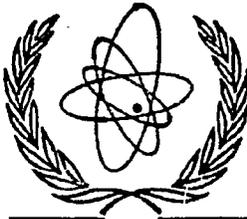


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INDC INTERNATIONAL NUCLEAR DATA COMMITTEE

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METHOD FOR THE INTERPOLATION, EVALUATION AND
COMPACT PRESENTATION OF INELASTIC NEUTRON
SCATTERING DATA

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Note:

This document (INDC-257E) is an English translation of an original (INDC-257) in Russian. The data tables 1-7 have been entered in the neutron data file DASTAR of the IAEA Nuclear Data Unit, under the following accession-numbers:

DASTAR-00721	Niobium (table 1)
DASTAR-00722	Iron (table 2)
DASTAR-00723	Nickel 58 (table 3)
DASTAR-00724	Nickel 60 (table 4)
DASTAR-00725	Lead 206 (table 5)
DASTAR-00726	Lead 207 (table 6)
DASTAR-00727	Lead 208 (table 7)

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IAEA Nuclear Data Unit
Kärntnerring 11-13
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68-8166

Translated from Russian

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METHOD FOR THE INTERPOLATION, EVALUATION AND COMPACT
PRESENTATION OF INELASTIC NEUTRON SCATTERING DATA

The authors propose a method for the interpolation, evaluation and compact presentation of data on the spectra of inelastically scattered neutrons [1], for subsequent use especially in calculating reactor constants.

All inelastic neutron scattering data take the form of a set of excitation functions for groups of nuclear levels with width 0.2 MeV, such that the mean energies in each group are 0.1; 0.3; 0.5; MeV. The excitation function of each group of levels is approximated by an analytical form of the type:

$$S(x) = \sum_{i=1}^3 C_i x^{d_i} \exp(-b_i x),$$

where X represents the excess initial energy of the neutrons above the excitation threshold for the level. C_i and d_i are expressed in turn through the parameters A , T , f , F as follows:

$$C_1 = AT^{-d_1}(1-f-F)\exp(d_1);$$

$$C_2 = AT^{-d_2}f\exp(d_2);$$

$$C_3 = AT^{-d_3}F\exp(d_3);$$

$$d_i = b_i T.$$

Seven parameters of an approximating function (A , T , b_1 , b_2 , b_3 , f , F) are found for each group of levels by comparison with the experimental data.

As a result, all information on inelastic neutron scattering for one nucleus in the initial energy range 0-15 MeV reduces to values for the 120 constants set out in Table 1 for niobium-93.

The first eight numbers in Table 1 indicate the numbers of the first eight conditional groups of levels containing as little as one real excitation level of the nucleus. The following 56 numbers indicate the values of seven parameters for each of the first eight levels. The remaining 56 constants are coefficients of the power series expansion of the energy dependence of each of the seven parameters:

$$A(E) = \sum_{i=0}^7 a_i E^i.$$

We can obtain the necessary information on inelastic neutron scattering by using the constants given in Table 1 and a computer programme. In particular, it is simple to calculate the spectra of inelastically scattered neutrons, level excitation functions, inelastic scattering cross-sections, grouped inelastic transition cross-sections, etc. The appropriate programmes in ALGOL-60 for the calculation of these values are given below.

The programme (No. 1) for calculating the spectra brings in as single figures the atomic weight of the nucleus (AT), the energy spacing (h), the block of numbers "M", consisting of the coefficients in Table 1 less the first column, which forms the following block "L", and, finally, the initial neutron energies (EN). The spectra are printed out with the given spacing h, the first point corresponding to zero energy. With a spacing of 0.2 MeV, the spectra are normalized with reference to the full inelastic cross-sections (in barns) plus the doubled cross-section for the (n, 2n) reaction.

