



INTERNATIONAL ATOMIC ENERGY AGENCY  
**NUCLEAR DATA SERVICES**

DOCUMENTATION SERIES OF THE IAEA NUCLEAR DATA SECTION

IAEA-NDS- 11

( Rev. 4 )

ENDL-84

The Lawrence Livermore National Laboratory  
Evaluated Nuclear Data Library  
in  
the ENDF-V Format

Contents and Documentation

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Abstract

This document summarizes the contents of the evaluated nuclear data library (ENDL) by the Lawrence Livermore National Laboratory, USA, converted to ENDF-V format. The library contains evaluated data for all significant neutron reactions in the energy range from  $10^{-4}$  ev to 20 Mev for 94 elements or isotopes. The entire library or selective retrievals from it can be obtained on magnetic tape, free of charge, from the IAEA Nuclear Data Section.

May 1985

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IAEA NUCLEAR DATA SECTION, P.O. BOX 100, A-1400 VIENNA

24-Cr- 09 (MAY 1985)	Date:	Lab: LIVERMORE	Author: R.J.HOWERTJN
Ref: UCRL-5400, VOL. 15		Card Images: 1753	
File Type		Reaction Type	O-Value

ENDL-84

This library contains the Evaluated Nuclear Data Library (ENDL) of the Lawrence Livermore National Laboratory as of 1984. The library has been converted to the ENDF-V format by R.J. Howerton, Lawrence Livermore National Laboratory. The library contains evaluated neutron cross sections and differential data and neutron induced photon production data for 94 materials and includes a total of 231348 records. All evaluations are complete, covering all relevant reactions for incident energies from  $10^{-4}$  ev to 20 MeV. For the actual contents of any given evaluation see the enclosed summary listings. All cross sections are represented in tabulated linearly interpolable form; resonance parameters are not used.

ENDL vs. ENDF-V format

The Lawrence Livermore National Laboratory Evaluated Nuclear Data Library is now available in Livermore's ENDL format and in the ENDF-V format. The ENDL format is more general than the ENDF-V format in terms of the forms in which nuclear data can be represented in the ENDL format. As such it is not possible to accurately translate the entire ENDL library to the ENDF-V format. Evaluations which appear in the ENDL library in the ENDL format and not in the ENDF-V format include  $^2\text{H}$  and  $^9\text{Be}$  which use a double differential representation of the secondary energy-angle distribution in the ENDL format, which is not allowed in the ENDF-V format.

Since the contents of the two libraries are not identical the ENDL-84 library does not supersede the ENDL-82 library; they rather complement each other for use in a variety of applications.

ENDL-84 vs. ENDL-78

Since the last version of the ENDL in the ENDF format (i.e. ENDL-78) nine new evaluations have been added to the library, namely:  $^{74}\text{As}$ ,  $^{75}\text{As}$ ,  $^{88}\text{Y}$ ,  $^{89}\text{Y}$ ,  $^{72}\text{Hf}$ ,  $^{209}\text{Bi}$ ,  $^{235}\text{Np}$ ,  $^{236}\text{Np}$  and  $^{238}\text{Np}$ .

ENDF-V Format

The ENDF-V format is described in R. Kinsey: Data formats and procedures for the evaluated nuclear data file ENDF, BNL-NCS-50496 (ENDF-102), 2nd edition, dated October 1979, also available as microfiche IAEA-NDS-10/102, with a Supplement by B.A. Magurno dated November 1983. For a quick reference to the ENDF/B format file (MF) numbers and reaction (MT) numbers see document IAEA-NDS-10.

CONTENTS AND DOCUMENTATION

A documentation of the library contents and summary of evaluation is given in

R.J. Howerton, M.G. MacGregor, Lawrence Livermore Laboratory Report UCRL-50400 Vol. 15, "The LLL Evaluated Nuclear Data Library (ENDL)". See in particular

Part B, Rev.1 (October 1978): Graphs of Cross-Sections from the Library.

Part D, Rev.1 (May 1978): Descriptions of Individual Evaluations for Z=0-98.

Vol.16, Rev.2 (July 1978): Tabular and graphical presentation of 175 neutron group constants derived from the LLL Evaluated Neutron Data Library (ENDL).

For the evaluation of fission neutron yields see also:

R.J. Howerton, Nucl. Sci. and Eng. 62, 438 (1977).

Cross-reference: Also available are the LLNL Evaluated Neutron Activation Cross-Section Library of 1982 (see IAEA-NDS-55) and the LLNL Evaluated Charged-Particle Data Library of 1982 (see IAEA-NDS-56), both in ENDL Transmittal format.

SUMMARY DOCUMENTATION

The following pages summarize the contents of this library by first presenting a table containing a one line summary for each evaluation. This table is followed by detailed tables containing a one line summary for each reaction or data type within each evaluation. In the detailed table for each evaluation the evaluation year (DATE) has been left blank. The ENDL library is periodically reviewed and distributed by R.J. Howerton. All evaluations in the library should be considered to have been reviewed and either found satisfactory, or updated to improve them by the date of distribution. As such the evaluations in this library should be considered to be up to date as of their distribution date 1984.

ACKNOWLEDGEMENTS

The Nuclear Data Section of the IAEA and D.E. Cullen in particular wish to thank R.J. Howerton, Lawrence Livermore National Laboratory for investing the time and effort required to translate ENDL to the ENDF-V format and for making this library available to the IAEA Nuclear Data Section.

LIST OF MATERIALS AND MAT NUMBERS

If not otherwise specified, the evaluations include all neutron cross-sections and differential data and neutron induced photon production data. For further details see the following pages.

Z	E1	A	MAT NO.	SPECIFICATION	Z	E1	A	MAT NO.
1-H	-	1	7801		67-Ho	-165		7854
1-H	-	3	7803	Neutron cross sections only	72-Hf	-0		8305
2-He	-	3	7804		73-Ta	-181		7855
2-He	-	4	7805	Neutron cross sections only	74-W	-0		7856
3-Li	-	6	7806		75-Re	-185		7858
3-Li	-	7	7807		75-Re	-187		7859
4-Be	-	7	7808	Neutron cross sections only	78-Pt	-0		7860
5-B	-	10	7810		79-Au	-197		7861
5-B	-	11	7811		82-Pb	-0		7862
6-C	-	12	7812		83-Bi	-209		8306
7-N	-	14	7813		90-Th	-231		7863
8-O	-	16	7814		90-Th	-232		7864
9-F	-	19	7815		90-Th	-233		7865
11-Na	-	23	7816		92-U	-233		7866
12-Mg	-	0	7817		92-U	-234		7867
13-Al	-	27	7819		92-U	-235		7868
14-Si	-	0	7820		92-U	-236		7869
15-P	-	31	7821		92-U	-237		7870
16-S	-	32	7822		92-U	-238		7871
17-C	-	0	7823		92-U	-239		7872
18-Ar	-	0	7824		92-U	-240		7873
19-K	-	0	7825		93-Np	-235		8307
20-Ca	-	0	7826		93-Np	-236		8308
22-Ti	-	0	7828		93-Np	-237		7874
23-V	-	51	7829		93-Np	-238		8309
24-Cr	-	0	7830		94-Pu	-238		7875
25-Mn	-	55	7831		94-Pu	-239		7876
26-Fe	-	0	7832		94-Pu	-240		7877
27-Co	-	59	7835		94-Pu	-241		7878
28-Ni	-	0	7836		94-Pu	-242		7880
28-Ni	-	58	7837		94-Pu	-243		7881
29-Cu	-	0	7838		95-Am	-241		7882
31-Ga	-	0	7840		95-Am	-242m		7883
33-As	-	74	8301		95-Am	-243		7884
33-As	-	75	8302		96-Cm	-242		7885
39-Y	-	88	8303		96-Cm	-243		7886
39-Y	-	89	8304		96-Cm	-244		7887
40-Zr	-	0	7841		96-Cm	-245		7888
41-Nb	-	93	7843		96-Cm	-246		7889
42-Mo	-	0	7844		96-Cm	-247		7890
47-Ag	-	107	7845		96-Cm	-248		7891
47-Ag	-	109	7846		97-Bk	-249		7892
48-Cd	-	0	7847		98-Cf	-249		7893
50-Sn	-	0	7850		98-Cf	-250		7894
56-Ba	-	138	7851		98-Cf	-251		7895
63-Eu	-	0	7852		98-Cf	-252		7896
64-Gd	-	0	7853		Fiss. Prod			7897

Note: MAT=7897 is a crude representation of a "typical" fission product which should be used with care.

1-H - 1g	Mat. No: 7801 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 383	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section (n,p) radiative capture cross section <0> : Lab:average elastic scattering cosine, <>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	2.22470+ 6
			Angular dist. of secondary neutrons	Elastic scattering cross section	
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	2.22470+ 6
			Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	2.22470+ 6
			Energy dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	2.22470+ 6
1-H - 3g	Mat. No: 7803 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 383	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section direct (n,2n) cross section <0> : Lab:average elastic scattering cosine, <>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	-6.26000+ 6
			Angular dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section	-6.26000+ 6
			Energy dist. of secondary neutrons	direct (n,2n) cross section	-6.26000+ 6
2-He - 3g	Mat. No: 7804 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 319	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section (n,p) radiative capture cross section (n,b) cross section (n,d) cross section <0> : Lab:average elastic scattering cosine, <>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	2.05780+ 7 -3.50000+ 5 -3.27000+ 6
			Angular dist. of secondary neutrons	Elastic scattering cross section	
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	2.05780+ 7
			Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	2.05780+ 7
			Energy dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	2.05780+ 7
2-He - 4g	Mat. No: 7805 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 350	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section <0> : Lab:average elastic scattering cosine, <>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	
			Angular dist. of secondary neutrons	Elastic scattering cross section	
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	
			Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	
			Energy dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	
3-L1 - 6g	Mat. No: 7806 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 668	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section 2.18000+ 6 Ev (n,n') Level (n,n) radiative capture cross section (n,b) cross section (n,t) cross section <0> : Lab:average elastic scattering cosine, <>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	-2.18000+ 6 -3.69800+ 6 -1.49800+ 6 -2.18000+ 6 -2.26000+ 6 -2.73000+ 6 4.79000+ 6
			Angular dist. of secondary neutrons	Elastic scattering cross section (n,n') cross section 2.18000+ 6 Ev (n,n') Level	-1.46000+ 6 -2.16000+ 6
			Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') cross section	-3.69800+ 6 -1.44300+ 6
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	7.26000+ 6
			Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	7.26000+ 6
			Energy dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	7.26000+ 6

3-11-79	Mat. No: 7807 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 602	
	File Type	Reaction Type	Q-value
	General Information	Descriptive data and Dictionary	
	Resonance parameter data	Resonance information	
	Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section 4.77400+ 5 Ev (n,n') Level (n,g) radiative capture cross section	-4.77400+ 5 -7.26000+ 6 -2.47000+ 6 -4.77400+ 5 2.03000+ 6
		<ux> Lab:average elastic scattering cosine. <ex> average log energy decrement for elastic scat. <gx> : <ee>/<ee>	
	Angular dist. of secondary neutrons	Elastic scattering cross section (n,n') cross section 4.77400+ 5 Ev (n,n') Level direct (n,2n) cross section (n,n') cross section 4.77400+ 5 Ev (n,n') Level (n,g) radiative capture cross section	-2.47000+ 6 -4.77400+ 5 -7.26000+ 6 -2.47000+ 6 -4.77400+ 5 2.03000+ 6
		<ux> Lab:average elastic scattering cosine. <ex> average log energy decrement for elastic scat. <gx> : <ee>/<ee>	
	Energy dist. of secondary neutrons		
	Photon prod.Multiplicities(from neut.reactions)		
	Angular dist. of Photons (from neut.reactions)		
	Energy dist. of Photons (from neut.reactions)		
4-Ba-79	Mat. No: 7808 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 273	
	File Type	Reaction Type	Q-value
	General Information	Descriptive data and Dictionary	
	Resonance parameter data	Resonance information	
	Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section (n,p) cross section (n,a) cross section	
		<ux> Lab:average elastic scattering cosine. <ex> average log energy decrement for elastic scat. <gx> : <ee>/<ee>	1.65000+ 6 1.89920+ 7
	Angular dist. of secondary neutrons	Elastic scattering cross section	
5-B - 10g	Mat. No: 7810 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 369	
	File Type	Reaction Type	Q-value
	General Information	Descriptive data and Dictionary	
	Resonance parameter data	Resonance information	
	Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n' d2a) cross section 7.17000+ 5 Ev (n,n') Level 1.74000+ 6 Ev (n,n') Level 2.15400+ 6 Ev (n,n') Level 3.59000+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,a) cross section (n,d2a) cross section	-7.17000+ 5 -8.15800+ 6 -6.02000+ 6 -7.17000+ 5 -1.74000+ 6 -2.15400+ 6 -3.59000+ 6 1.14500+ 7 2.80000+ 6 3.30000+ 6
		<ux> Lab:average elastic scattering cosine. <ex> average log energy decrement for elastic scat. <gx> : <ee>/<ee>	
	Angular dist. of secondary neutrons	Elastic scattering cross section 7.17000+ 5 Ev (n,n') Level 1.74000+ 6 Ev (n,n') Level 2.15400+ 6 Ev (n,n') Level 3.59000+ 6 Ev (n,n') Level	-7.17000+ 5 -1.74000+ 6 -2.15400+ 6 -3.59000+ 6
	Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n' d2a) cross section (n,n') to the continuum	-8.15800+ 6 -6.02000+ 6
	Photon prod.Multiplicities(from neut.reactions)	7.17000+ 5 Ev (n,n') Level 1.74000+ 6 Ev (n,n') Level 2.15400+ 6 Ev (n,n') Level 3.59000+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section	-7.17000+ 5 -1.74000+ 6 -2.15400+ 6 -3.59000+ 6 1.14500+ 7 2.80000+ 6
	Angular dist. of Photons (from neut.reactions)	(n,a) cross section	
	Energy dist. of Photons (from neut.reactions)	7.17000+ 5 Ev (n,n') Level 1.74000+ 6 Ev (n,n') Level 2.15400+ 6 Ev (n,n') Level 3.59000+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section	-7.17000+ 5 -1.74000+ 6 -2.15400+ 6 -3.59000+ 6 1.14500+ 7 2.80000+ 6
		<ux> Lab:average elastic scattering cosine. <ex> average log energy decrement for elastic scat. <gx> : <ee>/<ee>	
		(n,a) cross section	

