT (AS OPPOSED TO THE BINARY FORMAT) AND CAN HANDLE DATA IN ANY VERSION OF THE ENDF/B FORMAT (I.E., ENDF/B-I, II,III, IV, V OR VI FORMAT). IT IS ASSUMED THAT THE DATA IS CORRECTLY CODED IN THE ENDF/B FORMAT AND NO ERROR CHECKING IS PERFORMED. IN PARTICULAR IT IS ASSUMED THAT THE MAT, MF AND MT ON EACH LINE IS CORRECT. SEQUENCE NUMBERS (COLUMNS 76-80) ARE IGNORED ON INPUT, BUT WILL BE CORRECTLY OUTPUT ON ALL LINES. THE FORMAT OF SECTION MF=1, MT=451 AND ALL SECTIONS OF MF=6 MUST BE CORRECT. THE PROGRAM SKIPS ALL OTHER SECTIONS OF DATA AND AS SUCH IS INSENSITIVE TO THE FORMAT OF ALL OTHER SECTIONS. CONTENTS OF OUTPUT 5 ENDF/B FORMATTED OUTPUT FILES ARE PRODUCED FOR NEUTRON INCIDENT DATA, 1) ENDFB.MF4 - ANGULAR DISTRIBUTIONS AND LEGENDRE COEFFICIENTS FOR NEUTRONS 2) ENDFB.MF5 - TABULATED NEUTRON ENERGY SPECTRA 3) ENDFB.M12 - PHOTON EMISSION MULTIPLICITY 4) ENDFB.M14 - PHOTON EMISSION ANGULAR DISTRIBUTIONS (ALWAYS ISOTROPIC) 5) ENDFB.M15 - TABULATED PHOTON EMISSION SPECTRA	Sixpak
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THIS PROGRAM ONLY USES THE ENDF/B BCD OR CARD IMAGE FORMAT (AS OPPOSED TO THE BINARY FORMAT) AND CAN HANDLE DATA IN ANY VERSION OF THE ENDF/B FORMAT (I.E., ENDF/B-I, II,III, IV, V OR VI FORMAT). IT IS ASSUMED THAT THE DATA IS CORRECTLY CODED IN THE ENDF/B FORMAT AND NO ERROR CHECKING IS PERFORMED. IN PARTICULAR IT IS ASSUMED THAT THE MAT, MF AND MT ON EACH LINE IS CORRECT. SEQUENCE NUMBERS (COLUMNS 76-80) ARE IGNORED ON INPUT, BUT WILL BE CORRECTLY OUTPUT ON ALL LINES. THE FORMAT OF SECTION MF=1, MT=451 AND ALL SECTIONS OF MF=6 MUST BE CORRECT. THE PROGRAM SKIPS ALL OTHER SECTIONS OF DATA AND AS SUCH IS INSENSITIVE TO THE FORMAT OF ALL OTHER SECTIONS. CONTENTS OF OUTPUT 5 ENDF/B FORMATTED OUTPUT FILES ARE PRODUCED FOR NEUTRON INCIDENT DATA, 1) ENDFB.MF4 - ANGULAR DISTRIBUTIONS AND LEGENDRE COEFFICIENTS FOR NEUTRONS 2) ENDFB.MF5 - TABULATED NEUTRON ENERGY SPECTRA 3) ENDFB.M12 - PHOTON EMISSION MULTIPLICITY 4) ENDFB.M14 - PHOTON EMISSION ANGULAR DISTRIBUTIONS (ALWAYS ISOTROPIC) 5) ENDFB.M15 - TABULATED PHOTON EMISSION SPECTRA	Sixpak

MT NUMBER, E.G., MT=16 = (N,2N) = 2. Sixpak Sixpak THE ONLY EXCEPT IN ENDF/B-VI IS MT=201 = TOTAL NEUTRON PRODUCTION Sixpak WHERE AN ACTUAL ENERGY DEPENDENT YIELD IS INCLUDED IN MF=6. Sixpak HOWEVER, IN THIS CASE THE MF=3 CROSS SECTION INCLUDES THE Sixpak MULTIPLICITY (S. PEARLSTEIN, PRIVATE COMMUNICATION, JAN. 1992), Sixpak SIG(MT=201) = 2*SIG(N,2N)+3*SIG(N,3N)....ETC.Sixpak Sixpak SO THAT FOR ALL ENDF/B-VI DATA AS OF JANUARY 1992 THE MF=4 AND 5 Sixpak DATA OUTPUT BY THIS CODE CAN BE USED IN CONJUNCTION WITH THE MF=3 Sixpak CROSS SECTIONS - WITHOUT ANY REFERENCE TO THE MF=6 YIELD. Sixpak Sixpak PHOTONS Sixpak Sixpak UNLIKE THE NEUTRONS WHERE WITH ONLY ONE EXCEPTION (MT=201) THE Sixpak MF=6 YIELD IS ENERGY INDEPENDENT, IN THE CASE OF PHOTON EMISSION Sixpak ALMOST ALL OF THE PHOTONS HAVE AN ENERGY DEPENDENT YIELD. Sixpak Sixpak THIS PROGRAM WILL OUTPUT THE PHOTON MULTIPLICITY IN MF=12 AND Sixpak INDICATE THAT THERE IS A NORMALIZED DISTRIBUTION IN MF=15 Sixpak (LF=1 IN MF=12). Sixpak Sixpak THIS PROGRAM WILL OUTPUT THE NORMALIZED PHOTON SPECTRA IN MF=15. Sixpak CONTINUOUS ENERGY SPECTRA AND DISCRETE PHOTONS WILL ALL BE OUTPUT Sixpak AS NORMALIZED SPECTRA. Sixpak Sixpak THIS PROGRAM WILL ALSO OUTPUT MF=14 PHOTON ANGULAR DISTRIBUTION Sixpak DATA, ALWAYS USING THE ISOTROPIC FLAG TO MINIMIZE OUTPUT. Sixpak Sixpak WARNING OF ENERGY DEPENDENT YIELD Sixpak Sixpak THIS PROGRAM WILL PRINT A WARNING MESSAGE IF A SECTION OF DATA Sixpak BEING OUTPUT IN THE ENDF/B FORMAT HAS AN ENERGY DEPENDENT MF=6 Sixpak YIELD AND THE EMITTED PARTICLE IS A NEUTRON - SINCE THE ENDF/B Sixpak CONVENTION IS THAT FOR EACH MT NUMBER THE MULTIPLICITY IS IMPLIED Sixpak WE DO NOT EXPECT AN ENERGY DEPENDENT MULTIPLICITY FOR NEUTRON Sixpak EMISSION. Sixpak Sixpak USING THE OUTPUT Sixpak Sixpak NOTE. THAT IN USING THIS DATA. STARTING FROM THE RELATIONSHIP. Sixpak Sixpak F(E,EP,COS) = SIG(E)*Y(E)*GO(E,EP)*FO(E,COS)Sixpak Sixpak USING THE ENDF/B CONVENTION THAT THE MULTIPLICITY IS EITHER Sixpak IMPLIED BY THE MT NUMBER (E.G., MT=16 = N,2N - MULTIPLICITY = 2) Sixpak OR INCLUDED IN THE CROSS SECTION (E.G., MT=201 = TOTAL NEUTRON Sixpak PRODUCTION) ALL THE INFORMATION REQUIRED FOR A CALCULATION IS Sixpak AVAILABLE IN. Sixpak Sixpak MF=3- SIG(E) Sixpak - F0(E,COS) - FOR OUTGOING NEUTRONS MF=4Sixpak - G0(E,EP) - FOR OUTGOING NEUTRONS MF=5 Sixpak - FOR OUTGOING PHOTONS MF=12 - Y(E) Sixpak MF=14 - F0(E,COS) - FOR OUTGOING PHOTONS (ALWAYS ISOTROPIC) Sixpak - G0(E,EP) - FOR OUTGOING PHOTONS MF=15 Sixpak Sixpak DOCUMENTATION Sixpak Sixpak ONLY SECTIONS OF MF=4, 5, 12, 14, 15 ARE OUTPUT ON A ENDF/B FILE. Sixpak THE ONLY DOCUMENTATION IS THE ENDF/B TAPE LABEL (FIRST RECORD OF EACH FILE) WHICH IDENTIFIES THE DATA AS SIXPAK OUTPUT. Sixpak Sixpak REACTION INDEX Sixpak == Sixpak THIS PROGRAM DOES NOT USE THE REACTION INDEX WHICH IS GIVEN IN Sixpak SECTION MF=1, MT=451 OF EACH EVALUATION. Sixpak Sixpak SECTION SIZE Sixpak