The programme (No. 2) for calculating grouped inelastic transition cross-sections is based on formula 33 of Reference [2]. Here, the initial neutron spectrum is assumed to be close to the reactor spectrum, i.e. in the form $1/E$ where $E \leq 2$ MeV and $\exp(-0.72 E)$ with $E > 2$ MeV. The initial data for this programme are: integration step (h2, not greater than 0.05), number of groups (jj, not greater than 25), atomic weight (AT), number block "M" and "L" (the same as in the previous programme), block of boundary energies of the groups in descending order (E1 [1:26]- not more than 15 MeV). When there are less than 25 groups, block E 1 is increased to 26 by means of any differing numbers.

Programme No. 3 prints out the total inelastic neutron scattering cross-section (in barns) with the addition of the doubled cross-section for the (n, 2n) reaction as a function of the initial neutron energy. The energy spacing is 0.1 MeV, the first point corresponding to zero energy.

Programme No. 4 calculates the excitation function (in barns) of a group of nuclear levels with width 0.2 MeV and a given mean energy E.

Tables 2-7 contain co-efficients for natural iron and isotopes of lead and nickel.

REFERENCES

1. POPOV, V.I., SLUCHEVSKAYA, V.M., TRYKOVA, V.I: Paper ACC-68/11, Anglo-Soviet Seminar, Dubna, 1968.
2. Voprosy fiziki zaščity reaktorov (Problems of reactor shielding physics): Atomizdat, 1966.

Niobium

Table 1

Level No.	A	T	b_1	b_2	b_3	f	F
4	0,350	0,430	3,10	0,700	0,350	0,300	0,019
5	0,760	0,410	2,31	0,900	0,340	0,300	0,020
6	0,510	0,430	6,40	0,450	0,340	0,250	0,021
7	0,425	0,420	6,20	0,900	0,330	0,500	0,022
8	0,400	0,410	8,00	0,800	0,320	0,500	0,023
9	0,387	0,405	9,51	0,820	0,310	0,340	0,024
10	0,378	0,392	8,92	0,790	0,300	0,370	0,027
11	0,370	0,384	8,47	0,790	0,290	0,400	0,030

Table 1 continued

	A^*	T^*	b_1^*	b_2^*
a_0	0,502325	0,935691	0,294764. 10^2	0,472300. 10^1
a_1	-0,995131. 10^{-1}	-0,683107	-0,260377. 10^2	-0,561051. 10^1
a_2	0,264910. 10^{-1}	0,322234	0,129910. 10^2	0,292853. 10^1
a_3	-0,542351. 10^{-2}	-0,772896. 10^{-1}	-0,343117. 10^1	-0,714041
a_4	0,540908. 10^{-3}	0,108208. 10^{-1}	0,495361	0,921294. 10^{-1}
a_5	-0,185269. 10^{-4}	-0,878239. 10^{-3}	-0,394760. 10^{-1}	-0,653324. 10^{-2}
a_6	-0,3266674. 10^{-6}	0,379446. 10^{-4}	0,163303. 10^{-2}	0,241504. 10^{-3}
a_7	0,233698. 10^{-7}	-0,871972. 10^{-6}	-0,274016. 10^{-4}	-0,364573. 10^{-5}
	b_3^*	ψ^*	F^*	
a_0	0,753595	0,804476	-0,174117	
a_1	-0,632499	-0,898151	0,291110	
a_2	0,366272	0,560134	-0,166098	
a_3	-0,114493	-0,137140	0,485730. 10^{-1}	
a_4	0,194023. 10^{-1}	0,162376. 10^{-1}	-0,769760. 10^{-2}	
a_5	-0,176335. 10^{-2}	-0,990621. 10^{-3}	0,667053. 10^{-3}	
a_6	0,889324. 10^{-4}	0,294624. 10^{-4}	-0,297098. 10^{-4}	
a_7	-0,147439. 10^{-5}	-0,325788. 10^{-6}	0,531773. 10^{-6}	