5-B - 11g	Mat.No: 7811 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 630	
File Type		Reaction Type	Q-Value
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section 2.14000+ 6 Ev (n,n') Level 4.46000+ 6 Ev (n,n') Level 5.03000+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,t) cross section (n,a) cross section  <u> Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <e>/<u>	-2.14000+ 6 -1.14000+ 7 -2.14000+ 6 -4.46000+ 6 -5.03000+ 6  -3.37000+ 6 -1.07200+ 7 -9.56000+ 6 -6.64000+ 6
Angular dist. of secondary neutrons		Elastic scattering cross section 2.14000+ 6 Ev (n,n') Level 4.46000+ 6 Ev (n,n') Level 5.03000+ 6 Ev (n,n') Level  direct (n,2n) cross section (n,n') to the continuum  (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-2.14000+ 6 -4.46000+ 6 -5.03000+ 6  -1.14000+ 7
Energy dist. of secondary neutrons		direct (n,2n) cross section 2.14000+ 6 Ev (n,n') Level 4.46000+ 6 Ev (n,n') Level 5.03000+ 6 Ev (n,n') Level  (n,n') to the continuum  (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-1.14000+ 7 -2.14000+ 6 -4.46000+ 6 -5.03000+ 6  -3.37000+ 6 -1.07200+ 7 -6.64000+ 6
Photon prod. Multiplicities(from neut.reactions)		direct (n,2n) cross section 2.14000+ 6 Ev (n,n') Level 4.46000+ 6 Ev (n,n') Level 5.03000+ 6 Ev (n,n') Level  (n,n') to the continuum  (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-1.14000+ 7 -2.14000+ 6 -4.46000+ 6 -5.03000+ 6  -3.37000+ 6 -1.07200+ 7 -6.64000+ 6
Angular dist. of Photons (from neut.reactions)		direct (n,2n) cross section 2.14000+ 6 Ev (n,n') Level 4.46000+ 6 Ev (n,n') Level 5.03000+ 6 Ev (n,n') Level  (n,n') to the continuum  (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-1.14000+ 7 -2.14000+ 6 -4.46000+ 6 -5.03000+ 6  -3.37000+ 6 -1.07200+ 7 -6.64000+ 6
Energy dist. of Photons (from neut.reactions)		direct (n,2n) cross section 2.14000+ 6 Ev (n,n') Level 4.46000+ 6 Ev (n,n') Level 5.03000+ 6 Ev (n,n') Level  (n,n') to the continuum  (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-1.14000+ 7 -2.14000+ 6 -4.46000+ 6 -5.03000+ 6  -3.37000+ 6 -1.07200+ 7 -6.64000+ 6

6-C - 12g	Mat.No: 7812 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 752	
File Type		Reaction Type	Q-Value
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) (n,n'3a) cross section 4.43000+ 6 Ev (n,n') Level 7.65000+ 6 Ev (n,n') Level 9.63000+ 6 Ev (n,n') Level 1.01000+ 7 Ev (n,n') Level 1.08400+ 7 Ev (n,n') Level 1.18200+ 7 Ev (n,n') Level  (n,g) radiative capture cross section (n,p) cross section (n,d) cross section (n,a) cross section  <u> Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <e>/<u>	-4.43000+ 6 -7.36000+ 6 -4.43000+ 6 -7.65000+ 6 -9.63000+ 6 -1.01000+ 7 -1.08400+ 7 -1.18200+ 7  -4.94600+ 6 -1.25800+ 7 -1.37200+ 7  -5.70000+ 6
Angular dist. of secondary neutrons		Elastic scattering cross section 4.43000+ 6 Ev (n,n') Level 7.65000+ 6 Ev (n,n') Level 9.63000+ 6 Ev (n,n') Level 1.01000+ 7 Ev (n,n') Level 1.08400+ 7 Ev (n,n') Level 1.18200+ 7 Ev (n,n') Level  (n,n'3a) cross section	-4.43000+ 6 -7.65000+ 6 -9.63000+ 6 -1.01000+ 7 -1.08400+ 7 -1.18200+ 7  -7.36000+ 6
Energy dist. of secondary neutrons		(n,g) radiative capture cross section (n,p) cross section (n,d) cross section	4.94600+ 6 -1.25800+ 7 -1.37200+ 7
Photon prod. Multiplicities(from neut.reactions)		Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section (n,p) cross section (n,d) cross section	4.94600+ 6 -1.25800+ 7 -1.37200+ 7
Energy dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section (n,p) cross section (n,d) cross section	4.94600+ 6 -1.25800+ 7 -1.37200+ 7

7-N - 14g	Mat. No.: 7813 Date: 1978-01-15 Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 2846	
	File Type	Reaction Type	
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section 2.31300+ 6 Ev {n,n'} Level 3.94500+ 6 Ev {n,n'} Level 4.91300+ 6 Ev {n,n'} Level 5.69100+ 6 Ev {n,n'} Level 6.83400+ 6 Ev {n,n'} Level 6.19800+ 6 Ev {n,n'} Level 6.44400+ 6 Ev {n,n'} Level 7.02800+ 6 Ev {n,n'} Level 7.96600+ 6 Ev {n,n'} Level (n,g) radiative capture cross section  <u>> Lab: average elastic scattering cosine. <e></e>> average log energy decrement for elastic scat. <q> : <ee></ee> 5.46840+ 6 Ev {n,p} Level 6.09800+ 6 Ev {n,p} Level 6.59000+ 6 Ev {n,p} Level 6.91100+ 6 Ev {n,p} Level (n,p) continuum excited state 6.41200+ 6 Ev {n,d} Level 9.01000+ 6 Ev {n,d} Level 9.23500+ 6 Ev {n,d} Level (n,n) continuum excited state 8.46500+ 6 Ev {n,a} Level (n,a) continuum excited state 2.28270+ 6 Ev {n,a} Level 4.60270+ 6 Ev {n,a} Level 5.17670+ 6 Ev {n,a} Level 6.90070+ 6 Ev {n,a} Level 6.95170+ 6 Ev {n,a} Level 7.95370+ 6 Ev {n,a} Level 8.15370+ 6 Ev {n,a} Level 8.72370+ 6 Ev {n,a} Level 9.08770+ 6 Ev {n,a} Level 9.34210+ 5 Ev {n,a} Level (n,a) continuum	Q-Value -2.31300+ 6 -1.65100+ 6 -2.31300+ 6 -3.94500+ 6 -5.10600+ 6 -5.69100+ 6 -6.83400+ 6 -6.19800+ 6 -6.44400+ 6 -7.02800+ 6 -7.96600+ 6 -1.06300+ 7  -5.46840+ 6 -6.09800+ 6 -6.59000+ 6 -6.91100+ 6 -8.20200+ 6 -8.41200+ 6 -9.01000+ 6 -9.23500+ 6 -10.32600+ 6 -10.41400+ 6 -10.51600+ 6 -12.28270+ 6 -14.60270+ 6 -15.17670+ 6 -16.90070+ 6 -16.95170+ 6 -17.95370+ 6 -18.15370+ 6 -18.72370+ 6 -19.08770+ 6 -19.34210+ 5 -1.58000+ 5
Angular dist. of secondary neutrons		Elastic scattering cross section 2.31300+ 6 Ev {n,n'} Level 3.94500+ 6 Ev {n,n'} Level 4.91300+ 6 Ev {n,n'} Level 5.69100+ 6 Ev {n,n'} Level 6.83400+ 6 Ev {n,n'} Level  6.19800+ 6 Ev {n,n'} Level 6.44400+ 6 Ev {n,n'} Level 7.02800+ 6 Ev {n,n'} Level 7.96600+ 6 Ev {n,n'} Level  direct (n,2n) cross section (n,n') cross section (n,g) radiative capture cross section Non elastic cross section(total-elastic)	-2.31300+ 6 -3.94500+ 6 -4.91300+ 6 -5.69100+ 6 -6.83400+ 6  -6.19800+ 6 -6.44400+ 6 -7.02800+ 6 -7.96600+ 6  -1.05500+ 7 -7.55000+ 6 1.08300+ 7
Energy dist. of secondary neutrons		Non elastic cross section(total-elastic) (n,g) radiative capture cross section Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.08300+ 7 1.08300+ 7
Proton prod. Multiplicities (from neut.reactions)			
Photon prod. cross sections (from neut.reactions)			
Angular dist. of Photons (from neut.reactions)			
Energy dist. of Photons (from neut.reactions)			

8-O - 16g	Mat. No.: 7814 Date: 1978-01-15 Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1227	
	File Type	Reaction Type	
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section 6.10000+ 6 Ev {n,n'} Level 6.00000+ 6 Ev {n,n'} Level 8.88000+ 6 Ev {n,n'} Level 1.11000+ 7 Ev {n,n'} Level (n,n') to the continuum (n,d) cross section  <u>> Lab: average elastic scattering cosine. <e></e>> average log energy decrement for elastic scat. <q> : <ee></ee> 9.63600+ 6 Ev {n,p} Level 9.63600+ 6 Ev {n,p} Level 9.63600+ 6 Ev {n,p} Level 2.21500+ 6 Ev {n,a} Level 2.21500+ 6 Ev {n,a} Level  Elastic scattering cross section 6.10000+ 6 Ev {n,n'} Level 7.00000+ 6 Ev {n,n'} Level 8.88000+ 6 Ev {n,n'} Level 1.11000+ 7 Ev {n,n'} Level  direct (n,2n) cross section (n,n') cross section (n,n') cross section (n,n') to the continuum	Q-Value -6.10000+ 6 -1.56700+ 7 -7.15000+ 6 -11.24000+ 6 -1.03200+ 6 -8.74000+ 6 -8.88000+ 6 -1.11000+ 7  -9.63600+ 6 -9.63600+ 6 -9.63600+ 6 -2.21500+ 6 -2.21500+ 6  -6.10000+ 6 -7.00000+ 6 -8.88000+ 6 -1.11000+ 7  -1.55700+ 7 -7.15000+ 6 -1.21100+ 7
Angular dist. of secondary neutrons		Non elastic cross section(total-elastic)	
Energy dist. of secondary neutrons		Non elastic cross section(total-elastic)	
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)			
Energy dist. of Photons (from neut.reactions)			

9-F - 19g Mat. NO: 7815 LBD: LIVERMORE  
 Date: Author: R J. HOWERTON  
 Ref.: UCRL-5400, VOL. 15 Card Images: 3496