ALL OF THE DATA IN ENDF/B-VI, MF=6 ARE QUITE SMALL TABLES. AS SUCH Sixpak

THE PROGRAM SELECTS DATA TO BE PROCESSED BASED ON MAT/MT RANGES (MF=6 ASSUMED). THIS PROGRAM ALLOWS UP TO 100 MAT/MT RANGES TO BE SPECIFIED BY INPUT PARAMETERS. THE PROGRAM WILL ASSUME THAT THE ENDF/B TAPE IS IN MAT ORDER. THE PROGRAM WILL TERMINATE EXECUTION WHEN A MAT IS FOUND THAT IS ABOVE ALL REQUESTED MAT RANGES. PROGRAM OPERATION	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
EACH SECTION (MT) OF MF=6 DATA IS SUBDIVIDED INTO SUBSECTIONS - ONE SUBSECTION FOR EACH EMITTED PARTICLE.	Sixpak Sixpak Sixpak Sixpak
EACH SUBSECTION OF DATA IS CONSIDERED SEPARATELY. EACH SUBSECTION OF ENDF/B MF=6 DATA TO PROCESS IS IN THE FORM,	Sixpak Sixpak Sixpak
F(E,EP,COS) = SIG(E)*Y(E)*GO(E,EP)*F(E,EP,COS)	Sixpak Sixpak
SIG(E) = MF=3 CROSS SECTIONS Y(E) = YIELD (MULTIPLICITY)	Sixpak Sixpak
GO(E,EP) = ENERGY SPECTRUM	Sixpak
F(E, EP, COS) = ANGULAR DISTRIBUTION	Sixpak
GO(E,EP) = 1 WHEN INTEGRATED OVER EP (SECONDARY ENERGY)	Sixpak Sixpak
GO(E,EP) * F(E,EP,COS) = 1 WHEN INTEGRATED OVER EP AND COS	Sixpak
	Sixpak
THIS PROGRAM WILL DEFINE THE ZEROTH ORDER MOMENTS OF THE	Sixpak
ENERGY AND ANGULAR DISTRIBUTIONS,	Sixpak
G0(E,EP) = G0(E,EP)*F(E,EP,COS) INTEGRATED OVER COS	Sixpak Sixpak
FO(E,COS) = GO(E,EP)*F(E,EP,COS) INTEGRATED OVER EP	Sixpak
	Sixpak
FOR NEUTRON INDUCED REACTIONS THE ENDF/B FORMATTED OUTPUT WILL BE	Sixpak Sixpak
F0(E,COS) - IN ENDFB.MF4 FOR NEUTRONS OUT OF A REACTION	Sixpak
GO (E,EP) - IN ENDFB.MF5 FOR NEUTRONS OUT OF A REACTION	Sixpak
- IN ENDFB.M15 FOR PHOTONS OUT OF A REACTION	Sixpak Sixpak
FOR NEUTRONS INCIDENT AND NEUTRONS EMITTED THIS DATA WILL BE	Sixpak
OUTPUT IN MF=4 AND 5 FORMATS.	Sixpak
	Sixpak
FOR NEUTRONS INCIDENT AND PHOTONS EMITTED THIS DATA WILL BE OUTPUT IN MF=15 FORMAT - THE SPECTRA ARE OUTPUT AND THE	Sixpak Sixpak
ANGULAR DISTRIBUTION IS IGNORED.	Sixpak
	Sixpak
ALL PHOTON EMISSION IN THE ENDF/B-VI LIBRARY AS OF JANUARY 1992	Sixpak
IS ISOTROPIC AND AS SUCH NO DISTRIBUTION OF PHOTON ANGULAR DISTRIBUTIONS NEED BE OUTPUT - IT IS ALWAYS ISOTROPIC.	Sixpak
DISTRIBUTIONS NEED BE COTFOT - IT IS ALWAYS ISOTROFIC.	Sixpak Sixpak
FOR ALL OTHER COMBINATIONS INCIDENT AND EMITTED PARTICLES	Sixpak
THERE WILL BE NO ENDF/B FORMATTED OUTPUT.	Sixpak
VADIANTONO EDOM ENDE/D MANUAT	Sixpak
VARIATIONS FROM ENDF/B MANUAL	Sixpak Sixpak
LAW=1, LANG=2 = KALBACH-MANN	Sixpak
	Sixpak
FOR THE DISTRIBUTIONS,	Sixpak Sixpak
F(MU,E,EP) = G0(E,EP)*A*(COSH(MU*A)+R(E,EP)*SINH(MU*A))	Sixpak
	Sixpak
GO(E,EP) = 1 - WHEN INTEGRATED OVER EP.	Sixpak
A*(COSH(MU*A)+R(E,EP)*SINH(MU*A)) = 2 - WHEN INTEGRATD OVER MU	Sixpak Sixpak
11 (SOUNDED MILE MILE INTEGRALD OVER MO	JIAPAK

THIS MEANS AS DEFINED IN THE ENDF/B MANUAL THE DISTRIBUTIONS ARE NORMALIZED TO 2, INSTEAD OF 1. IN ORDER TO OBTAIN CORRECTLY NORMALIZED DISTRIBUTIONS THE DISTRIBUTION SHOULD BE DEFINED TO INCLUDE A FACTOR OF 1/2 MULTIPLYING THE ANGULAR PART OF THE DISTRIBUTION.

