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PROGRESS REPORT ON NUCLEAR DATA RESEARCH IN THE FEDERAL REPUBLIC OF GERMANY

for the Period April 1, 1983 to March 31, 1984

June 1984

Edited by S.M. Qaim

Institut für Chemie (1): Nuklearchemie Kernforschungsanlage Jülich GmbH

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FOREWORD

This report has been prepared to promote exchange of nuclear data research information between the Federal Republic of Germany and the other member states of NEA and IAEA. It brings together progress reports from KfK Karlsruhe, KFA Jülich, GKSS-Geesthacht, the Universities of Kiel, Köln, Mainz, Marburg, Stuttgart and München, as well as from PTB Braunschweig and FIZ Karlsruhe. The emphasis in the work reported here has been on measurement, evaluation and compilation of application oriented nuclear data, such as those relevant to fission and fusion reactor technologies, development of intense spallation neutron sources, production of medically important short-lived radioisotopes etc.

Each contribution is presented under the laboratory heading where the work was done. If the work is relevant to requests in the World Request List for Nuclear Data, WRENDA 81/82 (INDC(SEC)-78/URSF), the corresponding request identification numbers have been listed after the title and authors' names of the respective contribution.

Jülich, June 1984

S.M. Qaim

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- 2. Systematics of Angular-Dependent Neutron and Charged-Particle 1 Production by 590 MeV Protons on Thin Targets with $12 \le A \le 238$ S. Cierjacks, Y. Hino, F. Raupp
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KERNFORSCHUNGSZENTRUM KARLSRUHE INSTITUT FÜR KERNPHYSIK II

1. Documentation of Neutron and Charged Particle Yields and Spectra from High-E.ergy Proton Bombardment of Thick Heavy Metal Targets

S. Cierjacks, F. Raupp

The complete documentation of differential neutron and charged particle yields and spectra from 590 MeV and 1100 MeV proton bombardment of thick heavy metal targets measured within the feasibility study for a new German spallation neutron source (SNQ) has been finalized. While the data obtained for thick lead targets at the two incident proton energies have been documented in recent years, final results for thick uranium targets were compiled in a recent report [1]. The data are presented in tabulated and graphical form, in order to facilitate their use in subsequent work on the study, construction and operation of advanced spallation neutron sources. Absolute yield and spectra measurements presented in the report concern neutron and charged particle production from 590 MeV proton bombardment of a 10 x 10 cm², 40 cm long, uranium block. Differential data are given for three laboratory angles of 30°, 90° and 150° and five penetration depths (in steps of 5 cm) into target. Charged particle yields and spectra have been measured for secondary protons, deuterons, tritons and pions ($\pi^{+} + \pi^{-}$).

2. Systematics of Angular-Dependent Neutron and Charged-Particle Production by 590 MeV protons on Thin Targets with $12 \le A \le 238$

S. Cierjacks, Y. Hino¹, F. Raupp

Data analyses of all neutron production cross section measurements performed in recent years at the SIN cyclotron have been completed. The results are presently used for extended intercomparisons of measurements and theoretical predictions provided by the HETC Monte Carlo code [2] (see Sect. 3). A journal publication of the experimental results of neutron production cross sections is in progress. The analysis of charged-particle production cross section measurements performed for various metal targets and several emission angles has been continued during the period covered in this report. Their final evaluation is expected to give differential production cross sections for secondary protons, deuterons, tritons and pions at five laboratory angles between 23° and 157° and thin metal targets of C, Al, Fe, Nb, In, Ta, Pb and U. Experimental data of this type provide valuable additional information for comparisons with theoretical predictions from modern intranuclear cascade models.

3. <u>Comparisons of Measured and Calculated Differential Neutron and</u> Charged-Particle Production Cross Sections

S. Cierjacks, Y. Hino¹, D. Filges², P. Cloth², T.W. Armstrong²

The systematic measurements of neutron and charged-particle production cross sections [3] (described in Sect. 2) were primarily performed in order to test the accuracy of theoretical predictions from intranuclear cascade evaporation models used in modern high-energy nucleon-meson transport codes, e.g. the HETC Monte Carlo code. In continuation of our previous work various additional total and partial production cross sections have been calculated. The previous and the new calculations now provide a complete set of cross sections which cover the whole range of measured neutron and part of the charged-particle production data [4]. Comparisons of experimental and theoretical results revealed that the neutron production cross sections in the evaporation region $(E_n \lesssim 15 \text{ MeV})$ can be predicted for all targets except the very lightest (C and Al) and all emission angles with sufficient accuracy (^30%). In the cascade region, however, large discrepancies continue to persist. The differences between measurements and theoretical predictions increase with increasing emission energy and increasing emission angle. Typically the calculated cross sections at $E_n \sim 100$ MeV and emission angles in the forward direction are a factor of 2-3 lower than the measured ones. The corresponding factor increases to a value of about 10 for a backward laboratory angle of 150°.

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- F. Raupp, Messung der orts- und winkelabhängigen Spektren schneller Neutronen und geladener Sekundärteilchen aus Spallationsreaktionen von 590 MeV Protonen in dicken Urantargets, KfK-Report, KfK 3511B, Kernforschungszentrum Karlsruhe, April 1983
- T.W. Armstrong and K.C. Chandler, HETC A High-Energy Transport Code, Nucl. Sci. Eng. 49 (1972) 110
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KERNFORSCHUNGSZENTRUM KARLSRUHE INSTITUT FÜR KERNPHYSIK III

1. <u>3 MV Van de Graafr-Accelerator</u>

1.1 The Capture Width of the 34.8 keV s-Wave Neutron Resonance in ²⁷A1*

K. Wisshak, F. Käppeler, G. Reffo⁺

The neutron capture width of the s-wave resonance at 34.8 keV in ²⁷Al has been determined using a setup with extremely low neutron sensitivity. This feature is important because this resonance exhibits a very large scattering to capture ratio. A pulsed 3-MV Van de Graaff accelerator and a kinematically collimated neutron beam, produced via the ⁷Li(p,n) reaction. was used in the experiment. Capture gamma-rays were observed by three Moxon-Rae detectors with graphite-, bismuth-graphite-, and bismuth-converter, respectively. The samples were positioned at a neutron flight path of only 9 cm. Thus events due to capture of resonance scattered neutrons in the detectors or in surrounding materials are completely discriminated by their additional time of flight. The data obtained with the individual detectors were corrected for the efficiency of the different converter materials. For that purpose, theoretical calculations of the capture gamma-ray spectra of the measured isotope and of gold, which was used as a standard, were performed. The final radiative width is $g\Gamma_{v} = 1.22 \pm 0.07$ eV. The accuracy is nearly a factor of three better than in previous experiments.

* submitted for publ. to Nucl. Sci. Eng. E.N.E.A., Bologna, Italy

1.2 Neutron Capture Cross Sections of ⁴⁶Ca and ⁴⁸Ca

F. Käppeler, G. Walter, G.J. Mathews⁺

The neutron capture cross sections of 46,48 Ca were measured for the first time by activation of a few mg of sample material. The neutron energy spectra used in these activations peaked at 25 and 97 keV. The results obtained for 48 Ca agree very well with a 1/v-extrapolation of the thermal cross section while the 46 Ca results lie by a factor of \sim 5 above such an extrapolation. The

cross sections are of relevance to an understanding of the origin of 46,48 Ca.

University of California Lawrence Livermore National Laboratory, USA

1.3 Neutron Capture in s-Wave Resonances of ⁶⁴Ni^{*}

K. Wisshak, F. Käppeler, R.L. Macklin⁺, G. Reffo⁺⁺, F. Fabbri⁺⁺

The neutron capture width of the s-wave resonances at 13.9 and 33.8 keV in 64 Ni have been determined using a setup with extremely low neutron sensitivity completely different from all previous experiments on this isotope. This feature is important because these resonances exhibit a very large scattering to capture ratio. A pulsed 3-MV Van de Graaff accelerator and a kinematically collimated neutron beam, produced via the 7 Li(p,n) reaction, was used in the experiments. Capture gamma-rays were observed by three Moxon-Rae detectors with graphite-, bismuth-graphite-, and bismuth-converter, respectively. The samples were positioned at a neutron flight path of only 6-8 cm. Thus events due to capture of resonance scattered neutrons in the detectors or in sourrounding materials are completely discriminated by their additional time of flight. The short flight path and the high neutron flux at the sample position allowed for a signal to background ratio of \sim 1 even for the broad resonance at 33.8 keV. The data obtained with the individual detectors were corrected for the efficiency of the different converter materials. For that purpose, detailed theoretical calculations of the capture gamma-ray spectra of the measured isotope and of gold, which was used as a standard, were performed. The final radiative widths are $\Gamma_{1}(13.9 \text{ keV}) = 1.01 \pm 0.07 \text{ eV}$ and $\Gamma_{1}(33.8 \text{ keV}) = 1.16 \pm 0.08 \text{ eV}$, considerably smaller than the rough estimates obtained in previous work.