File Type	Reaction Type	Q-Value
<b>General Information</b>		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
	<b>Descriptive data and Dictionary</b>	
	<b>Resonance Information</b>	
	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n'a) cross section (n,n,p) cross section 1.0000+ 5 Ev (n,n') Level 1.9700+ 5 Ev (n,n') Level 1.41700+ 6 Ev (n,n') Level 1.45800+ 6 Ev (n,n') Level 1.55400+ 6 Ev (n,n') Level 2.78900+ 6 Ev (n,n') Level 3.86700+ 6 Ev (n,n') Level 3.99800+ 6 Ev (n,n') Level 4.03200+ 6 Ev (n,n') Level 4.37800+ 6 Ev (n,n') Level 4.45600+ 6 Ev (n,n') Level 4.55700+ 6 Ev (n,n') Level 4.64800+ 6 Ev (n,n') Level 4.68300+ 6 Ev (n,n') Level 5.10600+ 6 Ev (n,n') Level 5.34000+ 6 Ev (n,n') Level 5.46400+ 6 Ev (n,n') Level 5.49900+ 6 Ev (n,n') Level 5.54000+ 6 Ev (n,n') Level 5.63000+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,d) cross section (n,t) cross section (n,a) cross section	-1.10000+ 5 -1.04310+ 7 -4.87800+ 6 -7.99300+ 6 1.10000+ 6 -1.97000+ 6 -1.41700+ 6 -1.45800+ 6 1.55400+ 6 -2.78900+ 6 -3.90700+ 6 -3.99800+ 6 -4.03200+ 6 -4.37800+ 6 -4.45600+ 6 -4.55700+ 6 -4.64800+ 6 -4.68300+ 6 -5.10600+ 6 -5.34000+ 6 -5.46400+ 6 -5.49900+ 6 -5.54000+ 6 -5.63000+ 6 6.60000+ 6 -4.03620+ 6 -5.76800+ 6 -7.55720+ 6 -1.52300+ 6
	cu> Lab: average elastic scattering cosine. cu> average log energy decrement for elastic scat. cg> 1 <ee>/<2e>	
<b>Angular dist. of secondary neutrons</b>		
	Elastic scattering cross section 1.0000+ 5 Ev (n,n') Level 1.9700+ 5 Ev (n,n') Level 1.41700+ 6 Ev (n,n') Level 1.45800+ 6 Ev (n,n') Level 1.55400+ 6 Ev (n,n') Level 2.78900+ 6 Ev (n,n') Level 3.86700+ 6 Ev (n,n') Level 3.99800+ 6 Ev (n,n') Level 4.03200+ 6 Ev (n,n') Level 4.37800+ 6 Ev (n,n') Level 4.55500+ 6 Ev (n,n') Level 4.55700+ 6 Ev (n,n') Level 4.64800+ 6 Ev (n,n') Level 4.68300+ 6 Ev (n,n') Level 5.10600+ 6 Ev (n,n') Level 5.34000+ 6 Ev (n,n') Level 5.42800+ 6 Ev (n,n') Level 5.46400+ 6 Ev (n,n') Level 5.49900+ 6 Ev (n,n') Level 5.54000+ 6 Ev (n,n') Level 5.63000+ 6 Ev (n,n') Level	-1.10000+ 5 -1.97000+ 6 -1.41700+ 6 -1.45800+ 6 -1.55400+ 6 -2.78900+ 6 -3.90700+ 6 -3.99800+ 6 -4.03200+ 6 -4.37800+ 6 -4.55500+ 6 -4.55700+ 6 -4.64800+ 6 -4.68300+ 6 -5.10600+ 6 -5.34000+ 6 -5.42800+ 6 -5.46400+ 6 -5.49900+ 6 -5.54000+ 6 -5.63000+ 6
<b>Energy dist. of secondary neutrons</b>		
	direct (n,2n) cross section (n,n'a) cross section (n,n,p) cross section (n,n') to the continuum	-1.04310+ 7 -4.87800+ 6 -7.99300+ 6
<b>Photon prod. Multiplicities (from neut. reactions)</b>		
<b>Photon prod. cross sections (from neut. reactions)</b>		
<b>Angular dist. of Photons (from neut. reactions)</b>		
<b>Energy dist. of Photons (from neut. reactions)</b>		
	(n,g) radiative capture cross section Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) (n,g) radiative capture cross section Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.60000+ 6 6.60000+ 6 6.60000+ 6 6.60000+ 6

11-Ma- 230	Mo 7816	Lab: LIVERMORE Author: R.J.HOMERTON Ref: UCRL-5400, VOL. 15 Card Images: 200A	File Type	Reaction Type	U-Value
General Information				Descriptive data and Dictionary	
Resonance parameter data				Resonance information	
Neutron cross sections				Total cross section(sum of partials) Elastic scattering cross section: Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n'a) cross section (n,n'b) cross section 4.39200+ 5 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-4.39200+ 5 -1.24343+ 5 -1.04980+ 7 -8.79400+ 6 -4.39200+ 5 6.97000+ 6 -4.59700+ 6 -3.88000+ 6
Angular dist. of secondary neutrons				cus Lab-average elastic scattering cosine, cav-average log energy decrement for elastic scat. <gn : <eey><2e>	
Energy dist. of secondary neutrons				Elastic scattering cross section 4.39200+ 5 Ev (n,n') Level	-4.39200+ 5
Photon prod. Multiplicities (from neut.reactions)				direct (n,2n) cross section (n,n'a) cross section (n,n'b) cross section (n,n') to the continuum	-1.24240+ 7 -1.04980+ 7 -8.79400+ 6
Photon prod. cross sections (from neut.reactions)				(n,g) radiative capture cross section	6.97000+ 6
Angular dist. of Photons (from neut.reactions)				Non elastic cross section(total-elastic)	
Energy dist. of Photons (from neut.reactions)				Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.97000+ 6
				Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.97000+ 6

12 Mo- 09	Mat. No. 7817	Lab: LIVERMORE Author: R.J.HOMERTON Ref: UCRL-5400, VOL. 15 Card Images: 1294	File Type	Reaction Type	U-Value
General Information				Descriptive data and Dictionary	
Resonance parameter data				Resonance information	
Neutron cross sections				Total cross section(sum of partials) Elastic scattering cross section: Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section 1.37000+ 6 Ev (n,n') Level 1.61000+ 6 Ev (n,n') Level 1.83000+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-1.37000+ 6 -7.32400+ 6 -1.37000+ 6 -1.61000+ 6 -1.83000+ 6 9.09200+ 6 9.09200+ 6 2.55600+ 6
Angular dist. of secondary neutrons				cus Lab-average elastic scattering cosine cav-average log energy decrement for elastic scat. <gn : <eey><2e>	
Energy dist. of secondary neutrons				Elastic scattering cross section 1.37000+ 6 Ev (n,n') Level 1.61000+ 6 Ev (n,n') Level 1.83000+ 6 Ev (n,n') Level	-1.37000+ 6 -1.61000+ 6 -1.83000+ 6
Photon prod. Multiplicities (from neut.reactions)				direct (n,2n) cross section (n,n') to the continuum	-7.32400+ 6
Photon prod. cross sections (from neut.reactions)				(n,g) radiative capture cross section	9.09200+ 6
Angular dist. of Photons (from neut.reactions)				Non elastic cross section(total-elastic)	
Energy dist. of Photons (from neut.reactions)				Non elastic cross section(total-elastic) (n,g) radiative capture cross section	9.09200+ 6
				Non elastic cross section(total-elastic) (n,g) radiative capture cross section	9.09200+ 6

13-A1-27g	Mat. No. 7819 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 3549
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,p) cross section 8.43000+ 5 Ev (n,n') Level 1.27300+ 6 Ev (n,n') Level 2.21000+ 6 Ev (n,n') Level 2.73200+ 6 Ev (n,n') Level 2.98000+ 6 Ev (n,n') Level 3.00100+ 6 Ev (n,n') Level 3.67800+ 6 Ev (n,n') Level 3.69600+ 6 Ev (n,n') Level 4.05000+ 6 Ev (n,n') Level 4.40900+ 6 Ev (n,n') Level 4.50800+ 6 Ev (n,n') Level 4.58000+ 6 Ev (n,n') Level 4.81100+ 6 Ev (n,n') Level (n,g) to the continuum (n,g) radiative capture cross section (n,p) cross section (n,d) cross section (n,t) cross section (n,a) cross section  <ü> Lab:average elastic scattering cosine. <ü> average log energy decrement for elastic scat. <ü> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section 8.43000+ 5 Ev (n,n') Level 1.27300+ 6 Ev (n,n') Level 2.21000+ 6 Ev (n,n') Level 2.73200+ 6 Ev (n,n') Level 2.98000+ 6 Ev (n,n') Level 3.00100+ 6 Ev (n,n') Level 3.67800+ 6 Ev (n,n') Level 3.69600+ 6 Ev (n,n') Level 4.05000+ 6 Ev (n,n') Level 4.40900+ 6 Ev (n,n') Level 4.50800+ 6 Ev (n,n') Level 4.58000+ 6 Ev (n,n') Level 4.81100+ 6 Ev (n,n') Level
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,p) cross section (n,n') to the continuum  (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		7.73000+ 6
Photon prod. cross sections (from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) (n,g) radiative capture cross section  Non elastic cross section(total-elastic) (n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		7.73000+ 6

14-S1- 0g	Mat.No: 7820 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1940
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,p) cross section 1.27300+ 6 Ev (n,n') Level 1.77870+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,d) cross section (n,t) cross section  <ü> Lab:average elastic scattering cosine. <ü> average log energy decrement for elastic scat. <ü> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section 1.27300+ 6 Ev (n,n') Level 1.77870+ 6 Ev (n,n') Level
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,p) cross section (n,n') to the continuum  (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		8.47400+ 6
Photon prod. cross sections (from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) (n,g) radiative capture cross section  Non elastic cross section(total-elastic)
Energy dist. of Photons (from neut.reactions)		8.47400+ 6

15-P - 31g	Mat. No: 7821 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 871	Reaction Type  Q-value
	File Type		
General information	Descriptive data and Dictionary		
Resonance parameter data	Resonance information		
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section <u> Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	-1.23060+ 7 -7.29600+ 6 -1.23060+ 7 -7.29600+ 6 7.93700+ 6 -7.08000+ 6 -1.94400+ 6	
Angular dist. of secondary neutrons	Elastic scattering cross section		
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-1.23060+ 7 -7.29600+ 6	
Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	7.93700+ 6	
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)		
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.93700+ 6	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.93700+ 6	

16-S - 32g	Mat. No: 7822 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 891	Reaction Type  Q-value
	File Type		
General information	Descriptive data and Dictionary		
Resonance parameter data	Resonance information		
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section <u> Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	-1.50430+ 7 -8.86500+ 6 -1.50430+ 7 -8.86500+ 6 8.64200+ 6 -9.21164+ 6 -1.26900+ 7 1.52500+ 6	
Angular dist. of secondary neutrons	Elastic scattering cross section		
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-1.50430+ 7 -8.86500+ 6	
Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	8.64200+ 6	
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)		
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.64200+ 6	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.64200+ 6	

17-C1- 0g	Mat. No: 7823 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1947	Reaction Type  Q-value
	File Type		
General information	Descriptive data and Dictionary		
Resonance parameter data	Resonance information		
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section <u> Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	-1.26470+ 7 -6.95900+ 6 -6.37100+ 6 8.57900+ 6 6.15600+ 6 8.38000+ 6	
Angular dist. of secondary neutrons	Elastic scattering cross section		
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-1.26470+ 7 -6.95900+ 6 -6.37100+ 6	
Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	8.57900+ 6	
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)		
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.57900+ 6	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.57900+ 6	

18-Ar - 0g	Mat. No: 7824 Date: _____ Ref.: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 653	Reaction Type	O-Value
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General Information				
Resonance parameter data				
Neutron cross sections				
		Descriptive data and Dictionary		
		Resonance information		
		Total cross section(sum of partials)		
		Elastic scattering cross section		
		Total inelastic cross section(sum of MT=51to91)		
		direct (n,2n) cross section	-9.87000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	6.10000+ 6	
		(n,p) cross section	-6.72000+ 6	
		(n,a) cross section	-2.49000+ 6	
		<2> Lab:average elastic scattering cosine.		
		<2> average log energy decrement for elastic scat.		
		<2> : <ee>/<2e>		
		Elastic scattering cross section		
		direct (n,2n) cross section	-9.87000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	6.10000+ 6	
		Non elastic cross section(total-elastic)		
		(n,p) radiative capture cross section	6.10000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	6.10000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	6.10000+ 6	
		<2> Lab:average elastic scattering cosine.		
		<2> average log energy decrement for elastic scat.		
		<2> : <ee>/<2e>		
		Elastic scattering cross section		
		direct (n,2n) cross section	-1.30750+ 7	
		(n,n') cross section	-7.21700+ 6	
		(n,p) cross section	-6.38000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	7.80000+ 6	
		(n,p) cross section	2.17000+ 5	
		(n,a) cross section	1.36200+ 6	
		<2> Lab:average elastic scattering cosine.		
		<2> average log energy decrement for elastic scat.		
		<2> : <ee>/<2e>		
		Elastic scattering cross section		
		direct (n,2n) cross section	-1.30750+ 7	
		(n,n') cross section	-7.21700+ 6	
		(n,p) cross section	-6.38000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	7.80000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	7.80000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	7.80000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	7.80000+ 6	

19-K - 0g	Mat. No: 7825 Date: _____ Ref.: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1398	Reaction Type	O-Value
<hr/>				
General Information				
Resonance parameter data				
Neutron cross sections				
		Descriptive data and Dictionary		
		Resonance information		
		Total cross section(sum of partials)		
		Elastic scattering cross section		
		Total inelastic cross section(sum of MT=51to91)		
		direct (n,2n) cross section	-1.30750+ 7	
		(n,n') cross section	-7.21700+ 6	
		(n,p) cross section	-6.38000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	7.80000+ 6	
		(n,p) cross section	2.17000+ 5	
		(n,a) cross section	1.36200+ 6	
		<2> Lab:average elastic scattering cosine.		
		<2> average log energy decrement for elastic scat.		
		<2> : <ee>/<2e>		
		Elastic scattering cross section		
		direct (n,2n) cross section	-1.30750+ 7	
		(n,n') cross section	-7.21700+ 6	
		(n,p) cross section	-6.38000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	7.80000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	7.80000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	7.80000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	7.80000+ 6	