F(MU, E, EP) = GO(E, EP) *0.5*A*(COSH(MU*A)+R(E, EP)*SINH(MU*A))

THIS IS THE FORM USED IN THIS CODE

LAW=1, ND NOT 0 = DISCRETE SECONDARY ENERGY DISTRIBUTION

THE ENDF/B MANUAL SAYS THESE ARE FLAGGED WITH NEGATIVE ENERGIES. IN ENDF/B-VI ALL OF THESE HAVE POSITIVE ENERGY. THIS CODE DOES NOT CONSIDER THE ENDF/B-VI DATA TO BE IN ERROR.

WITH THE CONVENTION ACTUALLY USED IN ENDF/B-VI ALL SECONDARY ENERGIES SHOULD BE NON-NEGATIVE AND IN ASCENDING ENERGY ORDER FOR EACH INCIDENT ENERGY.

FROM THE ENDF/B MANUAL IT IS NOT OBVIOUS WHAT GO(E,EP) SHOULD BE FOR DISCRETE PHOTONS - PHYSICALLY THIS IS A DELTA FUNCTION. IN ENDF/B-VI IT IS ENTERED AS 1.0 = INTERPRETING IT AS INTEGRATED OVER SECONDARY ENERGY - IN WHICH CASE THE DELTA FUNCTION = 1.0.

LIMITATIONS

CHECKING DATA

THIS PROGRAM CHECKS ALL ENDF/B-VI MF=6 DATA. THE FOLLOWING CHECKS

ARE PERFORMED.

PARAMETERS

CODE.

ALL PARAMETERS ARE CHECKED FOR CONSISTENCY. IF PARAMETERS ARE NOT CONSISTENT THE PROGRAM MAY NOT BE ABLE TO PERFORM THE FOLLOWING TESTS AND WILL MERELY SKIP A SECTION OF DATA.

INTERPOLATION LAWS

ALL INTEGRATIONS ARE PERFORMED USING THE INTERPOLATION LAW GIVEN FOR SECONDARY ENERGY AND/OR COSINE. INTEGRATIONS ARE NOT PERFORMED OVER INCIDENT - ONLY INTEGRATION OVER SECONDARY ENERGY AND/OR COSINE ARE PERFORMED AT EACH INCIDENT ENERGY. THEREFORE THE INTERPOLATION LAW FOR INCIDENT ENERGY IS NOT USED BY THIS

ALL INTERPOLATION LAWS ARE CHECKED. ALL DATA ASSOCIATED WITH INTERPOLATION LAWS ARE CHECKED, E.G., NO NON-NEGATIVE VALUES REQUIRING LOG INTERPOLATION. IN ORDER TO PERFORM REQUIRED INTEGRALS OVER COS AND EP IT IS IMPERATIVE THAT THE INTERPOLATION LAWS BE COMPATIBLE WITH THE DATA.

ENDF/B-VI ALLOWS NEW INTERPOLATION LAWS FOR CORRESPONDING POINT AND UNIT BASE TRANSFORMATION INTERPOLATION. NONE OF THESE NEW INTERPOLATION LAWS ARE USED IN THE ENDF/B-VI LIBRARY AS OF JANUARY 1992 TO INTERPOLATE IN SECONDARY ENERGY OR COSINE. THEREFORE THIS PROGRAM CAN PERFORM ALL OF THE REQUIRED INTEGRALS OVER SECONDARY ENERGY AND/OR COSINE USING ONLY THE OLDER INTERPOLATION CODES. THIS PROGRAM ONLY PERFORMS INTEGRALS FOR EACH INCIDENT ENERGY, SO THAT INTERPOLATION IN INCIDENT ENERGY IS NOT PERFORMED BY THIS PROGRAM.

NEW INTERPOLATION SCHEMES ARE USED FOR INCIDENT ENERGY - FOR EXAMPLE, CORRESPONDING POINT INTERPOLATION IS SPECIFIED TO ALLOW INTERPOLATION IN G0 (E,EP) TO SIMULATE CASES WHERE THE INPUT ENERGY LIMIT IS DEFINED BY E-EP = A DIAGONAL CURVE ACROSS (E,EP) SPACE. THIS INTERPOLATION CODE CANNOT BE SPECIFIED IN THE MF=5 OUTPUT OF THIS CODE - MF=5 ONLY ALLOWS THE OLDER INTERPOLATION LAWS INT=1 THROUGH 5. THEREFORE THIS PROGRAM WILL USE THE CLOSEST

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CORRESPONDING INTERPOLATION CODE FOR OUTPUT TO MF=5. FOR USE WHERE THE OUTPUT OF THIS CODE = LOW ENERGY APPLICATIONS - THIS SHOULD HAVE LITTLE EFFECT ON RESULTS.

FOR CONSISTENCY WITH EARLIER VERSIONS OF ENDF/B IN CREATING THE ENDF/B OUTPUT, IF ANY INPUT INTERPOLATION LAW IS NOT IN THE RANGE 1-5. IT WILL FIRST BE TESTED TO SEE IF MOD (10) IT IS IN THIS RANGE, FINALLY IF EVEN THIS DOESN'T WORK IT IS SET EQUAL TO 2 (LINEARLY INTERPOLATION). THIS METHOD WILL EFFECTIVELY REPLACE CORRESPONDING POINT AND UNIT BASE TRANSFORMATION BY THE CLOSEST RELATED INTERPOLATION LAW 1 THROUGH 5 - AGAIN NOTE, AS OF JANUARY 1992 NONE OF THESE NEW LAWS ARE USED IN ENDF/B-VI. IF THIS MUST BE DONE FOR INTERPOLATION IN SECONDARY ENERGY OR COSINE AN ERROR MESSAGE WILL BE PRINTED - SINCE THIS WOULD EFFECT THE ACCURACY OF THE INTEGRALS PERFORMED BY THIS PROGRAM. IF THIS MUST BE DONE FOR INCIDENT ENERGY NO MESSAGE IS PRINTED - SINCE THIS WILL NOT EFFECT THE ACCURACY OF THE INTEGRALS PERFORMED BY THIS PROGRAM.