1
2
1

№	Iron				Table 2		
5	0,880	1,44	1,50	0,75	0,21	0,55	0,044
8	0,060	1,18	1,30	0,60	0,21	0,50	0,053
11	0,153	1,65	1,50	1,50	0,21	0,45	0,070
14	0,170	0,93	1,50	1,50	0,21	0,40	0,098
15	0,155	0,90	1,50	1,50	0,21	0,30	0,115
16	0,142	0,80	2,50	1,05	0,21	0,40	0,120
17	0,115	0,75	2,50	1,20	0,21	0,40	0,140
18	0,115	0,75	2,50	1,20	0,21	0,40	0,140

a	-0,726993	0,104858	0,192200.10	0,1196600.10
	0,884885	0,942695	0,700000.10 ⁻²	-0,740000.10 ⁻¹
	-0,370570	-0,569410	0	0
	0,816060.10 ⁻¹	0,165627	0	0
	-0,103238.10 ⁻¹	-0,250409.10 ⁻¹	0	0
	0,750041.10 ⁻³	0,207589.10 ⁻²	0	0
	-0,289680.10 ⁻⁴	-0,903437.10 ⁻⁴	0	0
	0,460004.10 ⁻⁶	0,161447.10 ⁻⁵	0	0

0,210

0

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0,390

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0

0,140

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1

Nickel 58

Table 3

8	0,750	1,10	1,70	1,30	0,13	0,40	0,18
13	0,150	1,20	3,00	2,00	0,25	0,40	0,18
14	0,170	0,98	3,00	2,00	0,25	0,40	0,18
15	0,142	1,03	2,74	1,84	0,25	0,40	0,18
16	0,134	0,98	2,48	1,68	0,25	0,40	0,18
17	0,112	0,94	2,20	1,50	0,25	0,40	0,18
18	0,107	0,90	2,06	1,44	0,25	0,40	0,18
19	0,103	0,87	1,94	1,34	0,25	0,40	0,18

0,141141	0,112461.10 ²	0,109.10	0,130.10
-0,220868	-0,968317.10	0,200	0
0,278219.10 ⁻²	0,365318.10	0	0
-0,366106.10 ⁻³	-0,728360	0	0
0,528589.10 ⁻⁴	0,835490.10 ⁻¹	0	0
-0,532945.10 ⁻⁵	-0,552069.10 ⁻²	0	0
0,278046.10 ⁻⁶	0,194508.10 ⁻³	0	0
-0,560105.10 ⁻⁸	-0,282138.10 ⁻⁵	0	0

0,250	0,400	0,180
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0

Nickel 60

Table 4

7	0,583	1,27	1,60	1,30	0,12	0,40	0,18
11	0,180	0,94	3,00	2,30	0,25	0,40	0,18
12	0,180	0,94	3,00	2,30	0,25	0,40	0,18
13	0,150	0,80	2,50	2,50	0,25	0,40	0,18
14	0,150	0,80	2,50	2,50	0,25	0,40	0,18
16	0,123	0,80	2,18	1,96	0,25	0,40	0,18
17	0,116	0,79	2,00	1,50	0,25	0,40	0,18
18	0,108	0,81	1,92	1,44	0,25	0,40	0,18

0,470809.10 ⁻⁴	0,112461.10 ²	0,109.10	0,130.10
0,186409	-0,968317.10	0,200	0
-0,925975.10 ⁻¹	0,365318.10	0	0
0,207398.10 ⁻¹	-0,728360	0	0
-0,252979.10 ⁻²	0,835490.10 ⁻¹	0	0
0,173893.10 ⁻³	-0,552069.10 ⁻²	0	0
-0,634184.10 ⁻⁵	0,194508.10 ⁻³	0	0
0,956325.10 ⁻⁷	-0,282138.10 ⁻⁵	0	0

Table 4 continued

0,250
0
0
0
0
0
0
0
0

0,400
0
0
0
0
0
0
0
0

0,180
0
0
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0
0
0

Lead 206

Table 5

5	0,488	I,66	I,50	I,50	0,II	0,60	0,080
7	0,250	I,38	I,50	I,50	0,II	0,60	0,104
8	0,400	I,20	I,50	0,6I	0,II	0,60	0,II2
9	0,520	I,55	I,65	I,65	0,II	0,60	0,130
II	0,170	I,15	I,70	I,70	0,II	0,60	0,140
I2	0,170	I,15	I,75	I,75	0,II	0,60	0,140
I4	0,170	I,00	I,58	0,98	0,II	0,60	0,140
I6	0,170	I,00	I,83	0,60	0,II	0,60	0,140