++ E.N.E.A., Bologna, Italy

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KfK-report 3582, and Nucl. Sci. Eng. (in press)

⁺ Oak Ridge National Laboratory, USA

1.4 <u>Neutron Capture Cross Sections of the Krypton Isotopes and the</u> <u>s-Process Branching at ⁷⁹Se</u>*

G. Walter, B. Leugers, F. Käppeler, Z.Y. Bao⁺, D. Erbe, G. Rupp, G. Reffo⁺⁺, F. Fabbri⁺⁺

The input data for an analysis of the s-process branching at 79 Se have been significantly improved. The neutron capture cross sections for the stable krypton isotopes (except 86 Kr) were measured between 3 and 240 keV neutron energy. In addition, statistical model calculations of the (n,γ) -cross sections for all isotopes involved in this branching were performed. With these data and with other experimental results from literature a recommended set of Maxwellian average cross sections was established in the mass region 77<A<85. The relevant decay parameters of the involved unstable nuclei and the parameters for the s-process model are discussed as well. On this basis the following aspects are investigated: the temperature during the s-process, the decomposition into s- and r-process contributions and the solar krypton abundance.

1.5 Neutron Capture Cross Sections of the Stable Xenon Isotopes and their Application in Stellar Nucleosynthesis

H. Beer, F. Käppeler, G. Reffo⁺, G. Venturini⁺

The neutron capture cross sections of 124,132,134 Xe have been measured by the activation technique at 25 keV neutron energy. These data were supplemented by calculated capture cross sections for 128,129,130,131 Xe via the statistical model, based on carefully evaluated local parameter systematics. The complete set of capture cross sections obtained in this way is shown in Table I together with previous, more global calculations. The uncertainty of our theoretical results is estimated to be 20 %.

^{*} KfK-report 3652 (1984)

on leave from the Institute of Atomic Energy, Academia Sinica, Peking, China ** E.N.E.A., Bologna, Italy

Xe-	σ _{nγ} (mb)						
Isotopes	Harris (1)	Holmes et al.(2)	ENDF/B-V (3)	Benzi et al.(4)	presen calc.	t work exp.	
128	510	232	206	239	249		
129	1454	666	560	572	470		
130	207	143	156	189	153		
131	570	587	310	491	348		
132	74.9	91	85	116	65	61+4	
134	37.5	47.5	4 î	60	33	29+2	

Table I Comparison of the Xenon Cross Sections Obtained in this Work with Various Calculations at kT = 30 keV

With these cross sections we determined the solar xenon abundance through s-process systematics and decomposed the abundances of ^{129,131,132} Xe into their s- and r-process contributions. Furtheron, various isotopic anomalies in the xenon isotopes, which were detected in several meteorites, are discussed in de-tail.

- (1) M.J. Harris, Astrophys. Space Sci. 77 (1981) 357
- (2) J.A. Holmes, S.E. Woosley, W.A. Fowler, B.A. Zimmerman, At. Data and Nucl. Data Tables <u>18</u> (1978)
- (3) Kinsey, ENDF/B Summary Documentation, BNL-NCS-17541 (1979)
- (4) V. Benzi, R.D'Orazi, G. Reffo, M. Vaccari, CNEN-report RT/FI6 (1972)
- Astrophysics and Space Science <u>97</u> (1983) 95
- E.N.E.A., Bologna, Italy

1.6 <u>Neutron Capture Nucleosynthesis of Neodymium Isotopes and the s-Process</u> from A = 130 to 150 *

G.J. Mathews, F. Käppeler

New measurements of neutron capture cross sections for 142,143,144 _{Nd are} 'reported. These are combined with other recent measurements and applied to a detailed study of the s-process and r-process systematics for A = 130 to 150 nuclei. The influence of these results on the interpretation of isotopic

submitted to the Astrophysical Journal

^{*} University of California, Lawrence Livermore National Laboratory, USA

anomalies observed in acid insoluble residues and inclusions from the Allende meteorite is also examined. The uncertainties in the s-process oN curve are significantly diminished in the present work and a fit is obtained which is consistent with all of the s-process-only isotopes in this region. A somewhat larger value than previous determinations is obtained for the mean neutron exposure for heavy nuclei in the s-process, $\tau_0 = 0.29 - 0.35 \text{ mb}^{-1}$. The derived r-process abundances decrease systematically from the A = 130 peak but exhibit a pronounced odd-even effect. The new results tend to confirm the hypothesis that the isotopic anomalies in materials from the Allende meteorite are the result of an unusual mixture of average solar-system s-process and r-process material, but a previously unobserved odd-even effect may be present in the r-process anomalies of inclusion EK1-4-1.

1.7 The s-Process Branching at 151 *

H. Beer, F. Käppeler, K. Yokoi⁺, K. Takahashi⁺

The s-process branching in the mass region $150 \le A \le 154$, initiated by the 151 Sm B⁻ decay, is reinvestigated, particularly in connection with the solar 152 Gd abundance. The Maxwellian averaged neutron capture cross sections for kT = 25 keV are measured for 152 Sm, 151 Eu (to the 9.3 hour isomeric state of 152 Eu), 152 Gd, 158 Gd, and 160 Gd. The B-decay rates of the unstable nuclei involved in the branching are calculated theoretically. In addition, it is shown that the thermal equilibration between the ground state and the isomeric state in 152 Eu under plausible s-process conditions is achieved on a time scale shorter than those for B-decay and neutron capture. With these results and the neutron capture cross sections from literature for the other concerned nuclei, a branching analysis is performed within the steady flow model of the s-process. This study yields constraints for s-process models, particularly with regard to temperature and neutron density.

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^{* .}The Astrophysical Journal <u>278</u> (1984) 388

Technische Hochschule Darmstadt

1.8 Self-Absorption of Neutron Capture Gamma-Rays in Gold Samples

K. Wisshak, G. Walter, F. Käppeler

The self absorption of neutron capture gamma-rays in gold samples has been determined experimentally for two standard setups used in measurements of neutron capture cross sections. One makes use of an artificially collimated neutron beam and two C_6D_6 detectors, the other of kinematically collimated neutrons and three Moxon-Rae detectors. With a gold sample of 1 mm thickness correction factors up to 12 % were found for an actual neutron capture cross section measurement using the first setup while they are only 4 % for the second setup. The present data allow to determine the correction in an actual measurement with an accuracy of 0.5 - 1 %.

Nucl. Instr. Meth. 219 (1984) 136

1.9 Large Barium Fluoride Detectors

K. Wisshak, F. Käppeler

Large BaF₂ crystals of 1 - 2 l volume and up to 15 cm thickness were investigated with respect to their application as gamma-ray detectors. In particular, we were interested in the light transmission in the UV region, and the energy and time resolution. We found that an energy resolution of \sim 12 % (662 keV) and a time resolution of \sim 0.4 ns (⁶⁰Co, 300 keV threshold) can be obtained simultaneously. For these features BaF₂ is superior to NaI or BGO in cases where good timing is essential. Gamma-rays and alpha particles can be clearly discriminated, as for the latter the fast component does not show up in the scintillation light.

Nucl. Instr. Meth. (in press)

es (No

*

1.10 Calculated Efficiency of a 4π Detector of BGO or BaF₂ for Monoenergetic Gamma Rays and Gamma Cascades Following Neutron Capture*

K. Wisshak, F. Käppeler, G. Schatz

The applicability of a spherical shell of BGO or BaF_2 as a 4π detector for high precision measurements of neutron capture cross sections was investigated. Firstly, the efficiency of both scintillator materials for monoenergetic gamma rays was calculated in the energy range from 0.5 to 10 MeV. Configurations with different thickness and inner radii were considered. Secondly, neutron capture cascades were calculated for several isotopes with widely different capture gamma ray spectra according to the statistical model. Both informations together allowed to determine the efficiency of an actual detector for neutron capture events in dependence of the threshold energy. A thickness of 10 cm BGO or 17.5 cm BaF₂ proved to be sufficient to register more than 95 % of all capture events above a threshold energy of 3 MeV. This reduces the systematic uncertainty due to the detector efficiency in an absolute cross-section measurement to less than 1 % and in a relative measurement using a gold standard to less than 0.5 %.