SPECTRA AND ANGULAR DISTRIBUTIONS

ALL SPECTRA AND ANGULAR DISTRIBUTIONS ARE CHECKED TO INSURE THEY ARE NORMALIZED AND DO NOT INCLUDE ANY NEGATIVE VALUES.

LEGENDRE COEFFICIENTS

THE NORMALIZATION, FO, CANNOT BE NEGATIVE.

LEGENDRE COEFFICIENTS IN NORMAL FORM ARE CHECKED TO INSURE THEY ARE IN THE RANGE -1 TO +1 = THE LEGENDRE EXPANSION OF A DELTA FUNCTION AT COS=+1 OR -1 - COEFFICIENTS SHOULD NOT EXCEED WHAT YOU GET FROM A DELTA FUNCTION.

ANGULAR DISTRIBUTIONS ARE CHECKED AT COS = -1, 0 AND +1.

CREATING ENDF/B OUTPUT

------ Sixpak THIS PROGRAM CAN CREATE EQUIVALENT MF =4, 5, 12, 14, 15 DATA FOR ALL OF THE DATA INCLUDED IN ENDF/B-VI AS OF JANUARY 1992, EXCEPT FOR 1 SECTION OF LAW=6 DATA (SEE DETAILS BELOW).

THIS PROGRAM HAS NOT BEEN TESTED ON OTHER DATA LIBRARIES, E.G., JEF, JENDL, ETC.

THE PROGRAM HAS THE FOLLOWING LIMITATION AS FAR AS CREATING ENDF/B FORMATTED OUTPUT.

ISOTROPIC PHOTON EMISSION

FOR PHOTON EMISSION THE DISTRIBUTIONS ARE ASSUMED TO BE ISOTROPIC AND ONLY THE MULTIPLICITY IS OUTPUT IN MF=12, ISOTROPIC ANGULAR DISTRIBUTIONS IN MF=14 AND THE SPECTRA IN MF=15. ALL ENDF/B-VI MF=6 DATA AS OF JANUARY 1992 INCLUDE ONLY ISOTROPIC PHOTON EMISSION - SO THAT THIS IS NOT A LIMITATION ON TRANSLATING ENDF/B-VI DATA.

EITHER TABULATED OR LEGENDRE COEFFICIENTS

FOR LAW=2 THE REPRESENTATION, EITHER TABULATED OR LEGENDRE COEFFICIENTS, CAN BE SPECIFIED FOR EACH INCIDENT ENERGY.

IN ORDER TO OBTAIN CORRECT ENDF/B OUTPUT THE REPRESENTATION MUST BE THE SAME FOR ALL INCIDENT ENERGIES = MF=4 DATA CAN ONLY BE TABULATED OR LEGENDRE OVER THE ENTIRE ENERGY RANGE.

YIELD AND OUTPUT NORMALIZATION

THE YIELD INCLUDED WITH EACH SECTION OF DATA IS NOT USED FOR OUTPUT FOR NEUTRONS, BUT IS INCLUDED IN THE OUTPUT FOR PHOTONS. IN ALL CASES THE ANGULAR DISTRIBUTIONS AND SPECTRA OUTPUT ARE NORMALIZED TO UNITY.