0,102354.I0	0,268355.I0	0,173.I0	0,137634.I0
-0,969240	-0,538958	-0,560.I0 ⁻¹	-0,123902.I0
0,440724	-0,271318	0	0,524766
-0,104168	0,146782	0	-0,971397.I0 ⁻¹
0,138384.I0 ⁻¹	-0,265253.I0 ⁻¹	0	0,959970.I0 ⁻²
-0,103709.I0 ⁻²	0,233055.I0 ⁻²	0	-0,525617.I0 ⁻³
0,409733.I0 ⁻⁴	-0,101166.I0 ⁻³	0	0,149935.I0 ⁻⁴
-0,663906.I0 ⁻⁶	0,174077.I0 ⁻⁵	0	-0,172717.I0 ⁻⁶

0,II	0,6	0,14
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0

Lead 207

Table 6

3	0,730	I,60	2,00	I,35	0,II	0,60	0,I4
5	0,560	I,78	2,00	2,00	0,II	0,60	0,I4
9	0,I30	I,30	2,00	I,40	0,II	0,60	0,I4
I2	0,I40	I,26	4,00	I,55	0,II	0,60	0,I4
I4	0,420	I,04	2,50	0,95	0,II	0,60	0,I4
I6	0,320	0,90	I,80	I,20	0,II	0,60	0,I4
I8	0,220	I,60	2,00	I,70	0,II	0,60	0,I4
I9	0,200	0,80	3,00	I,50	0,II	0,60	0,I4

0,I02354.I0	0,268355.I0	0,I73.I0	0,I37634.I0
-0,969240	-0,538958	-0,560.I0 ^{-I}	-0,I23902.I0
0,440724	-0,27I3I8	0	0,524766
-0,I04I68	0,I46782	0	-0,97I397.I0 ^{-I}
0,I38384.I0 ^{-I}	-0,265253.I0 ^{-I}	0	0,959970.I0 ⁻²
-0,I03709.I0 ⁻²	0,233055.I0 ⁻²	0	-0,5256I7.I0 ⁻³
0,409733.I0 ⁻⁴	-0,I0II66.I0 ⁻³	0	0,I49935.I0 ⁻⁴
-0,663906.I0 ⁻⁶	0,I74077.I0 ⁻⁵	0	-0,I727I7.I0 ⁻⁶

0,II	0,6	0,I4
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0

Lead 208

Table 7

I4	0,700	1,30	3,5	1,0	0,18	0,5	0,18
I6	0,203	0,65	4,0	1,0	0,18	0,4	0,18
I7	0,202	0,66	4,0	1,0	0,18	0,4	0,18
I8	0,200	0,66	4,0	1,0	0,18	0,4	0,18
I9	0,200	0,77	4,0	1,0	0,18	0,5	0,18
20	0,244	0,75	3,0	1,0	0,18	0,6	0,18
21	0,238	0,72	3,0	1,0	0,18	0,6	0,18
22	0,234	0,72	3,0	1,0	0,18	0,6	0,18

-0,120828.10	0,268355.10	0,362.10	1
0,787958	-0,538958	-0,152	0
-0,120431	-0,271318	0	0
-0,733367.10 ⁻²	0,146782	0	0
0,397512.10 ⁻²	-0,265253.10 ⁻¹	0	0
-0,440108.10 ⁻³	0,233055.10 ⁻²	0	0
0,210243.10 ⁻⁴	-0,101166.10 ⁻³	0	0
-0,379657.10 ⁻⁶	0,174077.10 ⁻⁵	0	0