Nucl. Instr. Meth. 221 (1984) 385

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KERNFORSCHUNGSZENTRUM KARLSRUHE INSTITUT FÜR NEUTRONENPHYSIK UND REAKTORTECHNIK

1. Nuclear Data Evaluation

1.1 Neutron Data for Fusion Applications

B. Goel, I. Broeders, U. Fischer, H. Jahn, B. Krieg,H. Kuesters, M. Segev, E. Stein, E. Wiegner

(Relevant to request numbers 724007F, 724008F, 732004F, 762246F, 821046F, 832036F, 832002F, 792112R, 781222F, 832043F, 724058F, 792023F, 801021F)

Reliable nuclear data for fusion are becoming more and more important as operation of facilities such as the Joint European Torus (JET), design studies for its successor (NET) and increasingly detailed conceptual studies of fusion reactors and fission-fusion hybrids create data demand for the calculation of activation, radiation damage, shielding, tritium breeding etc. Our recent activities were concentrated on

- assessment of the relevant cross sections in the evaluated neutron data file KEDAK which at present contains mainly neutron cross sections for fission reactor calculations,
- b) participation in a benchmark exercise concerning theoretical models and codes for pre-equilibrium reactions,
- c) critical analysis of Pb(n, 2n) cross section data.

During the HIBALL study (heavy-ion induced fusion with inertial confinement, [1]) the ⁷Li(n,n't) cross section was evaluated at KfK on the basis of then available "new" experiments and of the integral tritium-breeding experiment performed here [2]. The result of this evaluation together with another evaluation from Los Alamos [3] and the recent experimental data [4-7] are displayed in Fig. 1. At the time of our evaluation only preliminary data from Geel were available. In the region around 6 MeV our evaluation is a bit higher than the final Geel-Jülich data (Liskien et al.) but well within the experimental uncertainties. The Harwell data (Swinhoe and Uttley) in Fig. 1 are consistently lower than the evaluations and other experimental data. Revised Harwell data [8] obtained with more precise multiple scattering corrections are about 3-6 % higher than those reproduced in Fig. 1. The accuracy of our evaluation is about 7 %. This may be regarded as sufficient, especially because in most recent blanket designs the ⁷Li(n,n't) process contributes only a few percent to the total breeding ratio. Only in the case of a Li or Li₂O blanket is a higher accuracy required. In some of the blanket designs the use of ZrH_x is proposed [9]. KERMA factors for both Zr and H are missing in the data base used. To make up for this deficiency we have implemented the MACK-IV code [10] at KfK and will use it to generate KERMA factors.

For the treatment of precompound processes in nuclear reactions we have modified our version of the HAUSER*4 code [11] so as to allow simultaneous generation of (n,n') and (n,2n) neutron emission spectra. The pre-equilibrium contribution is calculated with the geometry-dependent hybrid model [12]. As shown in Fig. 2, experimental data for ⁹³Nb are well reproduced with this code system. It should be noted that these benchmark results including the pre-equilibrium part are computed exclusively from the optical model, without adjustment of special pre-equilibrium parameters such as the squared matrix element required in the exciton model. In the high-energy tail observed in emission spectra induced by 14 MeV neutrons the calculated results are still within the very large error bars of the data. The role of collective excited states in this part of the emission spectrum requires further study.

In the neutron multiplication experiment on Pb by Takahashi et al. [13] more neutrons were observed than predicted with ENDF/B-V data. Another experiment by Aleksandrov et al. [14] leads to the same conclusion. These observations are in conflict with the (n,2n) cross section measurement by Frehaut et al. [15]. A critical analysis of the Takahashi results reveals that they imply 13 % of the Pb(n,2n) neutrons to remain above the (n,2n) threshold. This is kinematically highly improbable. Therefore the experiment should be reinvestigated.

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Fig. 1 ⁷Li(n,n't) cross section: comparison of evaluations (curves) and measurements (points).



Fig. 2 Neutron emission spectrum for ⁹³Nb. The curve was calculated with the geometry-dependent hybrid model, based only on optical model quantities (no fit of precompound parameters). The points are measured values.

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2. Nuclear Reaction Theory

2.1 Investigation of Precompound Reaction Models

H. Jahn

(Relevant to request numbers 692100F, 702007R, 714004R, 761075R, 781048F, 832025F, 832042F, 742129R, 792112R, 781222F, 832043F)

The predictive capability of most precompound models that are presently available is limited by too much arbitrariness in the choice of parameters. The origin of this arbitrariness was recently analysed in detail in a review of available precompound descriptions and their performance in the light of experimental data [1]. The main contestants are the exciton model on one hand and the hybrid model on the other.

The exciton model introduced by Cline and Blann [2] is based on the master equations, a system of balance equations for transitions between states with different numbers of particles and holes (excitons). Despite this formally appealing approach it is beset by difficulties concerning e. g. the pairing corrections to particle-hole as well as compound state densities, and especially by a lack of theoretical understanding of the effective matrix element that governs creation and annihilation of particle-hole pairs. The universality of its assumed mass and energy dependence is in serious doubt. Large excursions are observed from the postulated smooth trend, and attempts to blame them on shell effects were not very successful since nuclei between closed shells are afflicted too. Another ambiguity, about the Pauli correction in the transition state densities, could recently be removed by Anzaldo [3] who used number theory to derive its correct form. Calculations of the transition rates from nucleon-nucleon scattering in nuclear matter gave values which were 4 to 10 times too low. Apparently direct processes are not included at all in the present form of the exciton model. Since its predictive power . is thus severely limited its usefulness is restricted to data fitting and inter- or extrapolation over limited energy ranges.

Blann overcame many of the difficulties of the exciton model by means of an "intermarriage" [4] between the exciton master equation [2] and the

Harp-Miller-Berne equation [5]. The influence of the diffuseness of the nuclear surface turned out to be important and led from the original hybrid model to the geometry-dependent hybrid model. Its optical-model version contains, apart from general nuclear parameters such as the nucleon numbers N and Z, only optical-model quantities, in particular the nuclear radius, the surface diffuseness, the imaginary well depth and the cross section for compound nucleus formation. It is the only existing model that takes the diffuseness of the nuclear surface into account. Furthermore, its 3-exciton $(n_0 = 3)$ component includes the clearly geometry-dependent statistical direct processes as Blann indicated and as its relationship to the Harp-Miller-Berne theory [5] suggests which latter is known to include the statistical direct reactions. Consistent with this we find that the geometry-dependent hybrid model describes, without fit, the ⁵⁶Fe data on double-differential neutron scattering at 7.54 and 14.6 MeV incident energy quite well, and is in good agreement with angle-integrated DWBA results at 14.6 MeV. Similarly good agreement between measured data and calculations with the geometry-dependent hybrid model was found for ⁵⁵Mn and ⁹³Nb.

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2.2 Statistical Distributions of R-matrix Elements and Cross Sections

· F.H. Fröhner

The problem of resonance-averaged partial cross sections (width fluctuation factors) has recently been reinvestigated in the context of the statistical model with information-theoretical methods such as entropy maximisation and analytic ergodicity. For pure scatterers the S-matrix reduces to a simple phase factor, $U = \exp(i\theta)$, and the phases are found to be distributed according to

$$p(\theta)d\theta = \operatorname{Re}\left(\frac{e^{i\theta}+\langle U\rangle}{e^{i\theta}-\langle U\rangle}\right)\frac{d\theta}{2\pi}, -\pi < \theta \leq \pi$$

where the expectation value $\langle U \rangle$ is the resonance-averaged (optical-model) S-function. This result, given in less compact form by Lopez et al. [1], follows directly from the fact that causality requires all S-matrix poles to lie below the real axis in the complex energy plane. Using the relationship between S- and R-matrix and expressing $\langle U \rangle$ in terms of the pole strength function s and the distant-level parameter \mathbb{R}^{∞} one finds that the R-function values obey a Cauchy distribution around the distant-level parameter, with a width proportional to the strength function,

$$p(R)dR = \frac{sdR}{(R-R^{\infty})^2 + \pi^2 s^2}, \quad -\infty < R < \infty$$