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LAW=0 =====	Sixpak
NO OUTPUT - INCIDENT NEUTRON - EMITTED PHOTON OR NEUTRON	Sixpak Sixpak
REACTIONS ARE NOT EXPECTED.	Sixpak
	Sixpak
LAW=1	Sixpak
====	Sixpak
FOR EACH INCIDENT ENERGY DISCRETE AND CONTINUOUS EMISSION SPECTRA	Sixpak
CANNOT BE MIXED TOGETHER - THEY MUST BE ALL EITHER DISCRETE OR CONTINUOUS. IF DISCRETE EMISSION IS GIVEN ONLY 1 SECONDARY	Sixpak
ENERGY (NEP=1) MAY BE GIVEN = A NORMALIZED DISTRIBUTION FOR A	Sixpak Sixpak
SINGLE DISCRETE EMISSION ENERGY. ALL OF THE ENDF/B-VI DATA AS	Sixpak
OF JANUARY 1992 CONFORM TO THESE LIMITATIONS.	Sixpak
	Sixpak
SINCE THE FLAG NA, TO INDICATE ISOTROPIC DISTRIBUTIONS, IS ONLY	Sixpak
GIVEN FOR EACH SECONDARY ENERGY (EP) THE PROGRAM CANNOT DECIDE	Sixpak
IN ADVANCE WHETHER OR NOT THE DISTRIBUTION WILL BE ISOTROPIC AT ALL INCIDENT ENERGIES. THEREFORE ISOTROPIC DISTRIBUTIONS	Sixpak Sixpak
WILL BE OUTPUT EITHER: LANG = 1 - AS 1 LEGENDRE COEFFICIENT = 0.0	Sixpak
OR LANG = NOT 1 - AS A 2 POINT ANGULAR DISTRIBUTION AT COS = -1.0	Sixpak
AND +1.0 WITH BOTH VALUES EQUAL TO 0.5 (A NORMALIZED ISOTROPIC	Sixpak
DISTRIBUTION).	Sixpak
	Sixpak
DISCRETE PHOTONS ARE OUTPUT IN MF=15 AS 3 POINT DISTRIBUTIONS	Sixpak
WITH SECONDARY ENERGY POINTS AT EP-DEP, EP, EP+DEP, WHERE DEP=0.001*EP. THE VALUES AT EP-DEP AND EP+DEP ARE 0.0, AND	Sixpak
AT EP THE VALUE IS 1000.0/EP TO NORMALIZE THE DISTRIBUTION.	Sixpak Sixpak
AT HE THE VALUE TO TOUT, O/ HE TO NORWENDED THE DISTRIBUTION.	Sixpak
LAW=2	Sixpak
=====	Sixpak
NO LIMITATION ON REPRESENTATIONS.	Sixpak
	Sixpak
LAW=3	Sixpak Sixpak
NO LIMITATION ON REPRESENTATIONS.	Sixpak
no amiliation on retreepartitions.	Sixpak
LAW=4	Sixpak
=====	Sixpak
NO OUTPUT - INCIDENT NEUTRON - EMITTED PHOTON OR NEUTRON	Sixpak
REACTIONS ARE NOT EXPECTED.	Sixpak
LAW=5	Sixpak Sixpak
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NO OURDING THAT PART NEWFOUND THAT THE PROPERTY OF MENTERS OF	
NO OUTPUT - INCIDENT NEUTRON - EMITTED PHOTON OR NEUTRON	Sixpak Sixpak
REACTIONS ARE NOT EXPECTED.	Sixpak Sixpak Sixpak
	Sixpak
	Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 =====	Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 =====	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P.	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER,	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY	Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER,	Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY ENERGIES.	Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY	Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ====== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY ENERGIES. 2) LINEAR = FOR EACH INCIDENT COSINE THE INTERPOLATION LAW BETWEEN SECONDARY ENERGIES MUST BE LINEAR.	Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY ENERGIES. 2) LINEAR = FOR EACH INCIDENT COSINE THE INTERPOLATION LAW BETWEEN SECONDARY ENERGIES MUST BE LINEAR. THESE 2 PRESENTATIONS ARE THE ONLY ONES PRESENTED IN ENDF/B-VI	Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ====== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ====== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY ENERGIES. 2) LINEAR = FOR EACH INCIDENT COSINE THE INTERPOLATION LAW BETWEEN SECONDARY ENERGIES MUST BE LINEAR. THESE 2 PRESENTATIONS ARE THE ONLY ONES PRESENTED IN ENDF/B-VI AS OF JANUARY 1992 - SO THIS PROGRAM CAN TRANSLATED ALL LAW=7	Sixpak
REACTIONS ARE NOT EXPECTED. LAW=6 ===== NO OUTPUT - ENDF/B-VI ONLY INCLUDES 1 SECTION OF THIS TYPE OF DATA FOR (N,D) 2N,P. LAW=7 ===== FOR EACH INCIDENT ENERGY THE REPRESENTATION MUST BE EITHER, 1) SQUARE = FOR EACH INCIDENT COSINE EXACTLY THE SAME SECONDARY ENERGIES. 2) LINEAR = FOR EACH INCIDENT COSINE THE INTERPOLATION LAW BETWEEN SECONDARY ENERGIES MUST BE LINEAR. THESE 2 PRESENTATIONS ARE THE ONLY ONES PRESENTED IN ENDF/B-VI	Sixpak
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THAN THE INCIDENT ENERGY. FOR A PROCESS SUCH AS ELASTIC SCATTERING Sixpak WHERE FOR HEAVY MATERIALS THE SECONDARY ENERGY, EP, WILL ALWAYS Sixpak BE A LARGE FRACTION OF THE INCIDENT ENERGY, THIS ASSUMPTION IS Sixpak VALID. HOWEVER, FOR THE TYPICAL REACTIONS INCLUDED IN MF=6 THIS Sixpak IS NOT ALWAYS TRUE - IN MANY OF THESE CASES THE SECONDARY ENERGY Sixpak CAN EXTEND ALL THE WAY DOWN TO ZERO, AND IN PARTICULAR IT CAN Sixpak BE SMALL COMPARED TO THE CENTER-OF-MASS ENERGY - WHICH MAKES THE Sixpak TRANSFORMATION FROM CENTER-OF-MASS TO LAB IMPORTANT. THEREFORE Sixpak GENERALLY TO TREAT MF=6 DATA WE MUST CONSIDER THIS TRANSFORMATION. Sixpak Sixpak THE FOLLOWING DISCUSSING ONLY APPLIES TO SPECTRA THAT MAY BE Sixpak OUTPUT IN MF=5 = ONLY DATA FOR NEUTRONS INCIDENT AND EMITTED -Sixpak IN PARTICULAR THE FOLLOWING DEFINITIONS ARE NOT GENERAL - THEY Sixpak ARE ONLY VALID FOR INCIDENT AND EMITTED NEUTRONS. Sixpak Sixpak DOUBLE DIFFERENTIAL DATA IN MF=6 MAY BE GIVEN IN EITHER THE LAB Sixpak OR C.M. SYSTEM. SIMILARLY ANGULAR DISTRIBUTIONS IN MF=4 MAY BE Sixpak GIVEN IN EITHER THE LAB OR C.M. SYSTEM. IN CONTRAST ENERGY Sixpak SPECTRA IN MF=5 CAN ONLY BE GIVEN IN THE LABORATORY SYSTEM. Sixpak Sixpak THE ANGULAR DISTRIBUTIONS OUTPUT BY THIS CODE IN MF=4 ARE IN THE Sixpak SAME SYSTEM IN WHICH THEY ARE GIVEN IN MF=6 - EITHER LAB OR Sixpak CENTER-OF-MASS SYSTEM. Sixpak Sixpak THE ENERGY SPECTRA OUTPUT BY THIS CODE IN MF=5 MUST BE IN THE LAB Sixpak SYSTEM - THIS IS THE ONLY ALLOWED FORM FOR MF=5 DATA. Sixpak Sixpak FOR MF=6 SPECTRA GIVEN IN THE LAB SYSTEM THIS MERELY REQUIRES Sixpak COPYING THE GIVEN SPECTRA TO MF=5 OUTPUT. Sixpak Sixpak FOR MF=6 SPECTRA GIVEN IN THE CENTER-OF-MASS SYSTEM ONLY FIRST Sixpak ORDER CORRECTIONS IN THE SPECTRA AND USED AND THEY ARE THEN Sixpak OUTPUT IN MF=5 AS IN THE LAB SYSTEM - THE FIRST ORDER CORRECTIONS Sixpak ARE DESCRIBED BELOW. Sixpak Sixpak DEFINING, Sixpak MM = CENTER OF MASS MOTION Sixpak CM = OUTGOING (EMITTED) PARTICLE IN CENTER OF MASS Sixpak = OUTGOING (EMITTED) PARTICLE IN LAB Sixpak LAB = CM SCATTERING ANGLE RELATIVE TO INCIDENT DIRECTION Sixpak COS(CM) = COSINE OF THE CM SCATTERING ANGLE Sixpak Sixpak FOR NEUTRONS INCIDENT WITH AN ENERGY, E, AND THEREFORE A SPEED, Sixpak Sixpak VN(E) = 2*SQRT(E)/MASS(IN)Sixpak Sixpak THE CENTER-OF-MASS SPEED IS GIVEN BY, Sixpak Sixpak V(MM) = VN(E)/(1 + A)Sixpak Sixpak AND THE CENTER OF MASS ENERGY BY, Sixpak Sixpak E(MM) = 1/2*MASS(IN)*V(MM)**2Sixpak = 1/2*MASS(IN)*VN(E)**2/(1 + A)**2Sixpak = E/(1 + A)**2Sixpak Sixpak FOR DISTRIBUTIONS GIVEN IN MF=6 IN THE CM, THE SPEED, V(CM), Sixpak SHOULD BE VECTORIALLY ADDED TO THAT OF OUTGOING PARTICLES TO Sixpak DEFINE THE OUTGOING PARTICLES LAB VELOCITY, AND IN TURN IT'S Sixpak Sixpak ENERGY. Sixpak V(LAB)*COS(LAB) = V(MM) + V(CM)*COS(CM)Sixpak V(LAB)*SIN(LAB) =V(CM)*SIN(CM) Sixpak Sixpak V(LAB)**2 = V(MM)**2 + V(CM)**2 + 2*COS(CM)*V(MM)*V(CM)Sixpak Sixpak EP(LAB) = 0.5*MASS(OUT)*V(LAB)**2Sixpak Sixpak = E(MM) + EP(CM) + 2*COS(CM)*SQRT(E(MM)*EP(CM))Sixpak Sixpak