0,18	0,6	0,18
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0
0	0	0

ALGOL programme No. 1 for calculating the recommended
inelastically scattered neutron spectra.

```
begin real a, b, h, g, x, t, E, E1, EN, A, T, b1, b2, b3,  
f, F, d1, d2, d3, C1, C2, C3, En, AT, SN, W;  
integer i, j, k, m, n;  
array  $\mu$ [1:112], Aa, Ta, ba1, ba2, ba3, fa, Fa[1:50],  
Ab, Tb, bb1, bb2, bb3, fb, Fb[1:8], S, SP[0:150];  
integer array L[1:8];  
inout ('2 10-2', AT, h,  $\mu$ , L);  
L3: inout ('2 10-2', EN); n := L[8];  
for i := 1 step 1 until n do begin  
Aa[i] := Ta[i] := ba1[i] := ba2[i] := ba3[i] := fa[i] := Fa[i] := 0  
end;  
for i := 1 step 1 until 8 do begin j := L[i];  
Aa[j] :=  $\mu$ [i]; Ta[j] :=  $\mu$ [i+8]; ba1[j] :=  $\mu$ [i+16];  
ba2[j] :=  $\mu$ [i+24]; ba3[j] :=  $\mu$ [i+32]; fa[j] :=  $\mu$ [i+40];  
Fa[j] :=  $\mu$ [i+48]; Ab[i] :=  $\mu$ [i+56]; Tb[i] :=  $\mu$ [i+64];  
bb1[i] :=  $\mu$ [i+72]; bb2[i] :=  $\mu$ [i+80]; bb3[i] :=  $\mu$ [i+88];  
fb[i] :=  $\mu$ [i+96]; Fb[i] :=  $\mu$ [i+104]  
end;  
SN := 0; g := AT / (AT + 1);  
for j := 0 step 1 until 150 do begin  
E1 := j * h; E := EN * g - E1 / g;  
if E < 0 then begin SP[j] := 0; goto L1 end;  
for i := 1 step until n do begin  
if E  $\geq$  0.2 * (i-1) then begin if E < 0.2 * i then begin  
A := Aa[i]; T := Ta[i]; b1 := ba1[i]; b2 := ba2[i];  
b3 := ba3[i]; f := fa[i]; F := Fa[i]; goto L2 end  
end end;
```

$A := A\delta[8]; T := T\delta[8]; \delta 1 := \delta\delta 1[8]; \delta 2 := \delta\delta 2[8];$

$\delta 3 := \delta\delta 3[8]; \varphi := \varphi\delta[8]; F := F\delta[8];$

for $k := 7$ step -1 until 1 do begin

$A := A * E + A\delta[k]; T := T * E + T\delta[k]; \delta 1 := \delta 1 * E + \delta\delta 1[k];$

$\delta 2 := \delta 2 * E + \delta\delta 2[k]; \delta 3 := \delta 3 * E + \delta\delta 3[k]; \varphi := \varphi * E + \varphi\delta[k];$

$F := F * E + F\delta[k]$ end;

L2: $d 1 := \delta 1 * T; d 2 := \delta 2 * T; d 3 := \delta 3 * T;$

if $T > 0$ then $\delta := \delta(T)$ else $\delta := 0;$

$C 1 := A * \exp(d 1 * (1 - \delta)) * (1 - \varphi - F);$

$C 2 := A * \exp(d 2 * (1 - \delta)) * \varphi; C 3 := A * \exp(d 3 * (1 - \delta)) * F;$

$E_n := E / g; x := EN - E_n; \text{if } x > 0 \text{ then } \text{begin } a := \delta_n(x);$

$W := C 1 * \exp(d 1 * a - \delta 1 * x) + C 2 * \exp(d 2 * a - \delta 2 * x) +$
 $C 3 * \exp(d 3 * a - \delta 3 * x)$ end else $W := 0;$

$SP[j] := W / g; SN := SN + SP[j];$

L1: end; $SN := SN * h / 0.2;$

inout ('p2-10', EN, SN, SP); goto L3 end;