We note that the formal dependence on the R-function parameters (resonance energies and deduced widths) and their frequency distributions (Gaussian orthogonal ensemble distribution and Porter-Thomas distribution, respectively) are the same for the R-function of the single-channel case as for the diagonal elements R_{cc} of the general multi-channel case. Thus the R_{cc} obey analogous Cauchy distributions, with the appropriate strength functions s_c and distant-level parameters R_c . The cross section distribution for a pure scatterer is also readily derived as

$$p(\sigma)d\sigma = \frac{1}{\pi \sin \theta} \operatorname{Re} \left(\frac{e^{i\theta} + \langle U \rangle}{e^{i\theta} - \langle U \rangle} + \frac{e^{i\theta} + \langle U \rangle}{e^{i\theta} - \langle U \rangle} \right) \frac{d\sigma}{4\pi \lambda^2}, \quad 0 \leq \sigma \leq 4\pi \lambda^2$$

with

θ

=
$$\left| \arccos\left(1 - \frac{\sigma}{2\pi \lambda^2} \right) \right|$$
,

and
$$\langle U \rangle = e^{-2i\phi} \frac{1 - (R^{\circ} + i\pi s)L^{\circ*}}{1 - (R^{\circ} + i\pi s)L^{\circ}}$$

This result should be useful for applications such as sample thickness corrections to resonance-averaged transmission data and the preparation of probability tables for Monte Carlo or multi-band self-shielding calculations, as long as elastic scattering is the dominant reaction. Fig. 3 shows s-wave cross section distributions for ⁵⁶Fe below the first inelastic threshold calculated in this way.

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Fig. 3 Frequency distribution of the total neutron cross section for ⁵⁶Fe at two energies below the first inelastic threshold. The peaks correspond to the potential-scattering cross section.

INSTITUT FÜR CHEMIE (1): NUKLEARCHEMIE KERNFORSCHUNGSANLAGE JÜLICH

1. Neutron Data

1.1 Study of (n,t) Reactions

S.M. Qaim, G. Stöcklin, R. Wölfle

In continuation of our radiochemical studies on fast-neutron induced trinucleon emission reactions [cf. 1] we completed our measurements on (n,t) reactions induced by 30 MeV d(Be) break-up neutrons [2]. The neutron spectrum was characterized by the multiple foil activation technique using various threshold reactions; for unfolding the iterative code SAND II was used. Tritium was separated from each irradiated sample by vacuum extraction and counted in the gas phase using a low-level $\hat{\beta}$ detector. The measured cross sections are given in Fig. 1 as a function of Z of the target element. The trend is similar to that with a 53 MeV d(Be) break-up neutron spectrum reported previously from our laboratory. Apart from the initial decrease, the cross section is almost constant over the entire range of Z = 22 to 83. This observation suggests that triton emission from medium and heavy mass nuclei (A = 48 to 209) occurs via surface reactions. In order to shed some light on reaction mechanism, HauserFeshbach calculations were performed. The calculated data are also shown as a trend in Fig. 1. The smooth curve drawn through the experimental points and the theory agree within a factor of 2 in the region of Z = 13 to 20. It appears that the (n,t) cross section of nuclei in the (2s,1d) shell is described by the statistical model. For heavier nuclei, however, non-





statistical processes appear to be more important.

1.2 Cross-Section Measurements of Neutron Threshold Reactions in the Energy Region of 5 to 10 MeV S.M. Qaim, M.M. Rahman, R. Wölfle (Relevant to request identification numbers: 692159 R, 702010 R, 724055 F, 722148 F, 722149 F, 724056 F, 732032 F, 732044 R, 752244 F, 762107 F, 762108 F,

762130 F, 762242 F, 792209 R, 792110 R, 792210 R)

A deuterium gas target was used to produce quasi-monoenergetic neutrons via the reaction 2 H(d,n) 3 He at our compact cyclotron. By varying the energy of the incident deuteron beam between 3 and 7 MeV, neutrons of energy between 5 and 10 MeV are obtained in the 0[°] direction. Some of the characteristics of this neutron source were investigated. The neutron spectrum was characterized by the multiple foil activation technique in combination with the iterative code SAND II, and the results were qualitatively similar to those from time of flight measurements. Using this neutron source crosssection measurements on some isotopes of nickel, mentioned in last year's report, were completed. Use was made of the activation technique, wherever necessary radiochemical separations, and X- or Y-ray counting. The results for (n, α) reactions are given in Fig. 2 [cf. 3]. Cross sections were also measured for (n,p) reactions on 61 Ni and 62 Ni.



Fig. 2 Excitation functions of (n,α) reactions on ⁵⁸Ni, ⁶²Ni and ⁶⁴Ni. The broken lines give experimental trends and the solid lines KEDAK values [3].

Molybdenum is a potential constituent of the first wall of a fusion reactor. The cross-section data base for this element was, however, found to be rather weak. We therefore started measurements on ${}^{92,95,96,97,98}_{Mo(n,p)}$, ${}^{92,98}_{Mo(n,\alpha)}$ and ${}^{100}_{Mo(n,2n)}{}^{99}_{Mo}$ reactions. Statistical model analysis of the data is in progress.

2. Charged Particle Data for Radioisotope Production

Z. Kovacs, S.M. Qaim, G. Stöcklin

In continuation of our studies [cf. 4-6] on the production of medically important short-lived radioisotopes, excitation functions of some nuclear reactions relevant to the production of 117m Sn and 75 Br were measured.

As discussed in last year's report, 117m Sn is a potentially important radioisotope for single photon emission computed tomography (SPECT). Studies on the four potentially useful reactions, viz. $^{nat}Cd(\alpha,xn)^{117}$ Sn, $^{nat}Cd(^{3}_{He},xn)^{117m}$ Sn, $^{nat}In(\alpha,pxn)^{117m}$ Sn and $^{nat}In(^{3}_{He},pxn)^{117m}$ Sn, were completed [7]. The data for the α -particle induced reactions on indium are shown in Fig. 3.

From the excitation functions and thick target yields the ${}^{116}Cd(\alpha, 3n){}^{117m}Sn$ and ${}^{115}In(\alpha, pn){}^{117m}Sn$ reactions appear to be most promising for the production of ${}^{117m}Sn$.

 75 Br(T_{1/2} = 1.6 h) is a positron emitter and has found application in positron emission computed tomography (PECT). The various nuclear processes used for its production have been recently reviewed [6]. The two major production methods make use of the 75 As(3 He,3n) 75 Br and



Fig. 3 Excitation functions for the formation of 113 Sn and 117m Sn in the interactions of α -particles on natural indium [7].

 76 Se(p,2n) 75 Br reactions. The cross-section data of the latter reaction, reported by the Groningen group, needed re-investigation. We made thin samples by electrodeposition of 96.5 % enriched 76 Se on Al and measured the excitation functions of the (p,n) and (p,2n) reactions up to 40 MeV by the stacked-foil technique. A more detailed analysis of the data is in progress.

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Fast-Chopper Time-of-Flight Spectrometer

H.-G. Priesmeyer, B. Asmussen, P. Fischer, U. Harz, P. Henkens

1. Transmission Measurements using the 24 keV-Ironfilter

The ironfilter - described in the last report - has been used to measure the total cross section of elemental lead, cadmium and gold. We find the following values in good agreement with BNL 325 data:

For the measurements the titanium filter-difference method was used. The high-energy neutron contamination, which is now assumed to be about 2 % of the total neutron intensity, will be determined more precisely by a high-resolution transmission measurement of the filter itself to be performed in the near future.

An experiment to determine the n,p total cross section on hydrogen gas for calibration purposes is in progress.

2. Resonance Parameters of the 5.4 eV Level in Dy-162

Request 82047 of the NNDC Compilation of Request for Nuclear Data is for better parameters of the 5.4 eV Dy-162 resonance, which is well in the range of the Kiel Fast-Chopper facility. Therefore very thin samples have been prepared to be able to investigate this rather intensive level. A 20 μ thick metallic natural dysprosium film evaporated on aluminium has been used to perform a measurement at 77 K in order to study the resonance shape with reduced Doppler broadening. The preliminary results achieved for an n = 0.000101 at/b-sample using natural Dy₂O₃, dissolved in 10 ml HNO₃ and diluted in 100 ml D₂O, are the following:

 $E_{o} \approx 5.45 \text{ eV}$ $\Gamma_{n}^{O} \approx 12 \text{ meV}$ $\Gamma_{n}^{P} = 112 \text{ meV}$

There is no indication of a resonance doublet, as can be seen from Fig. 1. Final resonance paremeters will be presented after a concluding experiment on a sample containing Dy-162 with 92.39 % enrichment.