WE CAN ALSO DEFINE THE REVERSE TRANSFORMATION USING,

```
Sixpak
V(CM)*COS(CM) = V(LAB)*COS(LAB) - V(MM)
                                                                    Sixpak
V(CM)*SIN(CM) = V(LAB)*SIN(LAB)
                                                                    Sixpak
                                                                    Sixpak
V(CM)**2 = V(MM)**2 + V(LAB)**2 - 2*COS(LAB)*V(MM)*V(LAB)
                                                                    Sixpak
                                                                    Sixpak
EP (CM)
        = 0.5*MASS(OUT)*V(CM)**2
                                                                    Sixpak
                                                                    Sixpak
         = E (MM) + EP (LAB) - 2*COS (LAB) *SQRT (E (MM) *EP (LAB))
                                                                    Sixpak
                                                                    Sixpak
WE CAN DEFINE COS(LAB) FROM THE RELATIONSHIP,
                                                                    Sixpak
                                                                    Sixpak
V(LAB)*COS(LAB) = V(MM) + V(CM)*COS(CM)
                                                                    Sixpak
                                                                    Sixpak
COS (LAB)
                = [V(MM) + V(CM) * COS(CM)]/V(LAB)
                                                                    Sixpak
                                                                   Sixpak
                 [V(MM) + V(CM)*COS(CM)]
                                                                    Sixpak
COS (LAB)
                                                                    Sixpak
                 SQRT [V (MM) **2+V (CM) **2+2*COS (CM) *V (MM) *V (CM) ]
                                                                    Sixpak
                                                                    Sixpak
OR COS(CM) FROM THE RELATIONSHIP,
                                                                    Sixpak
                                                                    Sixpak
V(CM)*COS(CM) = V(LAB)*COS(LAB) - V(MM)
                                                                    Sixpak
                                                                    Sixpak
COS (CM)
                = [V(LAB) * COS(LAB) - V(MM)]/V(CM)
                                                                    Sixpak
                                                                    Sixpak
                 [V(LAB)*COS(LAB) - V(MM)]
                                                                    Sixpak
COS (CM)
                =-----
                                                                   Sixpak
                 SQRT[V(LAB)**2+V(CM)**2-2*COS(LAB)*V(LAB)*V(MM)] Sixpak
                                                                    Sixpak
THE JACOBIAN CAN BE DEFINED FROM,
                                                                    Sixpak
                                                                    Sixpak
V(LAB)*COS(LAB) = V(MM) + V(CM)*COS(CM)
                                                                    Sixpak
                                                                    Sixpak
J = D[COS(CM)]/D[COS(LAB)] = V(LAB)/V(CM)
                                                                    Sixpak
                           = SQRT[EP(LAB)/EP(CM)]
                                                                    Sixpak
                                                                    Sixpak
WITH THESE DEFINITIONS OF EP(LAB) AND COS(LAB) IN TERMS OF E(MM),
                                                                    Sixpak
EP(CM) AND COS(CM) IT IS POSSIBLE TO PERFORM A POINT-BY-POINT
                                                                    Sixpak
TRANSFORMATION OF DISTRIBUTIONS FROM THE CM TO LAB SYSTEM USING
                                                                    Sixpak
THESE DEFINITIONS - OR IF WE WISHED WE COULD PERFORM THE REVERSE
                                                                    Sixpak
TRANSFORMATION USING THE ABOVE RELATIONSHIPS AND THE IDENTITY,
                                                                    Sixpak
                                                                    Sixpak
F(E, EP(LAB), COS(LAB))*D(COS(LAB))=F(E, EP(CM), COS(CM))*D(COS(CM))
                                                                    Sixpak
                                                                    Sixpak
THIS IS NOT WHAT WILL BE DONE HERE, SINCE WE WILL ONLY BE
                                                                   Sixpak
INTERESTED IN THE ZEROTH ORDER MOMENTS OF THESE DISTRIBUTIONS,
                                                                    Sixpak
BUT WE WILL BE INTERESTED IN DEFINING THOSE MOMENTS IN THE
                                                                    Sixpak
LAB SYSTEM IN TERMS OF MF=6 SPECTRA GIVEN IN THE CM SYSTEM USING.
                                                                   Sixpak
                                                                    Sixpak
F(E,EP(LAB),COS(LAB)) = F(E,EP(CM),COS(CM))*J
                                                                    Sixpak
                                                                    Sixpak
                                                                   Sixpak
THE LIMITS OF EP(LAB) ARE DEFINED BY SETTING COS(CM) = +1 OR -1.
                                                                    Sixpak
EP (LAB)
         = (SQRT(EP(CM)) + SQRT(E(MM)))**2 FOR COS(CM) = +1
                                                                   Sixpak
         = (SQRT(EP(CM)) - SQRT(E(MM)))**2 FOR COS(CM) = -1
                                                                    Sixpak
                                                                    Sixpak
IN THIS FORM WE CAN SEE THAT AS LONG AS THE SECONDARY ENERGY IN
                                                                    Sixpak
THE CENTER-OF-MASS SYSTEM, EP(CM), IS MUCH LARGER THAN THE
                                                                    Sixpak
ENERGY OF THE CENTER-OF-MASS, E (MM), THE CENTER-OF-MASS AND LAB
                                                                   Sixpak
ENERGIES WILL BE ALMOST EQUAL - SIMILARLY FOR THE COSINE, IN
                                                                    Sixpak
THIS CASE COS(LAB) AND COS(CM) WILL BE ALMOST EQUAL - HOWEVER,
                                                                    Sixpak
FOR THE MF=6 DATA WE CANNOT ASSUME THAT THIS IS TRUE.
                                                                    Sixpak
                                                                    Sixpak
TO FIRST ORDER THE ANGULAR DEPENDENCE CAN BE IGNORED,
                                                                    Sixpak
                                                                    Sixpak
EP(LAB)
         = E(MM) + EP(CM)
                                                                    Sixpak
                                                                    Sixpak
ALL THIS SAYS IS THAT TO FIRST ORDER THE EFFECT OF TRANSFORMING
                                                                   Sixpak
FROM THE CM TO LAB SYSTEM IS TO INCREASE THE ENERGY OF THE
                                                                    Sixpak
EMITTED PARTICLE IN THE CENTER-OF-MASS SYSTEM BY THE ENERGY OF
                                                                    Sixpak
```

THE CENTER-OF-MASS TO DEFINE THE LAB ENERGY.