ALGOL programme No. 2 for calculating grouped inelastic transition cross-sections.

```
begin real a, b, h1, h2, h3, A, T, b1, b2, b3, f, F, d1, d2, d3  
E, EN, En, AT, g, t, C1, C2, C3, S, Cd, al, bl, x, W;  
integer i, j, k, m, n, jj, l, ii;  
array M[1:112], Aa, Ta, ba1, ba2, ba3, fa, Fa[1:50],  
Ab, Tb, bb1, bb2, bb3, fb, Fb[0:8], MA, MT, Mb1, Mb2, Mb3,  
Mf, MF[1:150], E1[0:25], C2[0:30], Sm[1:25],  
SG[1:25], G[1:10];  
integer array L[1:8];  
inout('2 10-2', h2, jj, AT, M, L, E1);  
h1:=0.1; h3:=0.5; g:=AT/(AT+1); n:=L[8];  
for i:=1 step 1 until n do begin  
Aa[i]:=Ta[i]:=ba1[i]:=ba2[i]:=ba3[i]:=fa[i]:=Fa[i]:=0  
end;  
for i:=1 step 1 until 8 do begin j:=L[i];  
Aa[j]:=M[i]; Ta[j]:=M[i+8]; ba1[j]:=M[i+16];  
ba2[j]:=M[i+24]; ba3[j]:=M[i+32]; fa[j]:=M[i+40];  
Fa[j]:=M[i+48]; Ab[i]:=M[i+56]; Tb[i]:=M[i+64];  
bb1[i]:=M[i+72]; bb2[i]:=M[i+80]; bb3[i]:=M[i+88];  
fb[i]:=M[i+96]; Fb[i]:=M[i+104]  
end;  
for j:=1 step 1 until 150 do begin  
E:=(2*j-1)*h1/2;  
for i:=1 step 1 until n do begin  
if E ≥ 0.2*(i-1) then begin if E < 0.2*i then begin  
A:=Aa[i]; T:=Ta[i]; b1:=ba1[i]; b2:=ba2[i];  
b3:=ba3[i]; f:=fa[i]; F:=Fa[i]; goto L2 end  
end end;  
L2:  
A:=Ab[8]; T:=Tb[8]; b1:=bb1[8]; b2:=bb2[8];  
b3:=bb3[8]; f:=fb[8]; F:=Fb[8];
```

```
for k = 7 step 1 until 1 do begin
  A = A * E + A6[k]; T = T * E + T6[k]; b1 = b1 * E + b61[k];
  b2 = b2 * E + b62[k]; b3 = b3 * E + b63[k]; f = f * E + f6[k];
  F = F * E + F6[k] end;
L2: MA[j] = A, MT[j] = T; M61[j] = b1; M62[j] = b2;
  M63[j] = b3; Mf[j] = f; MF[j] = F
  end;
for j = 1 step 1 until 25 do begin S = 0;
  EN = E1[j] - h2/2;
L4: EN = EN + h2; if EN > E1[j-1] + h2/2 then goto L5;
  if EN < 2 then Cd = 1.042/EN else Cd = exp(-0.72 * EN);
  if abs(EN - E1[j-1]) > (h2/2 + 10 - 4) then m = 1
  else begin Cd = Cd * (E1[j-1] - (EN - h2/2)) / h2; m = 0 end;
  S = S + Cd; if m = 1 then goto L4;
L5: Sm[j] = S * h2 / (E1[j-1] - E1[j])
  end;
for i = 1 step 1 until 25 do SG[i] = 0;
for j = 1 step 1 until 25 do begin
  for k = 1 step 1 until 25 do begin
    S = 0; if j > jj then goto L8;
    if k > jj then goto L7;
    if k < j then goto L3;
    for l = 1 step 1 until 150 do begin E = h1 * (l - 0.5);
      al = E1[k-1] + E/g; if E1[j-1] < al then al = E1[j-1];
      bl = E1[k] + E/g; if E1[j] > bl then bl = E1[j];
      if al > bl then begin EN = bl - h2/2;
L1: EN = EN + h2; if EN > al + h2/2 then goto L6;
      if EN < 2 then Cd = 1.042/EN else Cd = exp(-0.72 * EN);
      A = MA[l], T = MT[l]; b1 = M61[l]; b2 = M62[l];
      b3 = M63[l]; f = Mf[l]; F = MF[l]; H = b1 * T;
      A2 = b2 * T; A3 = b3 * T;
```

```
if  $T > 0$  then  $b := \ln(T)$  else  $b := 0$ ;  
L1:  $C1 := A * \exp(d1 * (1 - b)) * (1 - f - F)$ ;  
 $C2 := A * \exp(d2 * (1 - b)) * f$ ;  $C3 := A * \exp(d3 * (1 - b)) * F$ ;  
 $En := E/g$ ;  $x := EN - En$ ;  
if  $x > 0$  then begin  $a := \ln(x)$ ;  
 $W := C1 * \exp(d1 * a - b1 * x) + C2 * \exp(d2 * a - b2 * x) +$   
 $C3 * \exp(d3 * a - b3 * x)$  end else  $W := 0$ ;  
if  $\text{abs}(EN - a1) > (h2/2 + 10^{-4})$  then  $m := 1$   
else begin  $W := W * (a1 - (EN - h2/2)) / h2$ ;  $m := 0$  end;  
 $S := S + W * Cd$ ; if  $m = 1$  then goto L1 end; L6: end;  
 $S := S * h2 * h1 / 0.2 / Sm[j] / (E1[j-1] - E1[j])$ ;  
L3:  $SG[k] := S$  end; L7: inout('p2-10', j, SG) end;  
L8: end;
```