20-Ca - 0g	Mat. No: 7826 Date: _____ Ref.: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1671	Reaction Type	O-Value
<hr/>				
General Information				
Resonance parameter data				
Neutron cross sections				
		Descriptive data and Dictionary		
		Resonance information		
		Total cross section(sum of partials)		
		Elastic scattering cross section		
		Total inelastic cross section(sum of MT=51to91)		
		direct (n,2n) cross section	-1.00300+ 7	
		(n,n') cross section	-7.14000+ 6	
		(n,p) cross section	-8.34000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	8.36000+ 6	
		(n,p) cross section	-5.51000+ 5	
		(n,a) cross section	1.68000+ 6	
		<2> Lab:average elastic scattering cosine.		
		<2> average log energy decrement for elastic scat.		
		<2> : <ee>/<2e>		
		Elastic scattering cross section		
		direct (n,2n) cross section	-1.00300+ 7	
		(n,n') cross section	-7.14000+ 6	
		(n,p) cross section	-8.34000+ 6	
		(n,n') to the continuum		
		(n,g) radiative capture cross section	8.36000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	8.36000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	8.36000+ 6	
		Non elastic cross section(total-elastic)		
		(n,g) radiative capture cross section	8.36000+ 6	

82-Pb - 0g	Mat. No: 7862 Date: _____ Ref.: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1036
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MICROCOPY RESOLUTION TEST CHART  
NATIONAL BUREAU OF STANDARDS  
STANDARD REFERENCE MATERIAL 1010a  
(ANSI and ISO TEST CHART No. 2)

22-11-0g	Mat No: 7c28 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1593	Reaction Type	Q-Value
File Type				
General Information				
Resonance parameter data				
Neutron cross sections				
Descriptive data and Dictionary				
Resonance information				
Total cross section(sum of partials)				
Elastic scattering cross section				
(n,2n) inelastic cross section(sum of MT=51to91)				
direct (n,2n) cross section				
9.87000+ 5 Ev (n,n') Level				
(n,n') to the continuum				
(n,g) radiative capture cross section				
(n,p) cross section				
(n,a) cross section				
<uv> Lab: average elastic scattering cosine.				
<ex>: average log energy decrement for elastic scat.				
<gv> : <ee>/2e5				
Angular dist. of secondary neutrons				
Energy dist. of secondary neutrons				
Photon prod. Multiplicities (from neut. reactions)				
Photon prod. cross sections (from neut. reactions)				
Angular dist. of Photons (from neut. reactions)				
Energy dist. of Photons (from neut. reactions)				

23-V - 51g	Mat No: 7629 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 3307	Reaction Type	Q-Value
File Type				
General Information				
Resonance parameter data				
Neutron cross sections				
Descriptive data and Dictionary				
Resonance information				
Total cross section(sum of partials)				
Elastic scattering cross section				
Total inelastic cross section(sum of MT=51to91)				
direct (n,2n) cross section				
(n,n'a) cross section				
3.20100+ 5 Ev (n,n') Level				
9.29000+ 5 Ev (n,n') Level				
1.60900+ 6 Ev (n,n') Level				
1.81300+ 6 Ev (n,n') Level				
2.40900+ 6 Ev (n,n') Level				
2.67500+ 6 Ev (n,n') Level				
3.08000+ 6 Ev (n,n') Level				
(n,n') to the continuum				
(n,g) radiative capture cross section				
(n,d) cross section				
(n,t) cross section				
(n,a) cross section				
<uv> Lab: average elastic scattering cosine.				
<ex>: average log energy decrement for elastic scat.				
<gv> : <ee>/2e5				
Angular dist. of secondary neutrons				
Energy dist. of secondary neutrons				
Photon prod. Multiplicities (from neut. reactions)				
Photon prod. cross sections (from neut. reactions)				
Angular dist. of Photons (from neut. reactions)				
Energy dist. of Photons (from neut. reactions)				

90-Tr-231g	Mat No: 7863 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1886
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IAEA NUCLEAR DATA SECTION, P.O. BOX 100, A-1400 VIENNA

24-Cr- 0g	Mat. No: 7830 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1753	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section 5.62000+ 5 EV (n,n') Level 7.90000+ 5 EV (n,n') Level 9.95000+ 5 EV (n,n') Level 11.80000+ 5 EV (n,n') Level 1.83000+ 6 EV (n,n') Level 2.35000+ 6 EV (n,n') Level 2.64800+ 6 EV (n,n') Level 2.94000+ 6 EV (n,n') Level (n,g) to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-5.62000+ 5 -7.24050+ 6 -1.05050+ 7 -5.62000+ 5 -7.90000+ 5 -9.95000+ 5 -11.80000+ 5 -1.83000+ 6 -2.35000+ 6 -2.64800+ 6 -2.94000+ 6 9.25000+ 6 -3.16000+ 6 -1.21000+ 6
				<u> Lab:average elastic scattering cosine. <x>:average log energy decrement for elastic scat. <q> : <ee>/<2e>	
			Angular dist. of secondary neutrons	Elastic scattering cross section 5.62000+ 5 EV (n,n') Level 7.90000+ 5 EV (n,n') Level 9.95000+ 5 EV (n,n') Level 11.80000+ 5 EV (n,n') Level 1.83000+ 6 EV (n,n') Level 2.35000+ 6 EV (n,n') Level 2.64800+ 6 EV (n,n') Level 2.94000+ 6 EV (n,n') Level	-5.62000+ 5 -7.90000+ 5 -9.95000+ 5 -11.80000+ 5 -1.83000+ 6 -2.35000+ 6 -2.64800+ 6 -2.94000+ 6
			Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-7.94050+ 6 -1.05050+ 7
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	9.25000+ 6
			Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)	
			Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	9.25000+ 6
			Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	9.25000+ 6
25-Mn- 55g	Mat.No: 7831 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1173	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,D) cross section (n,a) cross section	-1.02300+ 7 -8.06800+ 6 -7.27000+ 6 -1.81000+ 6 -6.30000+ 5
				<u> Lab:average elastic scattering cosine. <x>:average log energy decrement for elastic scat. <q> : <ee>/<2e>	
			Angular dist. of secondary neutrons	Elastic scattering cross section	
			Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-1.02300+ 7 -8.06800+ 6
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	7.27000+ 6
			Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)	
			Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.27000+ 6
			Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.27000+ 6
26-Fer- 0g	Mat.No: 7832 Date: Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 2496	File Type	Reaction Type	Q-Value
			General Information	Descriptive data and Dictionary	
			Resonance parameter data	Resonance information	
			Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section 8.45000+ 5 EV (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,D) cross section (n,a) cross section	-8.45000+ 5 -1.12000+ 7 -8.45000+ 5 7.84500+ 6 -1.91000+ 6 3.20000+ 5
				<u> Lab:average elastic scattering cosine. <x>:average log energy decrement for elastic scat. <q> : <ee>/<2e>	
			Angular dist. of secondary neutrons	Elastic scattering cross section 8.45000+ 5 EV (n,n') Level	-8.45000+ 5
			Energy dist. of secondary neutrons	direct (n,2n) cross section (n,n') to the continuum	-1.12000+ 7
			Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	7.84500+ 6
			Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)	
			Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.84500+ 6
			Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.84500+ 6

27-Co- 50g Mat. No: 7835 Date: Ref. UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 4713																																																																																																																																					
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<b>Resonance parameter data</b>																																																																																																																																						
<b>Neutron cross sections</b>																																																																																																																																						
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28-NI- 0g Mat. No: 7836 Date: Ref. UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 3922																																																																																																																																																																			
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28-Ni- 58g	Mat. No: 7837 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 446C UCRL-5400, VOL. 15	
File Type		Reaction Type	Q-value
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,d) cross section (n,a) cross section	-1.21800+ 7 -8.18000+ 6 8.52000+ 6 4.00000+ 5 -5.95000+ 6 2.89000+ 6
Angular dist. of secondary neutrons		<Ü> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons		Elastic scattering cross section	
Photon prod. Multiplicities (from neut.reactions)		direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-1.21800+ 7 -8.18000+ 6
Photon prod. cross sections (from neut.reactions)		(n,g) radiative capture cross section	8.52000+ 6
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.52000+ 6
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.52000+ 6

29-Cu- 0g	Mat. No: 7838 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1367 UCRL-5400, VOL. 15	
File Type		Reaction Type	Q-value
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,d) cross section	-9.91000+ 6 -5.77000+ 6 -6.12000+ 6 7.75000+ 6 7.25000+ 6 1.72000+ 6
Angular dist. of secondary neutrons		<Ü> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons		Elastic scattering cross section	
Photon prod. Multiplicities (from neut.reactions)		direct (n,2n) cross section (n,n') cross section (n,n') to the continuum	-9.91000+ 6 -5.77000+ 6 -6.12000+ 6
Photon prod. cross sections (from neut.reactions)		(n,g) radiative capture cross section	7.75000+ 6
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.75000+ 6
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.75000+ 6

31-Ga- 0g	Mat. No: 7840 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1017 UCRL-5400, VOL. 15	
File Type		Reaction Type	Q-value
General Information		Descriptive data and Dictionary	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,d) cross section	-9.30000+ 6 6.97000+ 6 -1.30000+ 5 2.56000+ 6
Angular dist. of secondary neutrons		<Ü> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons		Elastic scattering cross section	
Photon prod. Multiplicities (from neut.reactions)		direct (n,2n) cross section (n,n') to the continuum	-9.30000+ 6
Photon prod. cross sections (from neut.reactions)		(n,g) radiative capture cross section	6.97000+ 6
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.97000+ 6
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.97000+ 6

File Type	Reaction Type	Q-value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section  <Ü> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <e>/<2e>	-7.98200+ 6 -1.87710+ 7
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-7.98200+ 6 -1.87710+ 7
Photon prod. Multiplicities (from neut.reactions)	Non elastic cross section(total-elastic)	1.02460+ 7
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.02460+ 7
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.02460+ 7
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.02460+ 7
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section  <Ü> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <e>/<2e>	-7.98200+ 6 -1.87710+ 7
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-7.98200+ 6 -1.87710+ 7
Photon prod. Multiplicities (from neut.reactions)	Non elastic cross section(total-elastic)	1.02460+ 7
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.02460+ 7
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.02460+ 7
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	1.02460+ 7

File Type	Reaction Type	Q-value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section  <Ü> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <e>/<2e>	-1.02460+ 7 -1.82280+ 7
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-1.02460+ 7 -1.82280+ 7
Photon prod. Multiplicities (from neut.reactions)	Non elastic cross section(total-elastic)	7.32800+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.32800+ 6
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.32800+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	7.32800+ 6

Angular dist. of secondary neutrons	(n,t) cross section <cos> Lab:average elastic scattering cosine, <ex>:average log energy decrement for elastic scat. <gx> : <ex>/<ex>	4.79000+ 6
Energy dist. of secondary neutrons	Elastic scattering cross section (n,n') cross section 2.18000+ 6 Ev (n,n') Level	-1.46000+ 6 -2.16000+ 6
Photon prod. Multiplicities (from neut. reactions)	direct (n,2n) cross section	-3.69000+ 6
Angular dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	7.26000+ 6
Energy dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	7.26000+ 6
	(n,g) radiative capture cross section	7.26000+ 6

File Type	Reaction Type	O-Value
<b>General Information</b>		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		
(n,n') to the continuum		
(n,g) radiative capture cross section		
(n,p) cross section		
(n,a) cross section		
<cos> Lab:average elastic scattering cosine,		
<ex>:average log energy decrement for elastic scat.		
<gx> : <ex>/<ex>		
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
<b>Photon prod. Multiplicities (from neut. reactions)</b>		
<b>Photon prod. cross sections (from neut. reactions)</b>		
<b>Angular dist. of Photons (from neut. reactions)</b>		
<b>Energy dist. of Photons (from neut. reactions)</b>		

File Type	Reaction Type	O-Value
<b>General Information</b>		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		
9.09200+ 5 Ev (n,n') Level		
1.50700+ 6 Ev (n,n') Level		
1.74450+ 6 Ev (n,n') Level		
2.22400+ 6 Ev (n,n') Level		
2.56700+ 6 Ev (n,n') Level		
2.88900+ 6 Ev (n,n') Level		
3.10400+ 6 Ev (n,n') Level		
(n,n') to the continuum		
(n,g) radiative capture cross section		
(n,p) cross section		
(n,a) cross section		
<cos> Lab:average elastic scattering cosine,		
<ex>:average log energy decrement for elastic scat.		
<gx> : <ex>/<ex>		
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
<b>Photon prod. Multiplicities (from neut. reactions)</b>		
<b>Photon prod. cross sections (from neut. reactions)</b>		
<b>Angular dist. of Photons (from neut. reactions)</b>		
<b>Energy dist. of Photons (from neut. reactions)</b>		

40-2-- 02	Mat. No: 764 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWARTH UCRL-5400, VOL. 15 Card Images: 2271	File Type	Reaction Type	O-value
<b>General Information</b>					
<b>Resonance parameter data</b>					
<b>Neutron cross sections</b>					
<pre> Initial cross section(sum of partials) Inelastic cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n) cross section   9.19800+ 5 Lv (n,n') Level   7.23000+ 6 Lv (n,n') Level   1.49100+ 7 Lv (n,n') Level   9.19800+ 5 Ev (n,n') Level   7.23000+ 6 Ev (n,n') Level   1.38000+ 6 Ev (n,n') Level   1.48000+ 6 Ev (n,n') Level   1.66000+ 6 Ev (n,n') Level   1.66000+ 6 Ev (n,n') Level   1.85940+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section &lt;u&gt; Lab: average elastic scattering cosine, &lt;e&gt;: average log energy decrement for elastic scat. &lt;g&gt; : &lt;ee&gt;/&lt;2e&gt; </pre>					
<b>Angular dist. of secondary neutrons</b>					
<pre> Elastic scattering cross section   9.19800+ 5 Ev (n,n') Level   7.23000+ 6 Ev (n,n') Level   1.49100+ 7 Ev (n,n') Level   1.38000+ 6 Ev (n,n') Level   1.48000+ 6 Ev (n,n') Level   1.66000+ 6 Ev (n,n') Level   1.75000+ 6 Ev (n,n') Level   1.85940+ 6 Ev (n,n') Level </pre>					
<b>Energy dist. of secondary neutrons</b>					
<pre> direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum </pre>					
<b>Photon prod. Multiplicities (from neut.reactions)</b>					
<b>Photon prod. cross sections (from neut.reactions)</b>					
<b>Angular dist. of Photons (from neut.reactions)</b>					
<b>Energy dist. of Photons (from neut.reactions)</b>					