3. Resolution Function and Transmission of the Kiel Fast-Chopper Spectrometer

In order to improve the results of resonance shape analysis of the chopper measurements, an effort has been undertaken to determine the resolution function of the spectrometer to a higher accuracy. Completed up to now is a Monte-Carlo calculation in two dimensions for a "black" and a "grey" rotor/collimator system. The neutron transmission, i.e. especially critical and cut-off energies, is also calculated. Experimental verification of the theoretical results is underway.

4. Publications

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INSTITUT FÜR PHYSIK

GKSS-FORSCHUNGSZENTRUM GEESTHACHT

1. Neutron Generator Facility KORONA

R. Pepelnik, B. Anders, H.-U. Fanger, W. Michaelis

1.1 KORONA - a Neutron Generator combined with a Fast Rabbit System

At the GKSS Research Center Geesthacht a new intense 14 MeV neutron generator is in operation since 1981. The main components of the facility, named KORONA, are a sealed neutron tube and an integrated fast pneumatic rabbit system [1]. The neutron tube with a cylindrical acceleration structure was developed by Schmidt [2] for radiotherapy purposes. In order to meet the requirements for 14 MeV neutron activation analysis the interior space of the closed-end neutron tube had to be made accessible for a fast sample transfer system. A neutron flux of more than 3 x 10^{10} n/cm²s can be produced via the T(d,n)-reaction in the center of the cylindrical neutron generator target.

Solid as well as liquid samples can be irradiated in polyethylene containers with volumes of 0.55 ccm. The activated samples are pneumatically transferred to a 16 m distant detector station within 140 ms. For the investigation of short-lived radioisotopes cyclic activation is feasible.

The γ -ray spectroscopy system consists of a 93 ccm Ge(Li)detector, a Canberra-2001 preamplifier, a Canberra-2020 main amplifier, a Laben-8215 ADC and a Nuclear Data-66 MCA. Due to the high source strength of the generator, high initial activities of short-lived isotopes are induced with more than 10^6 Bq, typically. At maximum rates of 8 x 10^5 cps, counting losses of 99 % occur. Therefore, the γ -ray spectroscopy system has been improved considerably with regard to the capability of processing high counting rates, particularly by means of a DC-level controlled charge restoration for the preamplifier and a novel real-time correction of counting losses [3,4].

1.2 Neutron Energy and Neutron Flux Distribution

Several experimental and theoretical efforts have been made to determine the important neutron flux and neutron-energy distribution. The contribution of neutrons scattered in the structural material of the cylindrical target, the

effects of scattering in the rabbit guide, rabbit and sample, and the influence of the neutron spectrum on the average activation cross section were considered in a thorough theoretical study [5,6]. Experimentally, the spectrum was investigated using two different techniques: a) the activity-ratio method by means of the (n,2n)-reactions on Zr and Nb as well as on Zr and U, and b) the reaction-threshold technique with 22 different element samples [7,8]. The Zr/U method yielded an average energy of 14.7 ± 0.1 MeV, whereas the other methods gave slightly lower values. A recent study [9] using the experimental results showed that the averaged neutron spectrum has a median at 14.7 MeV with a FWHM of 600 keV. The amount of thermal neutrons was determined by several (n,γ) -reactions to be less than 5 x 10⁻³ of the total neutron flux.

The theoretically calculated neutron flux distribution was compared with activation measurements using small-sized wires at different positions inside the cylindrical target. The results confirm the rather constant neutron flux within the irradiation volume. The variation was determined to be within ± 5 % [10]. As a surprising result of this comparison it turned out that the divergence of the ion beam along the cylinder axis is much smaller than expected previously.

1.3 Reaction Cross-Section Measurements

The availability of the new high-intense 14 MeV neutron source has stimulated a considerable effort to measure unknown and to redetermine uncertain or wellknown, but strongly energy-dependent cross-sections. Table I summarizes the results of the investigations [7,11,12,13,14] performed. The cross-section measurements have so far been concentrated on reactions with short-lived product nuclei, which are of particular interest in many applications, e.g. for activation-analysis work. In this connection (n,n')-reactions also come to the fore. Where comparable cross-section data are available, the agreement with other studies [see Ref. 11] is quite good. Deviations in the case of strongly energy-dependent reactions are not surprising.

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Table I Cross Sections at 14.7 ± 0.	3 MeV
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Reaction	σ (mb)	Ref.	Reaction	σ (mb)	Ref.
⁵⁰ Ti(n,p) ⁵⁰ Sc	14.3 ± 2.1	[12]	⁶¹ Ni(n,p) ⁶¹ Co /	84 + 4*	[13]
⁵⁰ Cr(n,2n) ⁴⁹ Cr	27.2 ± 1.9	[13]	⁶² Ni(n,np) ⁶¹ Co	04 1 4"	[10]
52 Cr(n,p) 52 V	95 7 + 2 6*	[12]	⁶² Ni(n,p) ^{62g} Co	24.8 ± 1.2	[13]
⁵³ Cr(n,np) ⁵² V	05.7 - 2.0	[13]	⁶² Ni(n,p) ^{62m} Co	14.6 ± 0.9	[13]
⁵² Cr(n, 2n) ⁵¹ Cr(356 + 24*	[13]	⁶⁴ Ni(n,np) ⁶³ Co	3.6 ± 0.7	[13]
50 Cr(n, γ) 51 Cr \int	550 - 24	[13]	64 Ni(n, α) ⁶¹ Fe	3.7 ± 0.2	[13]
⁵³ Cr(n,p) ⁵³ V	47.2 ± 1.7	[13]	⁶⁸ Zn(n,p) ^{68g} Cu	5.0 ± 1.1	[14]
$^{54}Cr(n,\alpha)^{51}Ti$ (126+05*	[13]	⁶⁸ Zn(n,p) ^{68m} Cu	3.6 ± 0.6	[14]
$5_{Cr(n, ^{3}He)}^{5_{Ti}}$	12.0 2 0.3	[13]	⁶⁸ Zn(n,α) ⁶⁵ Ni	10.3 ± 1.8	[14]
⁵⁴ Cr(n,p) ⁵⁴ V	16.4 ± 0.5	[13]	⁷⁹ Br(n,n') ^{79m} Br	294 ± 16	[7]
⁵⁵ Mn(n,α) ⁵² V	23.2 ± 0.7	[13]	⁹⁰ Zr(n,p) ⁹⁰ my	9.8 ± 1.7	[14]
⁵⁵ Mn(n,2n) ⁵⁴ Mn	741 ± 22	[13]	⁹⁰ Zr(n,2n) ^{89m} Zr	75 ± 12	[14]
54 Fe(n,p) 54 Mn	307 ± 9	[13]	⁹⁰ Zr(n,a) ^{87m} Sr	3.2 ± 0.5	[14]
⁵⁴ Fe(n,2n) ⁵³ Fe	7.9 ± 0.7	[13]	¹⁶⁷ Er(n,n') ¹⁶⁷ mEr	252 ± 18	[7]
54 Fe (n, α) 51 Cr	88 ± 6	[13]	¹⁶⁸ Er(n,2n) ^{167m} Er	581 ± 43	[7]
⁵⁶ Fe(n,p) ⁵⁶ Mn /	111.0 + 5.5*	[13]	¹⁶⁸ Er(n,2n) ^{167m} Er	(795 + 59*	[7]
⁵⁷ Fe(n,np) ⁵⁶ Mn∫		[10]	¹⁶⁷ Er(n,n') ^{167m} Er		[/]
⁵⁷ Fe(n,p) ⁵⁷ Mn	89 ± 5	[13]	¹⁷⁴ Yb(n,p) ¹⁷⁴ Tm	3.0 ± 0.2	[7]
⁵⁸ Fe(n,p) ⁵⁸ Mn	13.6 ± 0.7	[13]	¹⁷⁶ Yb(n,n') ^{176m} Yb	19.7 ± 1.7	[7]
⁵⁹ Co(n,p) ⁵⁹ Fe	46.5 ± 2.3	[13]	¹⁸³ W(n,n') ^{183m} W	127 ± 14	[7]
⁵⁹ Co (n, 2n) ⁵⁸ Co	231 ± 10	[13]	¹⁸⁴ W(n,2n) ^{183m} W	656 ± 74	[7]
⁵⁹ Co(n,2n) ^{58m} Co	478 ± 24	[13]	¹⁸⁴ W(n,2n) ^{183m} W (715 + 81*	[7]
⁵⁹ Co(n,α) ⁵⁶ Mn	30.2 ± 1.5	[13]	¹⁸³ W(n,n') ^{183m} W \		.,,
⁵⁸ Ni(n,p) ^{58g} Co	150.5 ± 6.0	[13]	¹⁹⁰ Os(n,n') ^{190m} Os	14.0 ± 1.1	[7]
⁵⁸ Ni (n,p) ^{58m} Co	169 ± 10	[13]	¹⁹² Os(n,n') ¹⁹² mOs	2.6 ± 0.3	[7]
⁵⁸ Ni(n,np) ⁵⁷ Co	586 ± 30	[13]	¹⁹¹ Ir(n,n') ^{191m} Ir	221 ± 22	[7]
⁵⁸ Ni(n,2n) ⁵⁷ Ni	34.7 ± 1.7	[13]	²⁰⁸ Pb(n,2n) ^{207m} Pb	1365 ± 68*	[7]
⁶⁰ Ni(n,p) ⁶⁰ Co	131 ± 4**	[13]	²⁰⁷ Pb(n,n') ^{207m} Pb		
		1			

* The value is calculated taking the abundance of the first mentioned isotope.
 ** The value refers to the sum of groundstate and isomeric state cross-sections.