ALGOL programme No. 3 for calculating recommended inelastic
neutron cross-sections.

```
begin real a, b, h, g, t, x, W, E, E1, EN, En, B1, B2, B3,  
F1, f1, D1, D2, D3, AT;  
integer i, j, k, m, n;  
array M[1:112], A, T, b1, b2, b3, f, F, c1, c2, c3, d1,  
d2, d3[1:76], SN[0:150], Ab, Tb, bb1, bb2, bb3, fb,  
Fb[1:8];  
integer array L[1:8];  
inout ('2 10-2', AT, M, L);  
g := AT/(AT+1); h := 0.1; SN[0] := 0; n = L[8];  
for i := 1 step 1 until n do begin  
A[i] := T[i] := b1[i] := b2[i] := b3[i] := f[i] := F[i] := 0  
end;  
for i := 1 step 1 until 8 do begin j := L[i];  
A[j] := M[i]; T[j] := M[i+8]; b1[j] := M[i+16];  
b2[j] := M[i+24]; b3[j] := M[i+32]; f[j] := M[i+40];  
F[j] := M[i+48]; Ab[i] := M[i+56]; Tb[i] := M[i+64];  
bb1[i] := M[i+72]; bb2[i] := M[i+80]; bb3[i] := M[i+88];  
fb[i] := M[i+96]; Fb[i] := M[i+104]  
end;  
m := n-1;  
for i := m step 1 until 76 do begin  
E := (2*i-1)*0.1;  
a := Ab[8]; t := Tb[8]; B1 := bb1[8]; B2 := bb2[8];  
B3 := bb3[8]; F1 := fb[8]; f1 := Fb[8];  
for k := 7 step -1 until 1 do begin  
a := a * E + Ab[k]; t := t * E + Tb[k]; B1 := B1 * E + bb1[k];  
B2 := B2 * E + bb2[k]; B3 := B3 * E + bb3[k];  
F1 := F1 * E + fb[k]; f1 := f1 * E + Fb[k]  
end;
```

```
A[i] = a, T[i] = t, b1[i] := B1; b2[i] := B2; b3[i] := B3;
f[i] := F1, F[i] := f1 end;
for i := 1 step 1 until 76 do begin
  d1[i] := b1[i] * T[i]; d2[i] := b2[i] * T[i]; d3[i] := b3[i] * T[i];
  if T[i] > 0 then b := ln(T[i]) else b := 0;
  C1[i] := A[i] * exp(d1[i] * (1 - b)) * (1 - f[i] - F[i]);
  C2[i] := A[i] * exp(d2[i] * (1 - b)) * f[i];
  C3[i] := A[i] * exp(d3[i] * (1 - b)) * F[i]
  end;
for j := 1 step 1 until 150 do begin
  SN[j] := 0; EN := h * j;
  for i := 1 step 1 until 76 do begin
    E = (2 * i - 1) * 0.1; En := E / g; x := EN - En;
    if x > 0 then begin a := ln(x);
      W := C1[i] * exp(d1[i] * a - b1[i] * x) +
        C2[i] * exp(d2[i] * a - b2[i] * x) +
        C3[i] * exp(d3[i] * a - b3[i] * x)
      end else W := 0;
    SN[j] := SN[j] + W end
  end;
  end;
  inout ('p2-10', SN) end;
```