NOT ONLY THE ENERGY, BUT ALSO THE SPECTRA MUST BE TRANSFORMED. STARTING FROM THE DOUBLE DIFFERENTIAL DATA IN THE LAB SYSTEM, F(E,EP,COS(LAB)), WE CAN DEFINE THE LAB SCALAR SPECTRUM AS,

GO(E,EP) = INTEGRAL F(E,EP,COS(LAB))*D(COS(LAB))

THIS IS THE NORMAL CALCULATION DEFINED ABOVE AND USED FOR DATA GIVEN IN THE LAB SYSTEM.

STARTING FROM DATA IN THE CENTER OF MASS SYSTEM F(E,EP,COS(CM)), WE CAN USE THE RELATIONSHIP,

F(E,EP,COS(LAB))*D(COS(LAB)) = F(E,EP,COS(CM))*J*D(COS(LAB))

J = SQRT (EP (LAB) /EP (CM)) - THE JACOBIAN

= E(MM)/EP(CM) + 1 + 2*COS(CM)*SQRT(E(MM)/EP(CM))

AS IN THE CASE OF THE ENERGY, IN THIS FORM WE CAN SEE THAT AS LONG AS THE SECONDARY ENERGY IN THE CENTER-OF-MASS SYSTEM, EP(CM), IS LARGE COMPARED TO THE CENTER-OF-MASS ENERGY, E(MM), THE JACOBIAN IS ESSENTIALLY UNITY AND THE CENTER-OF-MASS AND LAB SPECTRA WILL BE VERY SIMILAR - AGAIN, GENERALLY WE CANNOT ASSUME THAT THIS IS TRUE FOR THE MF=6 SPECTRA.

THEREFORE WE CAN ALSO DEFINE THE LAB SCALAR SPECTRUM IN TERMS OF THE CM SPECTRUM IN THE FORM,

GO(E,EP) = INTEGRAL F(E,EP,COS(CM))*J*D(COS(LAB))

CONSISTENT WITH THE ABOVE ASSUMPTION THAT THE ANGULAR DEPENDENCE OF EP(LAB) CAN BE IGNORED THE JACOBIAN WILL NOT BE USED IN PERFORMING THESE INTEGRALS - IN WHICH CASE THE INTEGRAL REDUCES TO EXACTLY THE SAME FORM AS IF THE DATA WERE IN THE LAB SYSTEM.

IT SHOULD BE NOTED THAT SINCE IN THIS CASE THE MF=4 ANGULAR DISTRIBUTIONS ARE GIVEN IN THE CM SYSTEM AND WHEN USED IN ANY APPLICATION THEY WILL BE TRANSFORMED TO THE LAB SYSTEM - WHEN THIS IS DONE THE JACOBIAN WILL BE APPLIED.

IN THIS CODE WHERE WE ARE MOSTLY CONCERNED WITH CONSERVING THE NUMBER OF EMITTED PARTICLES AND AVERAGE ENERGIES THE NEUTRON SPECTRA OUTPUT IN MF=5 WILL NOT BE COMPLETELY CONVERTED TO THE LAB SYSTEM - ONLY FIRST ORDER CORRECTIONS WILL BE INCLUDED BY INCREASING THE EMITTED PARTICLE ENERGY BY THE CENTER OF MASS ENERGY, I.E., FOR A CENTER OF MASS SPECTRUM TABULATED AT CENTER OF MASS ENERGIES EP(CM) THESE WILL ALL BE UNIFORMLY INCREASED BY E(MM) TO ACCOUNT FOR THE CENTER OF MASS MOTION - THE SPECTRA WILL NOT BE MODIFIED BY THE JACOBIAN FACTOR SQRT(EP(LAB)/EP(CM)) SINCE THIS WOULD REQUIRE A DETAILED TRANSFORMATION IN ENERGY AND COS(THETA) SPACE - WHICH IS JUDGED NOT TO BE WORTH PERFORMING WITHIN THE LIMITS OF WHERE THE OUTPUT FROM THIS CODE IS INTENDED TO BE USED.

SINCE THE ANGULAR DISTRIBUTION IS ALWAYS OUTPUT IN THE SAME SYSTEM AS WHICH IT IS GIVEN IN MF=6, NO TRANSFORMATION IS REQUIRED FOR THE MF=4 OUTPUT.

WHEN USED IN LOW ENERGY APPLICATIONS (E.G., FISSION REACTORS) THE HIGH ENERGY SPECTRA PRESENTED IN MF=6 WILL BE MOSTLY IMPORTANT SIMPLY IN CONSERVING PARTICLES, (E.G., AS IN (N,2N)) AND ENERGY AND THE DETAILS OF THE CORRELATION AND GROSS ENERGY SPECTRA WILL NOTE PLAY THAT IMPORTANT A ROLE. IN THIS CASE THE SPECTRA OUTPUT BY THIS PROGRAM IN MF=5 SHOULD BE ADEQUATE.

PLOTTAB FORMATTED OUTPUT

THIS PROGRAM CONTAINS ROUTINES TO PRODUCE OUTPUT THAT CAN BE USED AS INPUT TO THE PLOTTAB CODE TO OBTAIN GRAPHIC RESULTS.