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INSTITUT FÜR KERNCHEMIE UNIVERSITÄT ZU KÖLN

1. <u>Measurement and Hybrid Model Analysis of Integral</u> <u>Excitation Functions for p-induced Reactions up to 200 MeV</u>

R. Michel, F. Peiffer, R. Stück

1.1 Proton-induced Reactions on V, Mn and Co

By irradiation experiments at the "Cyclon" isochronous cyclotron at Louvain La Neuve and the synchrocyclotron of the IPN Orsay we extended our earlier measurements of integral excitation functions [1,2] from 45 MeV to 200 MeV [cf.3].

The cross sections were determined using the stacked foil technique. Three types of stacks covered the energy ranges from 45 to 80 MeV, from 77 to 160 MeV and from 150 to 200 MeV. The p-fluxes were determined by the ²²Na activity induced in Al-foils distributed all over the stacks. As monitor cross sections for the reaction 27 Al(p,3p3n) 22 Na the "valeur adopte" of Tobailem and de Lassus St. Genies [4] were taken. The nuclear data used for the calculation of cross sections were the same as in [1,2 and 3]. Since the energy ranges of the different experiments are overlapping, also with our earlier measurements below 45 MeV [1], the quality of the absolute determination of the cross sections was checked with high sensitivity with regard to flux determination errors. Moreover the negligibility of secondary particles was proved by applying the method to reactions with low threshold energies.

So we now have a comprehensive, self-consistent set of excitation functions up to 200 MeV. These new cross sections allow for a much more detailed analysis with regard to nuclear reaction theories. By a coincidence an improved version of the computer code "ALICE LIVER-MORE 82" [5] was at our disposal (courtesy of M. Blann), which now includes the Wapstra and Gove mass tables [6] and which, moreover, in contrast to the earlier version of the hybrid model of preequilibrium reactions [7,8] now includes multiple preequilibrium emission. Further it allows to take into account broken exciton numbers according to [9].

According to M. Blann [10] this version should be capable to perform <u>a priori</u> calculations up to 200 MeV. Since the target elements Mn and Co are, and V can be regarded, as single isotope elements, the comparsion of theoretical and experimental cross sections is of particular interest here. It turns out that, for some reactions and generally for higher energies, severe discrepancies are to be observed in <u>a priori</u> calculations. Therefore a set of optimal parameters was searched for. But this search did not result in an unambiguous choice of parameters [11]. While generally (p,xn)reactions are fairly well represented by the theory [1,12],



Fig. 1 Experimental cross sections and theoretical excitation functions for the reaction ⁵⁹Co(p,pxn) ⁵⁸Co. For references of other authors see [3], for the work of other authors below 45 MeV see [1].

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Fig.2 Experimental cross sections and theoretical excitation functions for the reaction V(p,pxn)⁴⁸V. For references of other authors see [3], for the work of other authors below 45 MeV see [1].



Fig.3 Experimental cross sections and theoretical excitation functions for the reaction $55 Mn(p, 5p5n)^{46}Sc.$

for (p,pn)-reactions strong discrepancies between theory and experiment were seen, pointing to important contributions of direct reactions at high energies (fig.1). These discrepancies vanish, however, for (p,pxn)- reactions with increasing x (fig.2) and also for some reactions with higher threshold energies (fig.3). Generally, the observed discrepancies are not so extreme as observed for α - and ³He-induced reactions [13,14]. The capability of <u>a priori</u> calculations is, however, relatively limited, since there exist still too many reactions exhibiting severe discrepancies which have to be studied in more detail [11].

1.2 Proton-induced Reactions on Ti, Fe and Ni between 45 and 80 MeV

For the cosmochemically important target elements Ti, Fe and Ni the gap between our earlier measurements below 45 MeV [1,15,16] and those above 80 MeV [18] was closed in the experiments at Louvain La Neuve. Tables 1-3 give these results for these target elements. The data fit extremely well with those of the earlier determination and do not change any statement of our earlier discussion of these excitation functions with regard to the theories of nuclear reactions [17]. The determination of excitation functions relevant to cosmochemistry is now continued by measurements at 600 MeV.

Table I Cross sections (mb) of proton induced reactions on natural titanium between 50 and 80 MeV

E_[MeV]	48 _{SC}	47 _{Sc}	46m+g _{Sc}	44m _{SC}	47 _{Ca}	43 _K	42 _K
79.47	1.92	22.7	49.5	23.0	0.052	1.34	5.7
<u>+</u> 0.34	±0.17	<u>+</u> 3.0	<u>+</u> 6.9	<u>+</u> 2.3	+0.012	<u>+</u> 0.19	<u>+</u> 1.0
74.93	1.94	21.3	47.0	21.9	0.050	1.22	5.6
<u>+</u> 0.44	<u>+</u> 0.19	<u>+</u> 2.8	<u>+</u> 6.6	<u>+</u> 2.0	<u>+</u> 0.012	<u>+</u> 0.17	±0.9
70.27	2.08	22.8	50.4	22.7	0.057	1.25	6.2
+ 0.51	<u>+</u> 0.21	<u>+</u> 3.0	<u>+</u> 7.1	± 2.3	<u>+</u> 0.012	<u>+</u> 0.19	<u>+</u> 1.2
65.26	2.10	22.5	51.2	20.4	0.044	1.20	6.0
<u>+</u> 0.57	<u>+</u> 0.23	+ 2.9	<u>+</u> 7.1	± 1.8	±0.010	<u>+</u> 0.16	±1.0
60.09	2.19	23.7	55.2	17.4	0.048	1.32	5.29
<u>+</u> 0.63	+0.22	± 3.1	± 7.7	<u>+</u> 1.7	<u>+</u> 0.014	<u>+</u> 0.20	<u>+</u> 0,85
54.43	2.22	24.0	59.9	14.9	0.041	1.44	3.51
± 0.68	<u>+</u> 0.20	<u>+</u> 3.1	<u>+</u> 7.8	± 1.5	<u>+</u> 0.012	<u>+</u> 0.19	±0.56
50.32	2.01	21.5	59.5	12.4		1.26	1.43
± 0.72	<u>+</u> 0.18	± 2.8	<u>+</u> 7.7	<u>+</u> 1.1		<u>+</u> 0.16	+0.29

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Table II Cross sections (mb) of p-induced reactions on natural iron between 79 and 49 MeV