41-ND- 93g	Mat. No: 7843 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWARTH UCRL-5400, VOL. 15 Card Images: 6024	File Type	Reaction type	O-value
<b>General Information</b>					
<b>Resonance parameter data</b>					
<b>Neutron cross sections</b>					
<pre> Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum </pre>					
<pre> (n,g) radiative capture cross section Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) (n,g) radiative capture cross section (n,g) radiative capture cross section </pre>					
<b>Angular dist. of secondary neutrons</b>					
<pre> Elastic scattering cross section   2.90000+ 4 Ev (n,n') Level   7.40000+ 5 Ev (n,n') Level   8.10000+ 5 Ev (n,n') Level   9.59000+ 5 Ev (n,n') Level   1.07000+ 6 Ev (n,n') Level   1.31500+ 6 Ev (n,n') Level   1.48840+ 6 Ev (n,n') Level   1.64004+ 6 Ev (n,n') Level   1.94700+ 6 Ev (n,n') Level   2.15900+ 6 Ev (n,n') Level   2.33500+ 6 Ev (n,n') Level   2.51400+ 6 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,a) cross section </pre>					
<pre> &lt;u&gt; Lab: average elastic scattering cosine, &lt;e&gt;: average log energy decrement for elastic scat. &lt;g&gt; : &lt;ee&gt;/&lt;2e&gt; </pre>					
<b>Energy dist. of secondary neutrons</b>					
<pre> Elastic scattering cross section   2.90000+ 4 Ev (n,n') Level   7.40000+ 5 Ev (n,n') Level   8.10000+ 5 Ev (n,n') Level   9.59000+ 5 Ev (n,n') Level   1.07000+ 6 Ev (n,n') Level   1.31500+ 6 Ev (n,n') Level   1.48840+ 6 Ev (n,n') Level   1.67400+ 6 Ev (n,n') Level   1.94700+ 6 Ev (n,n') Level   2.15900+ 6 Ev (n,n') Level   2.33500+ 6 Ev (n,n') Level   2.51900+ 6 Ev (n,n') Level </pre>					
<b>Photon prod. Multiplicities (from neut.reactions)</b>					
<b>Photon prod. cross sections (from neut.reactions)</b>					
<b>Angular dist. of Photons (from neut.reactions)</b>					
<b>Energy dist. of Photons (from neut.reactions)</b>					

42-Mo- 0g	Mat. No: 7844 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1233
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section <u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		Non elastic cross section(total-elastic)
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic)
		(n,g) radiative capture cross section

47-Ag-107g	Mat.No: 7845 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 2086
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section 9.30000+ 4 Ev (n,n') Level 1.19860+ 5 Ev (n,n') Level 3.25000+ 5 Ev (n,n') Level 4.23000+ 5 Ev (n,n') Level 7.62800+ 5 Ev (n,n') Level 9.22000+ 5 Ev (n,n') Level (n,n') to the continuum (n,g) Radiative capture cross section (n,d) cross section (n,i) cross section (n,a) cross section <u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section 9.30000+ 4 Ev (n,n') Level 1.19860+ 5 Ev (n,n') Level 3.25000+ 5 Ev (n,n') Level 4.23000+ 5 Ev (n,n') Level 7.62800+ 5 Ev (n,n') Level 9.22000+ 5 Ev (n,n') Level
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		Non elastic cross section(total-elastic)
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic)
		(n,g) radiative capture cross section

Energy dist. of secondary neutrons	8.88000+ 6 Ev {n,n'} Level 1.11000+ 7 Ev {n,n'} Level	-8.88000+ 6 -1.11000+ 7
Photon prod. cross sections(from neut.reactions)	direct (n,2n) cross section (n,n') cross section (n,p) cross section (n,n') to the continuum	-1.56000+ 7 -7.11000+ 7 -1.21000+ 7
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic)	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic)	

File Type	Reaction Type	Q-value
<b>General Information</b>		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
	Descriptive data and Dictionary	
	Resonance information	
	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section 8.79990+ 6 Ev {n,n'} Level 1.10990+ 7 Ev {n,n'} Level 3.10990+ 7 Ev {n,n'} Level 4.14990+ 7 Ev {n,n'} Level 7.01990+ 8 Ev {n,n'} Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section	-8.79990+ 4 -8.18180+ 6 -1.64600+ 7 -8.79990+ 4 -1.30990+ 5 -3.10990+ 5 -4.14990+ 5 -7.01990+ 5 6.80570+ 6 -3.33500+ 5 3.28800+ 6
	<U> Lab:average elastic scattering cosine. <ex>: average log energy decrement for elastic scat. <g> : <ee>/<2e>	
<b>Angular dist. of secondary neutrons</b>		
	Elastic scattering cross section 8.79990+ 6 Ev {n,n'} Level 1.10990+ 7 Ev {n,n'} Level 3.10990+ 7 Ev {n,n'} Level 4.14990+ 7 Ev {n,n'} Level 7.01990+ 8 Ev {n,n'} Level	-8.79990+ 4 -1.32200+ 5 -3.10990+ 5 -4.14990+ 5 -7.01990+ 5
<b>Energy dist. of secondary neutrons</b>		
	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-9.18180+ 6 -1.64600+ 7
<b>Photon prod. Multiplicities (from neut.reactions)</b>		
<b>Photon prod. cross sections (from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
<b>Energy dist. of Photons (from neut.reactions)</b>		
	Non elastic cross section(total-elastic)	
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	6.80570+ 6
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	6.80570+ 6
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	6.80570+ 6

File Type	Reaction Type	Q-value
<b>General Information</b>		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
	Descriptive data and Dictionary	
	Resonance information	
	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-8.62000+ 6 -1.61500+ 7 9.04000+ 6
	<U> Lab:average elastic scattering cosine. <ex>: average log energy decrement for elastic scat. <g> : <ee>/<2e>	
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-8.62000+ 6 -1.61500+ 7
<b>Photon prod. Multiplicities (from neut.reactions)</b>		
<b>Photon prod. cross sections (from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
<b>Energy dist. of Photons (from neut.reactions)</b>		
	Non elastic cross section(total-elastic)	
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	9.04000+ 6
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	9.04000+ 6
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	9.04000+ 6

File Type	Reaction Type	Q-value
<b>General Information</b>		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
	Descriptive data and Dictionary	
	Resonance information	
	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-9.30000+ 6 -1.50000+ 7 9.35000+ 6
	<U> Lab:average elastic scattering cosine. <ex>: average log energy decrement for elastic scat. <g> : <ee>/<2e>	
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-9.30000+ 6 -1.50000+ 7
<b>Photon prod. Multiplicities (from neut.reactions)</b>		
<b>Photon prod. cross sections (from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
<b>Energy dist. of Photons (from neut.reactions)</b>		
	Non elastic cross section(total-elastic)	
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	9.35000+ 6
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	9.35000+ 6
	Non elastic cross section(total-elastic)	
	(n,g) radiative capture cross section	9.35000+ 6

56-Ba-138g	Mat. No. 7851 Data Ref.: UCR-5400, VOL. 15	Lab. LIVERMORE Author: R.J. HOWERTON Card Images: 1025
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section <u> : Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		4.71000+ 6
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		4.71000+ 6
		Non elastic cross section(total-elastic)
		(n,g) radiative capture cross section
		4.71000+ 6

63-Eu- 0g	Mat. No. 7852 Date: Ref.: UCR-5400, VOL. 15	Lab. LIVERMORE Author: R.J. HOWERTON Card Images: 1074
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section <u> : Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		6.26000+ 6
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		6.26000+ 6
		Non elastic cross section(total-elastic)
		(n,g) radiative capture cross section
		6.26000+ 6

64-Gd- 0g	Mat. No. 7853 Date: Ref.: UCR-5400, VOL. 15	Lab. LIVERMORE Author: R.J. HOWERTON Card Images: 1223
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section <u> : Lab:average elastic scattering cosine, <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section
Photon prod. Multiplicities (from neut.reactions)		8.04000+ 6
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		8.04000+ 6
		Non elastic cross section(total-elastic)
		(n,g) radiative capture cross section
		8.04000+ 6

67-Ho-165p	Mat. No.: 76 Date: Ref.: UCRL-5400, VOL. 15	Lab.: LIVERMORE Author: R.J. HOWERTON Card Images: 5966
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section
		-7.99000+ 6 -1.46700+ 7
		<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum
		-7.99000+ 6 -1.46700+ 7
Photon prod. Multiplicities (from neut.reactions)		(n,g) radiative capture cross section
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic)
Energy dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
		6.24000+ 6
		6.24000+ 6
		6.24000+ 6
		6.24000+ 6
		6.24000+ 6

72-Hf- 0g	Mat. No. 8305 Date: Ref.: UCRL-5400, VOL. 15	Lab.: LIVERMORE Author: R.J. HOWERTON Card Images: 8853
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		Resonance information
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to-1) direct (n,2n) cross section (n,3n) cross section 9.30000+ 4 Ev {n,n'} Level 3.07000+ 5 Ev {n,n'} Level 6.30000+ 5 Ev {n,n'} Level 1.05000+ 6 Ev {n,n'} Level (n,n') to the continuum (n,g) radiative capture cross section
		-9.30000+ 4 -7.10000+ 6 -1.38900+ 7 -9.30000+ 4 -3.07000+ 5 -6.30000+ 5 -1.05000+ 6 7.18000+ 6
		<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>
Angular dist. of secondary neutrons		Elastic scattering cross section 9.30000+ 4 Ev {n,n'} Level 3.07000+ 5 Ev {n,n'} Level 6.30000+ 5 Ev {n,n'} Level 1.05000+ 6 Ev {n,n'} Level
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum
		-7.10000+ 6 -1.38900+ 7
Photon prod. Multiplicities (from neut.reactions)		(n,g) radiative capture cross section
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic)
Energy dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section
		7.18000+ 6
		7.18000+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum 1.36100+ 5 Ev (n,n') Level 1.58600+ 5 Ev (n,n') Level 3.01500+ 5 Ev (n,n') Level 3.37500+ 5 Ev (n,n') Level 4.82200+ 5 Ev (n,n') Level 4.95000+ 5 Ev (n,n') Level 6.00000+ 5 Ev (n,n') Level 7.20000+ 5 Ev (n,n') Level 9.25000+ 5 Ev (n,n') Level (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section	-1.36100+ 5 -7.63000+ 6 -1.42200+ 7 -1.30900+ 7 -1.58600+ 5 -3.01500+ 5 -3.37500+ 5 -4.82200+ 5 -4.95000+ 5 -6.00000+ 5 -7.20000+ 5 -9.25000+ 5 6.07000+ 6 -2.39000+ 5
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section 1.36100+ 5 Ev (n,n') Level 1.58600+ 5 Ev (n,n') Level 3.01500+ 5 Ev (n,n') Level 3.37500+ 5 Ev (n,n') Level 4.82200+ 5 Ev (n,n') Level 4.95000+ 5 Ev (n,n') Level 6.20000+ 5 Ev (n,n') Level 7.20000+ 5 Ev (n,n') Level 9.25000+ 5 Ev (n,n') Level direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-1.36100+ 5 -1.58600+ 5 -3.01500+ 5 -3.37500+ 5 -4.82200+ 5 -4.95000+ 5 -6.20000+ 5 -7.20000+ 5 -9.25000+ 5 -7.63000+ 6 -1.42200+ 7
Photon prod.Multiplicities(from neut.reactions)	(n,g) radiative capture cross section	6.07000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.07000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.07000+ 6
File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-7.45000+ 6 -1.30900+ 7 5.83000+ 6
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-7.46000+ 6 -1.30900+ 7
Photon prod.Multiplicities(from neut.reactions)	(n,g) radiative capture cross section	5.83000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	5.83000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	5.83000+ 6
File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-7.81000+ 6 -1.44700+ 7 6.17800+ 6
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-7.81000+ 6 -1.44700+ 7
Photon prod.Multiplicities(from neut.reactions)	(r,-) radiative capture cross section	6.17800+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.17800+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	6.17800+ 6

Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section	-1.26470+ 7
	(n,n' a) cross section	-6.99900+ 6
	(n,n' b) cross section	-6.37100+ 6
	(n,n' ) to the continuum	
Photon prod. Multiplicities (from neut. reactions)	(n,g) radiative capture cross section	8.57900+ 6
Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total elastic)	
Angular dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic)	8.57900+ 6
Energy dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	8.57900+ 6
	(n,g) radiative capture cross section	

75-RB-187g Mat. No. 7859 Lab: LIVERMORE  
Date: Author: R.J.HOWERTON  
Ref: UCRL-5400, VOL. 15 Card Images: 2567