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ALGOL programme No. 4 for calculating recommended excitation functions for groups of nuclear levels.

```

begin real a, b, h, g, x, t, E, E1, EN, En, A, T, b1, b2, b3,
      f, F, d1, d2, d3, c1, c2, c3, AT, SN, W;
integer i, j, k, m, n;
array M[1:112], Aa, Ta, ba1, ba2, ba3, fa, Fa[1:50],
      Ab, Tb, b61, b62, b63, f6, F6[1:8], S, SP[0:150];
integer array L[1:8];
inout ('z 10-2', AT, h, M, L);
L3: inout ('z 10-2', E);
n := L[8];
for i := 1 step 1 until n do begin
  Aa[i] := Ta[i] := ba1[i] := ba2[i] := ba3[i] := fa[i] := Fa[i] := 0
  end;
for i := 1 step 1 until 8 do begin j := L[i];
  Aa[j] := M[i]; Ta[j] := M[i+8]; ba1[j] := M[i+16];
  ba2[j] := M[i+24]; ba3[j] := M[i+32]; fa[j] := M[i+40];
  Fa[j] := M[i+48]; Ab[i] := M[i+56]; Tb[i] := M[i+64];
  b61[i] := M[i+72]; b62[i] := M[i+80]; b63[i] := M[i+88];
  f6[i] := M[i+96]; F6[i] := M[i+104]
  end;
g := AT / (AT + 1);
for i := 1 step 1 until n do begin
  if E ≥ 0.2 * (i - 1) then begin if E < 0.2 * i then begin
    A := Aa[i]; T := Ta[i]; b1 := ba1[i]; b2 := ba2[i];
    b3 := ba3[i]; f := fa[i]; F := Fa[i]; goto L2 end
  end end;
A := Ab[8]; T := Tb[8]; b1 := b61[8]; b2 := b62[8]; b3 := b63[8];
f := f6[8]; F := F6[8];
i := i - 7 step -1 until 1 do begin
  A := A * E + Ab[k]; T := T * E + Tb[k]; b1 := b1 * E + b61[k];
  b2 := b2 * E + b62[k]; b3 := b3 * E + b63[k]; f := f * E + f6[k];
  F := F * E + F6[k]
  end;

```

```
L2: d1:=b1*T; d2:=b2*T; d3:=b3*T;
  if T>0 then b:=ln(T) else b:=0;
  C1:=A*exp(d1*(1-b))*(1-f-F);
  C2:=A*exp(d2*(1-b))*f; C3:=A*exp(d3*(1-b))*F;
  En:=E/g;
  for i:=0 step 1 until 150 do begin
    EN:=i*h; x:=EN-En;
    if x>0 then begin a:=ln(x);
      W:=C1*exp(d1*a-b1*x)+C2*exp(d2*a-b2*x)+
        C3*exp(d3*a-b3*x)
      end else W:=0; S[i]:=W
    end;
  inout('p2-10', E, S); goto L3
end;
```