E [MeV]	54 _{Mn}	⁵² Mn	⁵¹ Cr	⁴⁸ Cr	48 _V	47 _{SC}	46m+g _{Sc}	^{44m} Sc
78.25	77.3	32.5	65.5	0.319	6.16	0.249	1.00	0.239
+ 0.39	<u>+</u> 6.2	<u>+</u> 2.9	<u>+</u> 5.2	<u>+</u> 0.048	<u>+</u> 0.60	<u>+</u> 0.045	<u>+</u> 0,12	<u>+</u> 0.033
73.66	83.0	34.5	59.3	0.278	4.62	0.240	0.621	0.228
<u>+</u> 0.49	<u>+</u> 6.6	± 3.1	<u>+</u> 4.7	+0.042	<u>+</u> 0.41	<u>+</u> 0.043	<u>+</u> 0.087	<u>+</u> 0.032
68.94	85.3	33.2	52.5	0.289	4.00	0.222	0.231	0.179
<u>+</u> 0.56	<u>+</u> 6.8	<u>+</u> 3.0	4.2	<u>+</u> 0.049	<u>+</u> 0.37	<u>+</u> 0.038	<u>+</u> 0.028	<u>+</u> 0.034
63.85	88.6	27.4	51.9	0.295	4.28	0.152	0.051	0.089
<u>+</u> 0.63	<u>+</u> 7.1	<u>+</u> 2.5	<u>+</u> 4.2	<u>+</u> 0.050	<u>+</u> 0.39	<u>+</u> 0.026	<u>+</u> 0.011	<u>+</u> 0.020
58.59 + 0.68	98.1 <u>+</u> 7.8	19.8 <u>+</u> 1.8	60.7 <u>+</u> 4.9	0,251 <u>+</u> 0,043	5.10 <u>+</u> 0.45	0.063 <u>+</u> 0.014		
53.46 <u>+</u> 0.74	114.B <u>+</u> 9.2	14.9 <u>+</u> 1.5	76.6 <u>+</u> 6.1	0,160 <u>+</u> 0,030	5.64 <u>+</u> 0.48	0.031 <u>+</u> 0.007		
49.30 <u>+</u> 0.78	122.8 <u>+</u> 9.8	12.2 <u>+</u> 1.1	86.3 <u>+</u> 6.9	0.053 <u>+</u> 0.017	4.71 <u>+</u> 0.39			

Table III Cross sections (mb) of p-induced reactions on natural nickel between 78 and 48 MeV

Ep[MeV]	57 _{Ni}	58m+g _{Co}	57 _{Co}	⁵⁶ co	55 _{Co}	⁵⁴ Mn	52 _{Mn}	⁵¹ cr	48 _V	46m+g _{Sc}
77.53 <u>+</u> 0.43	2.61 ±0.29			37.9 ± 4.9	5,69 +0,74	36.5 <u>+</u> 4.3	13.2 <u>+</u> 1.2	16.6 <u>+</u> 1.3	0.44 <u>+</u> 0.16	0.139 <u>+</u> 0.026
72.89 <u>+</u> 0.55	2.80 ±0.34	145 <u>+</u> 13	111 <u>+</u> 17	38.7 <u>+</u> 3.8	6、28 <u>+</u> 0.82	30.2 <u>+</u> 2.7	14.2 <u>+</u> 1.3	10.03 <u>+</u> 0.90	0.147 <u>+</u> 0.047	0.076 <u>+</u> 0.017
68.13 <u>+</u> 0.62	3.06 ±0.34	151 <u>+</u> 14	114 <u>+</u> 18	43.0 <u>+</u> 4.7	6.63 <u>+</u> 0.86	25.9 <u>+</u> 2.3	12.9 <u>+</u> 1.2	5.06 <u>+</u> 0.56	0.027 <u>+</u> 0.010	
63.01 ± 0.67	3.18 <u>+</u> 0,38	149 <u>+</u> 14	110 <u>+</u> 18	46.9 <u>+</u> 4.7	5.44 +0.71	22.4 <u>+</u> 2.0	8.01 <u>+</u> 0.72	3.29 ± 0.40		
57.70 <u>+</u> 0.73	3.94 <u>+</u> 0.39	168 <u>+</u> 15	126 <u>+</u> 18	60.0 ± 6.0	3.98 <u>+</u> 0.52	26.8 + 2.1	3.94 <u>+</u> 0.35	4.67 <u>+</u> 0.51		
52.81 ± 0.81	4.77 <u>+</u> 0.48	176 <u>+</u> 16	142 <u>+</u> 20	69.4 ± 6.3	2.15 <u>+</u> 0.30	33.0 <u>+</u> 2.6	1.83 <u>+</u> 0.20	6.23 ± 0.68		
48.61 <u>+</u> 0.84	5.87 <u>+</u> 0.59	175 <u>+</u> 16	154 <u>+</u> 22	63.5 <u>+</u> 6.4	0.87 <u>+</u> 0.19	40.0 <u>+</u> 3.6	0.66 <u>+</u> 0.14	6.94 <u>+</u> 0.76		

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INSTITUT FÜR KERNCHEMIE

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Nuclear Charge Distribution of Heavy Mass Fission Products in ²³⁵U(n_{th}, f)

H.O. Denschlag, Z.B. Alfassi^{*}, H. Braun, W. Ditz, W. Faubel, H. Faust^{**}, St. Hörner, W. Pörsch, R. Sehr, H. Schrader^{**}, and B. Sohnius

Fission products were separated according to mass, ionic charge state, and kinetic energy using the mass separator LOHENGRIN of the Institut Laue-Langevin (Grenoble). The yields of the single members within mass chains 130 to 147 were obtained by measuring the gamma rays emitted from the short-lived fission products immediately following their beta-decay. Absolute gamma-ray line intensities and branching ratios - when not known - were determined by separate radiochemical experiments [1,2,3].

The mass separator LOHENGRIN provides fission products of a well defined kinetic energy and of the ionic charge state realized while in transit through the separator. This feature is advantageous for a fundamental investigation of e.g. the correlation between specific scission configurations and fission yields and for the study of the

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dynamical ionization behaviour of heavy ions and related effects. For practical purposes and for a comparison with radiochemical yields, in principle, the yields at the various kinetic energies of the fragments and their various ionic charge states ought to be averaged. Such an analysis will be attempted in the future.

At present, in Table I we are presenting yields obtained for the mean kinetic energy and for the mean ionic charge state. These yields can be expected to generally agree with the averaged values except for chain 142 where the fission yields observed vary in a particularly strong way with the ionic charge state chosen. In this case, a summation over the charge states has been performed and the following yield values (relative to a "fractional cumulative yield" (YFC) of ¹⁴²Ba of 100%) have been obtained:

> YFC $({}^{142}Cs) = (37 +/- 4) \%$ YFI $({}^{142}Ba) = (63 +/- 7) \%$

These yields and the yields within the other mass chains (130 to 137 and 139 to 147)^{*} given in Table I are in general agreement with the radiochemical yield values [4].

The total results including values obtained at other kinetic energies of the fragments and at other ionic charge states for the respective mass chains may be found in the references given in the table for the respective mass chains.

* Mass chain 138 [6] has not been finally evaluated yet.

Table I Fractional independent (first chain members: -cumulative) yields in 235 U(n_{th}, f) measured at the mean kinetic energy of the fission fragments and at the mean ionic charge state of the fragments in transit through LOHENGRIN

 Nuclide	Fractional Yield	References	
130 _{Sn} 130 _{Sb} 130 _{Te}	0.69 +/- 0.04 0.28 +/- 0.04 0.03 +/- 0.09	[6] [6] [6]	
131 _{Sn} 131 _{Sb} 131 _{Te}	0.34 +/- 0.02 0.62 +/- 0.02 0.04 +/- 0.02	[6] [6] [6]	
132 _{Sn} 132 _{Sb} 132 _{Te}	0.14 +/- 0.02 0.45 +/- 0.03 0.41 +/- 0.03	[5] [5] [5]	
133 _{Sb} 133 _{Te}	0.44 +/- 0.02 0.56 +/- 0.02	[5, 7] [5, 7]	
134 _{Sb} 134Te 134 _I	0.06 +/- 0.01 0.82 +/- 0.08 0.12 +/- 0.02	[5, 8] [5, 8] [5, 8]	
135 _{Sb} 135 _{Te} 135 _I 135 _{Xe}	0.06 +/- 0.03 0.50 +/- 0.03 0.42 +/- 0.02 0.02 +/- 0.01	[5, 7] [5, 7] [5, 7] [5, 7]	
136 ₁ e 136 ₁ 136 _{Xe}	0.16 +/- 0.04 0.42 +/- 0.08 0.42 +/- 0.15	[5, 8] [5, 8] [5, 8]	
137Te 137I 137Xe	0.08 +/- 0.01 0.43 +/- 0.05 0.49 +/- 0.05	[5] [5] [5]	
139 I 139 Xe 139 Cs 139 Ba	0.19 +/- 0.03 0.74 +/- 0.04 0.08 +/- 0.05 0.02 +/- 0.05	[7] [7] [7] [7]	
140 140xe 140cs 140 _{Cs} 8a	0.04 +/- 0.02 0.72 +/- 0.02 0.22 +/- 0.02 0.03 +/- 0.05	[6] [6] [6] [6]	