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section (sum of partials) Elastic scattering cross section Total inelastic cross section (sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section <2> Lab-average elastic scattering cosine. <2> average log energy decrement for elastic scat. <2> : <ee>/<2e>	-7.32000+ 6 -1.35600+ 7 5.87300+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-7.32000+ 6 -1.35600+ 7
Photon prod. Multiplicities (from neut. reactions)	Non elastic cross section (total-elastic)	5.87300+ 6
Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total-elastic)	5.87300+ 6
Angular dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	5.87300+ 6
Energy dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) (n,g) radiative capture cross section	5.87300+ 6

78-PI- 0g Mat. No. 7860 Lab: LIVERMORE  
Date: Author: R.J.HOWERTON  
Ref: UCRL-5400, VOL. 15 Card Images: 1931

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section (sum of partials) Elastic scattering cross section Total inelastic cross section (sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section <2> Lab-average elastic scattering cosine. <2> average log energy decrement for elastic scat. <2> : <ee>/<2e>	-6.07000+ 6 -1.39000+ 7 7.72000+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-6.07000+ 6 -1.39000+ 7
Photon prod. Multiplicities (from neut. reactions)	Non elastic cross section (total-elastic)	7.72000+ 6
Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total-elastic)	7.72000+ 6
Angular dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	7.72000+ 6
Energy dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) (n,g) radiative capture cross section	7.72000+ 6

79-Au-197g Mat. No. 7861 Lab: LIVERMORE  
Date: Author: R.J.HOWERTON  
Ref: UCRL-5400, VOL. 15 Card Images: 3550

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section (sum of partials) Elastic scattering cross section Total inelastic cross section (sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section (n,p) cross section (n,a) cross section <2> Lab-average elastic scattering cosine. <2> average log energy decrement for elastic scat. <2> : <ee>/<2e>	-8.06000+ 6 -1.47500+ 7 6.51000+ 6 4.00000+ 6 7.03000+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-8.06000+ 6 -1.47500+ 7
Photon prod. Multiplicities (from neut. reactions)	Non elastic cross section (total-elastic)	6.51000+ 6
Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total-elastic)	6.51000+ 6
Angular dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	6.51000+ 6
Energy dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) (n,g) radiative capture cross section	6.51000+ 6

Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic)	
Energy dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	8.36000+ 6
	(n,g) radiative capture cross section	8.36000+ 6

File Type	Reaction Type	Q-value
General Information		
Resonance parameter data		
Neutron cross sections		
Photon prod.Multiplicities(from neut.reactions)		
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Energy dist. of Photons (from neut.reactions)		
Angular dist. of secondary neutrons		
Energy dist. of secondary neutrons		
Photon prod.Multiplicities(from neut.reactions)		
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Energy dist. of Photons (from neut.reactions)		

File Type	Reaction Type	Q-value
General Information		
Resonance parameter data		
Neutron cross sections		
Photon prod.Multiplicities(from neut.reactions)		
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Energy dist. of Photons (from neut.reactions)		
Angular dist. of secondary neutrons		
Energy dist. of secondary neutrons		
Photon prod.Multiplicities(from neut.reactions)		
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Energy dist. of Photons (from neut.reactions)		

96 Cm-243g	Mat. No: 7886	Lab: LIVERMORE
Date:	Author: R.J.HOWERTON	Card Images: 2966

90-In-231g	Mat No. 7863 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1886
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		<u> : average total(promt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission
Neutron cross sections		Resonance information
Angular dist. of secondary neutrons	Total cross section(sum of partials) Elastic scattering cross section Total fission cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-5.12000+ 6 -1.19100+ 7 1.80000+ 8 -1.71500+ 7 6.43000+ 6
Energy dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <ex>: <ee>/<2e>	
Photon prod.Multiplicities(from neut.reactions)	Elastic scattering cross section	
Photon prod cross sections(from neut.reactions)	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-5.12000+ 6 -1.19100+ 7 1.80000+ 8 -1.71500+ 7
Angular dist. of Photons (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.43000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.43000+ 6
	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.43000+ 6

90-In-232g	Mat No: 7864 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 6472
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		<u> : average total(promt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission
Neutron cross sections	Resonance information	
Angular dist. of secondary neutrons	Total cross section(sum of partials) Elastic scattering cross section Total fission cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-6.34000+ 6 -1.15600+ 7 1.70000+ 8 -1.83500+ 7 4.79000+ 6
Energy dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <ex>: <ee>/<2e>	
Photon prod.Multiplicities(from neut.reactions)	Elastic scattering cross section	
Photon prod.cross sections(from neut.reactions)	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-6.34000+ 6 -1.15600+ 7 1.70000+ 8 -1.83500+ 7
Angular dist. of Photons (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.70000+ 8 4.79000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.70000+ 8 4.79000+ 6
	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.70000+ 8 4.79000+ 6

Photon prod. Multiplicities (from neut.reactions)	(n,g) radiative capture cross section	7.84500+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section (total-elastic)	
Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	7.84500+ 6
Energy dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	7.84500+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <cv> :average total(promote/delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-4.78000+ 6 -1.12200+ 7 1.80000+ 8 -1.63400+ 7 6.18000+ 6
Angular dist. of secondary neutrons	<Ü> Lab: average elastic scattering cosine. <ez>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-4.78000+ 6 -1.12200+ 7 1.80000+ 8 -1.63400+ 7
Photon prod. Multiplicities (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.18000+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section (total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section (total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.18000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section (total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.18000+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <cv> :average total(promote/delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-5.90000+ 6 -1.31800+ 7 1.80000+ 8 6.84000+ 6
Angular dist. of secondary neutrons	<Ü> Lab: average elastic scattering cosine. <ez>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-5.90000+ 6 -1.31800+ 7 1.80000+ 8
Photon prod. Multiplicities (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.84000+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section (total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section (total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.84000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section (total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.84000+ 6

Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	8.00000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic)	8.60000+ 6

File Type	Reaction Type	Q-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<v> : average total(promt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-6.84000+ 6 -1.25900+ 7 1.80000+ 8 5.31000+ 6	
<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<ee>		
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	-6.84000+ 6 -1.25900+ 7 1.80000+ 8	
<b>Photon prod.Multiplicities(from neut.reactions)</b>		
<b>Photon prod.cross sections(from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.31000+ 6	
<b>Energy dist. of Photons (from neut.reactions)</b>		
Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.31000+ 6	

File Type	Reaction Type	Q-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<v> : average total(promt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-5.23000+ 6 -1.20000+ 7 1.80000+ 8 -1.78900+ 7 6.55000+ 6	
<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<ee>		
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	-5.23000+ 6 -1.20000+ 7 1.80000+ 8 -1.78900+ 7	
<b>Photon prod.Multiplicities(from neut.reactions)</b>		
<b>Photon prod.cross sections(from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.55000+ 6	
<b>Energy dist. of Photons (from neut.reactions)</b>		
Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.55000+ 6	

Photon prod.Multiplicities(from neut.reactions)	(n,g) radiative capture cross section	6.97000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	6.97000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic)	6.97000+ 6
	(n,g) radiative capture cross section	6.97000+ 6

File Type	Reaction Type	Q-value
<b>General Information</b>		
Descriptive data and Dictionary		
<v> :average total(promote+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-6.55000+ 6 -1.18500+ 7 1.80000+ 8 5.12000+ 6	
<>> Lab:average elastic scattering cosine. <>>average log energy decrement for elastic scat. <>> <ee>/<2e>		
<b>Angular dist. of secondary neutrons</b>		
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-6.55000+ 6 -1.18500+ 7 1.80000+ 8
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.12000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.12000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.12000+ 6

File Type	Reaction Type	Q-value
<b>General Information</b>		
Descriptive data and Dictionary		
<v> :average total(promote+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-5.12000+ 6 -1.16700+ 7 1.80000+ 8 6.14000+ 6	
<>> Lab:average elastic scattering cosine. <>>average log energy decrement for elastic scat. <>> <ee>/<2e>		
<b>Angular dist. of secondary neutrons</b>		
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-5.12000+ 6 -1.16700+ 7 1.80000+ 8
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.14000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.14000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.14000+ 6

92-U -238g	Mat.No: 7871 Date: REF: UCR-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 3455
	File Type	Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		<U> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission
Neutron cross sections		Resonance information
		Total cross section(sum of partials) Elastic scattering cross section Total Inelastic cross section(sum of MT=51to81) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section
		4.50000+ 4 Ev (n,n') Level 1.48000+ 5 Ev (n,n') Level 2.08000+ 5 Ev (n,n') Level 6.80000+ 5 Ev (n,n') Level 7.12000+ 5 Ev (n,n') Level 8.27000+ 5 Ev (n,n') Level 9.30000+ 5 Ev (n,n') Level 9.67000+ 5 Ev (n,n') Level 1.00000+ 6 Ev (n,n') Level 1.04100+ 6 Ev (n,n') Level 1.06000+ 6 Ev (n,n') Level 1.12000+ 6 Ev (n,n') Level 1.16000+ 6 Ev (n,n') Level 1.22000+ 6 Ev (n,n') Level 1.27000+ 6 Ev (n,n') Level 1.30000+ 6 Ev (n,n') Level 1.36000+ 6 Ev (n,n') Level 1.40000+ 6 Ev (n,n') Level 1.43700+ 6 Ev (n,n') Level 1.47000+ 6 Ev (n,n') Level 1.62500+ 6 Ev (n,n') Level 1.87500+ 6 Ev (n,n') Level (n,n') to the continuum
		<U> : average elastic scattering cosine. <E>:average log energy decrement for elastic scat. <Q> : kee>/c2es
Angular dist. of secondary neutrons		Elastic scattering cross section
		4.50000+ 4 Ev (n,n') Level 1.48000+ 5 Ev (n,n') Level 3.08000+ 5 Ev (n,n') Level 6.80000+ 5 Ev (n,n') Level 7.32000+ 5 Ev (n,n') Level 8.27000+ 5 Ev (n,n') Level 9.67000+ 5 Ev (n,n') Level 1.00000+ 6 Ev (n,n') Level
		1.04100+ 6 Ev (n,n') Level 1.06000+ 6 Ev (n,n') Level 1.12000+ 6 Ev (n,n') Level 1.16000+ 6 Ev (n,n') Level 1.22000+ 6 Ev (n,n') Level 1.27000+ 6 Ev (n,n') Level 1.30000+ 6 Ev (n,n') Level 1.36000+ 6 Ev (n,n') Level 1.40000+ 6 Ev (n,n') Level 1.43700+ 6 Ev (n,n') Level 1.47000+ 6 Ev (n,n') Level 1.62500+ 6 Ev (n,n') Level 1.87500+ 6 Ev (n,n') Level
Energy dist. of secondary neutrons		direct (n,2n) cross section (n,3n) cross section total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum
Photon prod. Multiplicities (from neut.reactions)		Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)
Angular dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <2> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-4.80000+ 6 -1.09500+ 7 -1.80000+ 8 -1.60700+ 7 5.93000+ 6
Angular dist. of secondary neutrons	<2> Lab: average elastic scattering cosine. <2> : average log energy decrement for elastic scat. <2> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-4.80000+ 6 -1.08500+ 7 -1.80000+ 8 -1.60700+ 7
Photon prod. Multiplicities (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.93000+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.93000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.93000+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <2> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-5.93000+ 6 -1.07400+ 7 -1.80000+ 8 -1.68800+ 7 6.08000+ 6
Angular dist. of secondary neutrons	<2> Lab: average elastic scattering cosine. <2> : average log energy decrement for elastic scat. <2> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-5.93000+ 6 -1.07400+ 7 -1.80000+ 8 -1.68800+ 7
Photon prod. Multiplicities (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.08000+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.08000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.08000+ 6


File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <v> :average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,n') cross section total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-6.98300+ 6 -1.31143+ 7 -1.80000+ 8 5.68500+ 6
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-6.98300+ 6 -1.31143+ 7 -1.80000+ 8
Photon prod. Multiplicities (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.68500+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.68500+ 6
Angular dist. of Photons (from neut.reactions)	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.68500+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <v> :average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-5.68500+ 6 -1.26660+ 7 1.80000+ 8 6.62800+ 6
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-5.68500+ 6 -1.26660+ 7 1.80000+ 8
Photon prod. Multiplicities (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.62800+ 6
Photon prod. cross sections (from neut.reactions)	Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.62800+ 6
Angular dist. of Photons (from neut.reactions)	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.62800+ 6

93-Nd-237g	Mat. No: 7874 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 2531
	File Type	Reaction Type
General Information	Descriptive data and Dictionary  <cv> : average total (prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	Q-Value
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section  <U> Lab:average elastic scattering cosine. <ex>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	-6.53700+ 6 -1.23100+ 7 1.80000+ 8 5.48000+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-6.53700+ 6 -1.23100+ 7 1.80000+ 8
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.48000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.48000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 5.48000+ 6

93-Np-238g	Mat. No: 8309 Date: Ref:	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1509
	File Type	Reaction Type
General Information	Descriptive data and Dictionary  <cv> : average total (prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	Q-Value
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section  <U> Lab:average elastic scattering cosine. <ex>:average log energy decrement for elastic scat. <g> : <ee>/<2e>	-5.48800+ 6 -1.21160+ 7 1.80000+ 8 6.21800+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-5.48800+ 6 -1.21160+ 7 1.80000+ 8
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.21800+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.21800+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	1.80000+ 8 6.21800+ 6

	Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-9.30000+ 6 -1.50000+ 7
	Photon prod. Multiplicities (from neut. reactions)	(n,g) radiative capture cross section	9.35000+ 6
	Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total-elastic)	
	Angular dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) (n,g) radiative capture cross section	9.35000+ 6
	Energy dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) (n,g) radiative capture cross section	9.35000+ 6

94-Pu-238g	Mat No: 7875 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J. HOWERTON Card Images: 2296	Reaction Type	O-Value
File Type				
General Information				
Resonance parameter data				
Neutron cross sections				
Descriptive data and Dictionary				
<v> : average total(promote+delayed)neutrons/fission				
Delayed neutrons from fission				
Prompt neutrons from fission				
Resonance information				
Total cross section (sum of partials)				
Elastic scattering cross section				
Total inelastic cross section (sum of MT=51to91)				
direct (n,2n) cross section				
(n,3n) cross section				
(n,n') to the continuum				
(n,g) radiative capture cross section				
<u> Lab: average elastic scattering cosine.				
<e> : average log energy decrement for elastic scat.				
<g> : <ee><2e>				
Elastic scattering cross section				
direct (n,2n) cross section				
(n,3n) cross section				
(n,n') to the continuum				
Total fission cross section (sum of MT=19to21,38)				
(n,g) radiative capture cross section				
Non elastic cross section (total-elastic)				
Total fission cross section (sum of MT=19to21,38)				
(n,g) radiative capture cross section				
Non elastic cross section (total-elastic)				
Total fission cross section (sum of MT=19to21,38)				
(n,g) radiative capture cross section				

94-Pu-239g	Mat No: 7876 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J. HOWERTON Card Images: 3662	Reaction Type	O-Value
File Type				
General Information				
Resonance parameter data				
Neutron cross sections				
Descriptive data and Dictionary				
<v> : average total(promote+delayed)neutrons/fission				
Delayed neutrons from fission				
Prompt neutrons from fission				
Resonance information				
Total cross section (sum of partials)				
Elastic scattering cross section				
Total inelastic cross section (sum of MT=51to91)				
direct (n,2n) cross section				
(n,3n) cross section				
(n,n') to the continuum				
(n,g) radiative capture cross section				
<u> Lab: average elastic scattering cosine.				
<e> : average log energy decrement for elastic scat.				
<g> : <ee><2e>				
Elastic scattering cross section				
5.70000+ 4 Ev (n,n') Level				
7.60000+ 4 Ev (n,n') Level				
1.64000+ 5 Ev (n,n') Level				
2.86000+ 5 Ev (n,n') Level				
3.31000+ 5 Ev (n,n') Level				
3.90000+ 5 Ev (n,n') Level				
(n,n') to the continuum				
(n,g) radiative capture cross section				
5.70000+ 4 Ev (n,n') Level				
7.60000+ 4 Ev (n,n') Level				
1.64000+ 5 Ev (n,n') Level				
2.86000+ 5 Ev (n,n') Level				
3.31000+ 5 Ev (n,n') Level				
3.90000+ 5 Ev (n,n') Level				
(n,n') to the continuum				
Total fission cross section (sum of MT=19to21,38)				
(n,g) radiative capture cross section				
Non elastic cross section (total-elastic)				
Total fission cross section (sum of MT=19to21,38)				
(n,g) radiative capture cross section				
Non elastic cross section (total-elastic)				
Total fission cross section (sum of MT=19to21,38)				
(n,g) radiative capture cross section				

	Photon prod. Multiplicities (from neut. reactions)	(n,g) radiative capture cross section	8.04000+ 6
	Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total-elastic)	
	Angular dist. of Photons (from neut. reactions)	(n,g) radiative capture cross section	8.04000+ 6
	Energy dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic)	8.04000+ 6
		(n,g) radiative capture cross section	8.04000+ 6

File Type	Reaction Type	Q-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Resonance information		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		
(n,3n) cross section		
Total fission cross section(sum of MT=19to21,38)		
4.50000+ 4 Ev (n,n') Level		-4.50000+ 4
1.42000+ 5 Ev (n,n') Level		-6.53000+ 6
1.42000+ 5 Ev (n,n') Level		-1.18000+ 7
2.96000+ 5 Ev (n,n') Level		-1.68000+ 8
2.96000+ 5 Ev (n,n') Level		-4.50000+ 4
5.99000+ 5 Ev (n,n') Level		-1.42000+ 5
5.99000+ 5 Ev (n,n') Level		-2.96000+ 5
8.90000+ 5 Ev (n,n') Level		-5.99000+ 5
(n,n') to the continuum		-8.90000+ 5
(n,g) radiative capture cross section		5.24000+ 6
<> : Lab:average elastic scattering cosine.		
<> : average log energy decrement for elastic scat.		
<> : <ee>/<2e>		
<b>Angular dist. of secondary neutrons</b>		
Elastic scattering cross section		
4.50000+ 4 Ev (n,n') Level		-4.50000+ 4
1.42000+ 5 Ev (n,n') Level		-1.42000+ 5
2.96000+ 5 Ev (n,n') Level		-2.96000+ 5
5.99000+ 5 Ev (n,n') Level		-5.99000+ 5
8.90000+ 5 Ev (n,n') Level		-8.90000+ 5
direct (n,2n) cross section		-6.53000+ 6
(n,3n) cross section		-1.21000+ 7
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,n') to the continuum		1.86000+ 8
Total fission cross section(sum of MT=19to21,38)		5.24000+ 6
(n,g) radiative capture cross section		5.24000+ 6
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		5.24000+ 6
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		5.24000+ 6
Energy dist. of Photons (from neut. reactions)		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		5.24000+ 6
Energy dist. of Photons (from neut. reactions)		

File Type	Reaction Type	Q-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Resonance information		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		-5.24000+ 6
(n,3n) cross section		-1.17800+ 7
Total fission cross section(sum of MT=19to21,38)		-1.86000+ 8
(n,4n) cross section		-1.72300+ 7
(n,n') to the continuum		6.30000+ 6
(n,g) radiative capture cross section		6.30000+ 6
<> : Lab:average elastic scattering cosine.		
<> : average log energy decrement for elastic scat.		
<> : <ee>/<2e>		
<b>Angular dist. of secondary neutrons</b>		
Elastic scattering cross section		
direct (n,2n) cross section		-5.24000+ 6
(n,3n) cross section		-1.17800+ 7
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,4n) cross section		-1.74300+ 7
(n,n') to the continuum		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Energy dist. of secondary neutrons		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Energy dist. of secondary neutrons		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Energy dist. of Photons (from neut. reactions)		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Energy dist. of Photons (from neut. reactions)		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		1.86000+ 8
(n,g) radiative capture cross section		6.30000+ 6
Energy dist. of Photons (from neut. reactions)		

94-Pu-242g	Mat. No. 7880 Date: UCRL-5400, VOL. 15 Ref: Card Images: 3197	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 3197	
	File Type	Reaction Type	Q-value
General Information		Descriptive data and Dictionary	
		<U> average total(prompt+delayed)neutrons/fission	
		Delayed neutrons from fission	
		Prompt neutrons from fission	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials)	
		Elastic scattering cross section	
		Total inelastic cross section(sum of MT=51to91)	-4.45000+ 4
		direct (n,2n) cross section	-6.30000+ 6
		(n,3n) cross section	-1.15400+ 7
		Total fission cross section(sum of MT=19to21,38)	-2.00000+ 8
		(n,n) cross section	-1.00000+ 8
		4.45000+ 4 Ev (n,n') Level	-4.45000+ 4
		1.49000+ 5 Ev (n,n') Level	-1.49000+ 5
		3.00000+ 5 Ev (n,n') Level	-3.00000+ 5
		6.80000+ 5 Ev (n,n') Level	-6.80000+ 5
		(n,n') to the continuum	
		(n,g) radiative capture cross section	5.04000+ 6
		<U> Lab:average elastic scattering cosine.	
		<U> average log energy decrement for elastic scat.	
		<g> : <ee>/<2e>	
Angular dist. of secondary neutrons		Elastic scattering cross section	
		4.45000+ 4 Ev (n,n') Level	-4.45000+ 4
		1.49000+ 5 Ev (n,n') Level	-1.49000+ 5
		3.00000+ 5 Ev (n,n') Level	-3.00000+ 5
		6.80000+ 5 Ev (n,n') Level	-6.80000+ 5
Energy dist. of secondary neutrons		direct (n,2n) cross section	-6.30000+ 6
		(n,3n) cross section	-1.15400+ 7
		Total fission cross section(sum of MT=19to21,38)	-2.00000+ 8
		(n,n) cross section	-1.00000+ 8
		(n,n') to the continuum	
		Total fission cross section(sum of MT=19to21,38)	2.00000+ 8
		(n,g) radiative capture cross section	5.04000+ 6
Photon prod. Multiplicities (from neut.reactions)		Non elastic cross section(total-elastic)	
Photon prod. cross sections (from neut.reactions)		Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)		Total fission cross section(sum of MT=19to21,38)	2.00000+ 8
Energy dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section	5.04000+ 6
		Non elastic cross section(total-elastic)	
		Total fission cross section(sum of MT=19to21,38)	2.00000+ 8
		(n,g) radiative capture cross section	5.04000+ 6

94-Pu-243g	Mat. No. 7881 Date: UCRL-5400, VOL. 15 Ref: Card Images: 2238	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 2238	
	File Type	Reaction Type	Q-value
General Information		Descriptive data and Dictionary	
		<U> average total(prompt+delayed)neutrons/fission	
		Delayed neutrons from fission	
		Prompt neutrons from fission	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials)	
		Elastic scattering cross section	
		Total inelastic cross section(sum of MT=51to91)	-5.05000+ 6
		direct (n,2n) cross section	-1.13200+ 7
		(n,3n) cross section	-2.00000+ 8
		Total fission cross section(sum of MT=19to21,38)	-1.65810+ 7
		(n,n) cross section	
		(n,n') to the continuum	
		(n,g) radiative capture cross section	6.02000+ 6
		<U> Lab:average elastic scattering cosine.	
		<U> average log energy decrement for elastic scat.	
		<g> : <ee>/<2e>	
Angular dist. of secondary neutrons		Elastic scattering cross section	
		direct (n,2n) cross section	-5.05000+ 6
		(n,3n) cross section	-1.13200+ 7
		Total fission cross section(sum of MT=19to21,38)	-2.00000+ 8
		(n,n) cross section	-1.65810+ 7
Energy dist. of secondary neutrons		Total fission cross section(sum of MT=19to21,38)	2.00000+ 8
		(n,g) radiative capture cross section	6.02000+ 6
		Non elastic cross section(total-elastic)	
Photon prod. Multiplicities (from neut.reactions)		Non elastic cross section(total-elastic)	
Photon prod. cross sections (from neut.reactions)		Total fission cross section(sum of MT=19to21,38)	2.00000+ 8
Angular dist. of Photons (from neut.reactions)		(n,g) radiative capture cross section	6.02000+ 6
Energy dist. of Photons (from neut.reactions)		Non elastic cross section(total-elastic)	
		Total fission cross section(sum of MT=19to21,38)	2.00000+ 8
		(n,g) radiative capture cross section	6.02000+ 6

(n,g) radiative capture cross section	0.17800+ 0
<U> Lab:average elastic scattering cosine.	
<e>:average log energy decrement for elastic scat.	
<g> : <ee>/<es>	
Elastic scattering cross section	
direct (n,2n) cross section	-7.81000+ 6
(n,3n) cross section	-1.44700+ 7
(n,n') to the continuum	
(n,n) radiative capture cross section	6.17800+ 6
Non elastic cross section(total-elastic)	
(n,n) radiative capture cross section	6.17800+ 6
Non elastic cross section(total-elastic)	6.17800+ 6
(n,g) radiative capture cross section	6.17800+ 6

File Type	Reaction Type	O-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<U> : average total(prompt+delayed)neutrons/fission		
Delayed neutrons from fission		
Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Resonance information		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		-5.93000+ 6
(n,3n) cross section		-1.26000+ 7
total fission cross section(sum of MT=19to21,38)		2.00000+ 8
(n,n') to the continuum		
(n,g) radiative capture cross section		5.48000+ 6
(n,g) radiative capture cross section		5.48000+ 6
<U> Lab:average elastic scattering cosine.		
<e>:average log energy decrement for elastic scat.		
<g> : <ee>/<es>		
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
<b>Photon prod.Multiplicities(from neut.reactions)</b>		
<b>Photon prod.cross sections(from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
<b>Energy dist. of Photons (from neut.reactions)</b>		

File Type	Reaction Type	C-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<U> : average total(prompt+delayed)neutrons/fission		
Delayed neutrons from fission		
Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Resonance information		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		-5.53000+ 6
(n,3n) cross section		-1.12500+ 7
total fission cross section(sum of MT=19to21,38)		1.92000+ 8
(n,n') cross section		-1.81300+ 7
(n,n) to the continuum		
(n,g) radiative capture cross section		6.38000+ 6
<U> Lab:average elastic scattering cosine.		
<e>:average log energy decrement for elastic scat.		
<g> : <ee>/<es>		
<b>Angular dist. of secondary neutrons</b>		
<b>Energy dist. of secondary neutrons</b>		
<b>Photon prod.Multiplicities(from neut.reactions)</b>		
<b>Photon prod.cross sections(from neut.reactions)</b>		
<b>Angular dist. of Photons (from neut.reactions)</b>		
<b>Energy dist. of Photons (from neut.reactions)</b>		