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mii mora	DOMESTI	EG ADE DEGICIED ONLY EGD HGE DY MHE AUMHOD MO GHEGY
HIS C	CODE. UN	ES ARE DESIGNED ONLY FOR USE BY THE AUTHOR TO CHECK SERS ARE ASKED NOT TO ACTIVATE OR TRY TO USE THESE LESS YOU COMPLETELY UNDERSTAND THIS CODE THE RESULTS
AN BE	E UNREL	IABLE IF YOU ACTIVATE THESE ROUTINES.
NPUT	FILES	
	DESCRI	
2 10		LINES (BCD - 80 CHARACTERS/RECORD) AL ENDF/B DATA (BCD - 80 CHARACTERS/RECORD)
-0	OKIGIN	and harry barra (bob of characteria) and control
UTPUI	r FILES	
NIT	DESCRI	PTION
		REPORT (BCD - 120 CHARACTERS/RECORD)
11 12		DATA MF=4 (BCD - 80 CHARACTERS/RECORD) DATA MF=5 (BCD - 80 CHARACTERS/RECORD)
14		DATA MF=15 (BCD - 80 CHARACTERS/RECORD)
17		DATA MF=12 (BCD - 80 CHARACTERS/RECORD)
18		DATA MF=14 (BCD - 80 CHARACTERS/RECORD)
15	•	B INPUT PARAMETERS (BCD - 80 CHARACTERS/RECORD)
16		B FORMATTED OUTPUT (BCD - 80 CHARACTERS/RECORD)
CRATO	CH FILE:	s
ONE		
PTION	NAL STA	NDARD FILE NAMES (SEE SUBROUTINE FILIO1 AND FILIO2)
NIT	FILE N	
2 3	SIXPAK	
3 10	SIXPAK ENDFB.	
11	ENDFB.	
12	ENDFB.	
14	ENDFB.	
17	ENDFB.	
18	ENDFB.	M14
4 -		
15	PLOTTA	B. INP
15 16		
-	PLOTTA	
16	PLOTTA:	B.CUR
16	PLOTTA	B.CUR
16 NPUT	PLOTTA	B.CUR TERS
16 NPUT	PLOTTAL PLOTTAL PARAMET	B.CUR
16 NPUT	PLOTTAL PLOTTAL PARAME COLS.	B.CUR TERS DESCRIPTION
NPUT	PLOTTAL PLOTTAL PARAME COLS.	B.CUR TERS DESCRIPTION
NPUT INE INE	PARAME COLS.	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME
16 NPUT =====	PARAME COLS. 1-72	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN)
NPUT INE INE	PARAME COLS. 1-72 1-6 9-11	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE
16 NPUT ==== INE 1	PARAME* COLS. 1-72 1-6 9-11 12-17	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MT FOR REQUESTED RANGE
NPUT ===== INE 1 2-N	PARAME' COLS. 1-72 1-6 9-11 12-17 20-22	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE
NPUT ===== INE 1 2-N	PARAME* COLS. 1-72 1-6 9-11 12-17 20-22 THE DE:	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL
NPUT ===== INE 1 2-N	PARAME* COLS. 1-72 1-6 9-11 12-17 20-22 THE DE:	B.CUR TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE
NPUT ===== INE 1 2-N EAVE	PARAMETERS	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN).
NPUT INE 1 2-N EAVE HEN U	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: JSE THE 100 MA	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS
NPUT ===== INE 1 2-N EAVE HEN U	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: JSE THE 100 MANATED B:	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY
NPUT INE 1 2-N EAVE HEN U	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: JSE THE 100 MANATED B:	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS
NPUT INE 1 2-N EAVE HEN U P TO ERMIN	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: USE THE 100 MA: NATED B: ALL DA:	DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY TA WILL BE PROCESSED.
16 INPUT INE 1 2-N EAVE HEN UP TO EERMING	PARAMETO EN PARAME	DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY TA WILL BE PROCESSED.
16 INPUT INE 1 2-N LEAVE THEN UP TO TERMINESLANK	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: USE THE 100 MA: NATED B: ALL DA:	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY TA WILL BE PROCESSED.
16 INPUT INE 1 2-N LEAVE THEN U TO TERMIN	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: USE THE 100 MA: NATED B: ALL DA: LE INPU	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY TA WILL BE PROCESSED. T NO. 1 MF=6 DATA ON AN ENDF/B TAPE. USE THE STANDARD INPUT
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INPUT INE I 2-N LEAVE THEN U TERMIN SLANK CXAMPI PROCES	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: USE THE 100 MA: NATED B: ALL DA: LE INPU: SS ALL I FILENAME FY THE :	TERS DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY TA WILL BE PROCESSED. T NO. 1 MF=6 DATA ON AN ENDF/B TAPE. USE THE STANDARD INPUT
INPUT INE I 2-N LEAVE THEN U TERMIN SLANK EXAMPI PROCES	PARAME: PARAME: COLS. 1-72 1-6 9-11 12-17 20-22 THE DE: USE THE 100 MA: NATED B: ALL DA: LE INPU: SS ALL I FILENAME FY THE :	DESCRIPTION ENDF/B INPUT DATA FILENAME (STANDARD OPTION = ENDFB.IN) MINIMUM MAT FOR REQUESTED RANGE MINIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE MAXIMUM MAT FOR REQUESTED RANGE FINITION OF THE FILENAME BLANK - THE PROGRAM WILL STANDARD FILENAME (ENDFB.IN). T/MT RANGES MAY BE SPECIFIED. THE LIST OF RANGES IS Y A BLANK LINE. IF THE FIRST INPUT LINE IS COMPLETELY TA WILL BE PROCESSED. T NO. 1 MF=6 DATA ON AN ENDF/B TAPE. USE THE STANDARD INPUT E ENDFB.IN IN THIS CASE THE USER CAN EITHER EXPLICITLY

1 1 9999 999	Si
(BLANK LINE, TERMINATES REQUEST LIST)	Si
	Si
OR BY INPUTTING 2 BLANK LINE = PROCESS EVERYTHING.	Si
	Si
EXAMPLE INPUT NO. 2	Si
	Si
PROCESS BE-9, MAT=425, MT=16. READ THE DATA FROM ENDFB6\BE9.	Si
IN THIS CASE THE FOLLOWING 3 INPUT LINES ARE REQUIRED,	Si
	Si
ENDFBB6\BE9	Si
425 16 425 16	Si
(BLANK LINE, TERMINATES REQUEST LIST)	Si
	Si
EXAMPLE INPUT NO. 3	Si
	Si
PROCESS ALL MT=16 (N,2N) DATA. THIS CAN BE DONE BY SPECIFYING THE	Si
MAXIMUM MAT RANGE = 1 TO 9999, AND MT=16 FOR THE MINIMUM AND	Si
MAXIMUM MT RANGE. READ THE DATA FROM ENDFB6\K300. IN THIS CASE	Si
CASE THE FOLLOWING 3 INPUT LINES ARE REQUIRED,	Si
· · · · · · · · · · · · · · · · · · ·	Si
ENDFB6\K300	Si
1 16 9999 16	Si
(BLANK LINE, TERMINATES REQUEST LIST)	Si
,,, , 	Si
	= Si

Sixpak

ENDFB.IN