Energy dist. of secondary neutrons	direct ( $n,2n$ ) cross section ( $n,3n$ ) cross section ( $n,n$ ) to the continuum	-8.05000+ 6 -1.47500+ 7
Photon prod. Multiplicities (from neut. reactions)	( $n,g$ ) radiative capture cross section	6.51000+ 6
Photon prod. cross sections (from neut. reactions)	Non elastic cross section (total-elastic)	
Angular dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) ( $n,g$ ) radiative capture cross section	6.51000+ 6
Energy dist. of Photons (from neut. reactions)	Non elastic cross section (total-elastic) ( $n,u$ ) radiative capture cross section	6.51000+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary  <cv> :average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance Information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum c= MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section  <cs> Lab:average elastic scattering cosine. <eg>: average log energy decrement for elastic scat. <cp> : <ee>/<2e>	-6.37380+ 6 -1.19010+ 7 -2.00000+ 8 -1.84240+ 7 5.36000+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum  Total fission cross section(sum of MI=19to21,38) (n,g) radiative capture cross section	-6.37380+ 6 -1.19010+ 7 -2.00000+ 8 -1.84240+ 7 2.00000+ 8 5.36000+ 6
Photon prod.Multiplicities(from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MI=19to21,38) (n,g) radiative capture cross section  Non elastic cross section(total-elastic) Total fission cross section(sum of MI=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.36000+ 6
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MI=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.36000+ 6

Mat. No: 7885 Date: 1980 Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 2789	
File Type		Reaction Type
General Information		Descriptive data and Dictionary
Resonance parameter data		<u> : average total(promoted+delayed)neutrons/fission Delayed neutrons from fission Promt neutrons from fission
Neutron cross sections		Resonance information
Angular dist. of secondary neutrons	Total cross section(sum of partials) Elastic Scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section	-6.97000+ -1.30300+ 2.00000+ 5.70000+
Energy dist. of secondary neutrons	<u> : Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <q> : <ee>/<ee>	
Photon prod. Multiplicities (from neut.reactions)	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum	-6.97000+ -1.30300+ 2.00000+ 5.70000+
Photon prod. cross sections (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 5.70000+
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total=elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 5.70000+
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total=elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 5.70000+

96-Cm-243g	Mat. No: 7886 Date: UCR-L5400, VOL. 15 Ref: Card Images: 2928	Lab: LIVERMORE Author: R.J.HOWERTON	
	File Type	Reaction Type	Q-value
General Information		Descriptive data and Dictionary <v> :average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section  <u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <q> : <ee>/<2e>	-5.69000+ 6 -1.26500+ 7 2.00000+ 8 -1.87400+ 7 6.80000+ 6
Angular dist. of secondary neutrons		Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum  Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section  Non elastic cross section(total-elastic)	-5.69000+ 6 -1.26600+ 7 2.00000+ 8 -1.87400+ 7 2.00000+ 8 6.80000+ 6
Energy dist. of secondary neutrons		Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section  Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.80000+ 6
Photon prod. Multiplicities(from neut.reactions)			
Photon prod. cross sections(from neut.reactions)			
Angular dist. of Photons (from neut.reactions)			
Energy dist. of Photons (from neut.reactions)			

96-Cm-244g	Mat.No: 7887 Date: UCR-L5400, VOL. 15 Ref: Card Images: 3227	Lab: LIVERMORE Author: R.J.HOWERTON	
	File Type	Reaction Type	Q-value
General Information		Descriptive data and Dictionary <v> :average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data		Resonance information	
Neutron cross sections		Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section  <u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <q> : <ee>/<2e>	-6.80000+ 6 -1.24900+ 7 2.00000+ 8 -1.94700+ 7 5.52000+ 6
Angular dist. of secondary neutrons		Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum  Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section  Non elastic cross section(total-elastic)	-6.80000+ 6 -1.24900+ 7 2.00000+ 8 -1.94700+ 7 2.00000+ 8 5.52000+ 6
Energy dist. of secondary neutrons		Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section  Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.52000+ 6
Photon prod. Multiplicities(from neut.reactions)			
Photon prod. cross sections(from neut.reactions)			
Angular dist. of Photons (from neut.reactions)			
Energy dist. of Photons (from neut.reactions)			

File Type	Reaction Type	Q-value
General Information	Descriptive data and Dictionary <cv> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-5.52000+ 6 -1.23200+ 7 2.00000+ 8 -1.80200+ 7 6.45000+ 6
Angular dist. of secondary neutrons	<cv> : Lab:average elastic scattering cosine. <ee>:average log energy decrement for elastic scat. <eg> : <ee>/<2es>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-5.52000+ 6 -1.23200+ 7 2.00000+ 8 -1.80200+ 7
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.45000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38)	2.00000+ 8 6.45000+ 6
Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.45000+ 6

File Type	Reaction Type	Q-value
General Information	Descriptive data and Dictionary <cv> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-6.45000+ 6 -1.87600+ 7 2.00000+ 8 -1.87600+ 7 5.16000+ 6
Angular dist. of secondary neutrons	<cv> : Lab:average elastic scattering cosine. <ee>:average log energy decrement for elastic scat. <eg> : <ee>/<2es>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-6.45000+ 6 -1.87600+ 7 2.00000+ 8 -1.87600+ 7
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.16000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38)	2.00000+ 8 5.16000+ 6
Angular dist. of Photons (from neut.reactions)	(n,g) radiative capture cross section	
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.16000+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <u> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section  <u> Lab:average elastic scattering cosine, <ex> : average log energy decrement for elastic scat. <gx> : <ex>/<2e>	-5.15000+ 6 -1.16000+ 7 2.00000+ 8 -1.71000+ 7  6.21000+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-5.15000+ 6 -1.16000+ 7 2.00000+ 8 -1.71000+ 7
Photon prod. Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.21000+ 6
Photon prod. cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.21000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.21000+ 6

File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary <u> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') to the continuum (n,g) radiative capture cross section  <u> Lab:average elastic scattering cosine, <ex> : average log energy decrement for elastic scat. <gx> : <ex>/<2e>	-6.20000+ 6 -1.13600+ 7 2.00000+ 8 -1.78100+ 7  4.71000+ 6
Angular dist. of secondary neutrons	Elastic scattering cross section	
Energy dist. of secondary neutrons	direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-6.20000+ 6 -1.13600+ 7 2.00000+ 8 -1.78100+ 7
Photon prod. Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.71000+ 6
Photon prod. cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.71000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.71000+ 6

File Type	Reaction Type	Q-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<v> : average total(prompt+delayed)neutrons/fission		
Delayed neutrons from fission		
Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Resonance information		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		-6.20000+ 6
(n,3n) cross section		-1.17800+ 7
(n,4n) cross section		2.00000+ 8
(n,n') cross section		-1.17800+ 7
(n,g) radiative capture cross section		4.97000+ 6
<u> : Lab:average elastic scattering cosine.		
<x>:average log energy decrement for elastic scat.		
<g> : <ed>/<2e>		
Angular dist. of secondary neutrons		
Energy dist. of secondary neutrons		
Elastic scattering cross section		
direct (n,2n) cross section		-6.20000+ 6
(n,3n) cross section		-1.17800+ 7
(n,4n) cross section		2.00000+ 8
(n,n') to the continuum		-1.17800+ 7
(n,g) radiative capture cross section		4.97000+ 6
Photon prod.Multiplicities(from neut.reactions)		
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		2.00000+ 8
(n,g) radiative capture cross section		4.97000+ 6
Energy dist. of Photons (from neut.reactions)		
Non elastic cross section(total-elastic)		
Total fission cross sect on(sum of MT=19to21,38)		2.00000+ 8
(n,g) radiative capture cross section		4.97000+ 6

File Type	Reaction Type	Q-Value
<b>General Information</b>		
Descriptive data and Dictionary		
<v> : average total(prompt+delayed)neutrons/fission		
Delayed neutrons from fission		
Prompt neutrons from fission		
<b>Resonance parameter data</b>		
<b>Neutron cross sections</b>		
Resonance information		
Total cross section(sum of partials)		
Elastic scattering cross section		
Total inelastic cross section(sum of MT=51to91)		
direct (n,2n) cross section		-5.56000+ 6
(n,3n) cross section		-1.17800+ 7
(n,4n) cross section		2.00000+ 8
(n,n') to the continuum		-1.85900+ 7
(n,g) radiative capture cross section		6.62000+ 6
<u> : Lab:average elastic scattering cosine.		
<x>:average log energy decrement for elastic scat.		
<g> : <ed>/<2e>		
Angular dist. of secondary neutrons		
Energy dist. of secondary neutrons		
Elastic scattering cross section		
direct (n,2n) cross section		-5.59000+ 6
(n,3n) cross section		-1.17800+ 7
(n,4n) cross section		2.02000+ 8
(n,n') to the continuum		-1.85900+ 7
Total fission cross section(sum of MT=19to21,38)		2.00000+ 8
(n,g) radiative capture cross section		6.62000+ 6
Photon prod.Multiplicities(from neut.reactions)		
Photon prod.cross sections(from neut.reactions)		
Angular dist. of Photons (from neut.reactions)		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		2.00000+ 8
(n,g) radiative capture cross section		6.62000+ 6
Energy dist. of Photons (from neut.reactions)		
Non elastic cross section(total-elastic)		
Total fission cross section(sum of MT=19to21,38)		2.00000+ 8
(n,g) radiative capture cross section		6.62000+ 6

98-Cf-250g

Mat.No: 7894

Lab: LIVERMORE  
Author: R.J.HUERTON  
Ref: UCRL-5400, VOL. 15  
Card Images: 1992

File Type	Reaction Type	Q-value
General Information	Descriptive data and Dictionary <v> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,n') cross section (n,n') to the continuum (n,g) radiative capture cross section	-6.61000+ 6 -1.22100+ 7 2.00000+ 8 -1.85900+ 7 5.11000+ 6
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-6.61000+ 6 -1.22100+ 7 2.00000+ 8 -1.85900+ 7
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.11000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.11000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 5.11000+ 6

98-Cf-251g

Mat.No: 7895

Lab: LIVERMORE  
Author: R.J.HUERTON  
Ref: UCRL-5400, VOL. 15  
Card Images: 2007

File Type	Reaction Type	Q-value
General Information	Descriptive data and Dictionary <v> : average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-5.10000+ 6 -1.17300+ 7 2.00000+ 8 -1.73200+ 7 6.17000+ 6
Angular dist. of secondary neutrons	<u> Lab:average elastic scattering cosine. <e>:average log energy decrement for elastic scat. <p> : <ee>/<2e>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-5.10000+ 6 -1.17300+ 7 2.00000+ 8 -1.73200+ 7
Photon prod.Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.17000+ 6
Photon prod.cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.17000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 6.17000+ 6

98-Cf-252g	Mat. No: 7896 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 3049
File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary  <u></u> average total(prompt+delayed)neutrons/fission Delayed neutrons from fission Prompt neutrons from fission	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-6.17000+ 6 -1.12800+ 7 2.00000+ 8 -1.79000+ 7 4.79000+ 6
Angular dist. of secondary neutrons	<u></u> Lab:average elastic scattering cosine. <ex>:average log energy decrement for elastic scat. <gp> : <gp>/<gp>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section Total fission cross section(sum of MT=19to21,38) (n,4n) cross section (n,n') to the continuum	-6.17000+ 6 -1.12800+ 7 2.00000+ 8 -1.79000+ 7
Photon prod. Multiplicities(from neut.reactions)	Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.79000+ 6
Photon prod. cross sections(from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.79000+ 6
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.79000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) Total fission cross section(sum of MT=19to21,38) (n,g) radiative capture cross section	2.00000+ 8 4.79000+ 6

Fiss. Prod.	Mat. No: 7897 Date: _____ Ref: UCRL-5400, VOL. 15	Lab: LIVERMORE Author: R.J.HOWERTON Card Images: 1237
File Type	Reaction Type	Q-Value
General Information	Descriptive data and Dictionary	
Resonance parameter data	Resonance information	
Neutron cross sections	Total cross section(sum of partials) Elastic scattering cross section Total inelastic cross section(sum of MT=51to91) direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum (n,g) radiative capture cross section	-8.00000+ 6 -1.53500+ 7 8.00000+ 6
Angular dist. of secondary neutrons	<u></u> Lab:average elastic scattering cosine. <ex>:average log energy decrement for elastic scat. <gp> : <gp>/<gp>	
Energy dist. of secondary neutrons	Elastic scattering cross section direct (n,2n) cross section (n,3n) cross section (n,n') to the continuum	-8.00000+ 6 -1.53500+ 7
Photon prod. Multiplicities(from neut.reactions)	(n,g) radiative capture cross section	8.00000+ 6
Photon prod. cross sections(from neut.reactions)	Non elastic cross section(total-elastic)	
Angular dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.00000+ 6
Energy dist. of Photons (from neut.reactions)	Non elastic cross section(total-elastic) (n,g) radiative capture cross section	8.00000+ 6