Nuclide	Fractional Yield	References
141 141 Xe 141 Cs	0.16 +/- 0.01 0.47 +/- 0.03	[6] [6]
142 _{Cs}	0.37 +/- 0.02	[6]
¹⁴² Ba	$0.78 + / - 0.01^*$	[9]
143 _{Ba} 143 _{La}	0.25 + 7 = 0.04 0.67 + 7 = 0.05 0.08 + 7 = 0.07	[9] [9]
144 _{Cs} 144 _{Ba}	0.02 +/- 0.01 0.77 +/- 0.04	[9] [9]
144 ²⁻ 145-	0.21 +/- 0.04	[9]
145 _{La} 145 _{Ce}	0.45 +/- 0.03 0.55 +/- 0.04 < 0.024	[9] [9] [9]
146 _{8a} 146 _{1a}	0.28 +/- 0.03 0.52 +/- 0.08	[9] [9]
146 <u>Ce</u> 146 _{Pr}	0.20 +/- 0.01 < 0.012	[9] [9]
147 _{La} 147 _{Ce}	0.41 +/- 0.01 0.41 +/- 0.08	[9] [9]
147 _{Pr}	0.18 +/- 0.08	[9]

Table T (continued)

*/ield not directly comparable to radio-chemical yield (see text)

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INSTITUT FÜR STRAHLENPHYSIK UNIVERSITÄT STUTTGART

Investigations of Analyzing Power and Differential Cross Section of Lead-208, Lead-206, Thorium-232, Tantalum, Tungsten and Thulium for $E_{n} = 7.75$ MeV

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Studies with the Stuttgart facility SCORPION for fast polarized neutron scattering experiments for the measuring of analyzing power and differential cross section for heavy mass and medium mass nuclei have been continued. At an energy of 7.75 MeV for the polarized neutrons of 60 % polarization investigations on ²⁰⁸Pb, ²⁰⁶Pb, ²³²Th, Ta, W and Tm have been performed. The accuracy of the data and also of the data analysis could be improved. For ²⁰⁸Pb and Tm only very small samples were available, therefore we had to measure near the sensitivity limit of our set up.

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Fig. 1 and 2 give examples of the data for 206 Pb^{*}.

Fig. 1 Analyzing Power of Lead-206 for $E_{p} = 7.75$ MeV

The analysis of the data, like fi..ite sample corrections and optical model calculations is in progress. Finite sample corrections for the analyzing power evaluations are made using the code JANE by E. Woye. This code will be adapted to correct also differential cross section data.

*The lending of a ²⁰⁶Pb sample by J. Harvey, Oak Ridge, is highly appreciated. Also we were in debt of gratitude to G. Haouat, Bruyères le Châtel, for lending us the ²⁰⁸Pb sample.

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Fig. 2 Differential cross section of Lead-206 for $E_n = 7.75 \text{ MeV}$

The code FANTI for the unfolding of proton recoil data, based on the older codes FERDOR and FORIST, has been proven in many cases, even worse ones, and is now completed. It will be available on request. REAKTORSTATION GARCHING; FACHBEREICH PHYSIK TECHNISCHE UNIVERSITÄT MONCHEN

Coherent Neutron Scattering Lengths and Free Scattering Cross Sections

<u>Gallium and Zinc</u>
 L. Koester, K. Knopf, W. Waschkowski

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Bound coherent neutron scattering lengths b and free cross sections σ_s were measured on natural Ga and Zn and on isotopically enriched samples. We obtained by means of Christiansen filter technique and transmission measurements the following data (to be published):

b(^{nat}Ga) = 7.288 (15) fm,

$$\sigma$$
 s/1.26 eV = 6.994 (11) b,
 σ s/5.19 eV = 6.899 (7) b,
 σ s/18.8 eV = 6.773 (14) b,
 σ s/18.8 eV = 8.23 (6) b,
 σ s/1.26 eV = 8.23 (6) b,
 σ s/5.2 eV = 8.49 (16) b,
b(⁷¹Ga) = 6.36 (3) fm.

From the energy dependence of the scatting cross section of Ga additional data on the resonance parameters could be derived.

 $b(^{nat}Zn) = 5.686 (14) \text{ fm}, \qquad \sigma \text{ s/1.26 eV} = 4.040 (6) \text{ b},$ $b(^{64}Zn) = 5.23 (2) \text{ fm}, \qquad \sigma \text{ s/1.26 eV} = 3.54 (10) \text{ b},$ $b(^{66}Zn) = 5.98 (3) \text{ fm}, \qquad \sigma \text{ s/1.26 eV} = 4.32 (12) \text{ b},$ $b(^{67}Zn) = 7.58 (4) \text{ fm},$ $b(^{68}Zn) = 6.04 (2) \text{ fm}.$

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2. Thorium

L. Koester, K. Knopf, W. Waschkowski

In continuation of our measurements of coherent bound scattering lengths and free scattering cross sections we obtainded the following nuc'ear data for thorium (to be published):

b = 10.30 (4) fm,	σ s/1.26 eV = 12.84 (16) b,
	σ s/5.2 eV = 12.42 (18) b,
	σ s/144 keV = 12.70 (10) b.

These values are in good agreement with the resonance parameters if we accept a bound level at -5.1 eV with the width of $g\sqrt[n]{n} = 2.07 \text{ meV}$ [1].

<u>Tellurium and Antimony</u>
 L. Koester, K. Knopf, W. Waschkowski

By means of Christiansen filter technique and transmission measurements we obtained the following coherent scatting lengths and total cross sections on natural tellurium and natural antimony:

b	(Te)	=	5.786	(14)	fm,	σ	tot/1.26 eV	=	6.68	(2)	b,
						σ	tot/5.2 eV	=	4.65	(2)	b,
b	(Sb)	=	5.54	(3)	fm,	σ	tot/1.26 eV	=	4.09	(2)	b,
						σ	tot/5.2 eV	=	9.17	(5)	b.

Reference:

G. Vasiliu, S. Mateescu, D. Gheorghe, M. Diodaru, E. Badescu, N. Dragon,
 O. Bujoreanu, C. Cracium, L. Pintiliescu, M. Zaharcu, D. Popescu,
 P. Statnicov, V. Arrigeanu: INDC (RUM) - 10 (1980)

INSTITUT FÜR KERNCHEMIE

PHILIPPS-UNIVERSITAT MARBURG

1. Gamma-Ray Catalog

U. Reus, W. Westmeier

Quantitative information on gamma rays from the decay of radioactive nuclides is required in many areas of nuclear science as well as related fields. We have therefore produced a compilation of decay properties of all known radionuclides, with the main emphasis on energies and absolute intensities of gamma rays. A first printed version of this catalog was issued in 1979, covering references to the literature through June 1978.

Revision of data for a second edition, including references through June 1982, was completed in 1983. The updated version contains information on 2526 nuclides and isomers with a total of more than 47,000 gamma rays and X-rays, the information on X-rays accompanying radioactive decay being a newly introduced feature. As before, the catalog is presented in two parts: In <u>Part I</u> gamma rays are listed in order of increasing energy for the purpose of identification of unknown gamma lines. In <u>Part II</u> complete data sets for each nuclide are listed in order of mass number A and nuclear charge Z of the nuclides. This part also contains additional information, references, and comments in case of any discrepancies.

The revised catalog has been published in "Atomic Data and Nuclear Data Tables", Volume 29, 1-406 (1983).

2. Alpha-Energy Table

W. Westmeier, R.A. Esterlund

A table of alpha-decay properties of all known alpha-emitting nuclides, which includes data on alpha energies, intensities, and the abundance of the alpha branch, is being compiled. The table is ordered by increasing energy and covers data on 534 alpha emitters with a total of 1621 energies at present.

Computer printout copies of the table are available on request.

1. Neutrons from 9Be + d

H. J. Brede, G. Dietze, K. Kudo*

The reaction 9 Be + d is used as an intense neutron source at the facility for the calibration of neutron therapy dosimeters. The purpose of these investigations is an accurate determination of the spectral neutron yield from the target which can be used to calculate the tissue kerma in free air at the reference position of the neutron field ("fluence method").

The neutron yield and the neutron spectrum, emitted from an aircooled 2 mm thick Be target which was bombarded by deuterons from the PTB cyclotron were measured with various deuteron energies from 9.4 MeV to 13.3 MeV. The time-of-flight spectrometer with a flight path of 12 m [1] and an NE 213 scintillation detector (5.07 cm in height, 5.06 cm in diameter) was used. The pulsed beam of the cyclotron and conventional $n-\gamma$ discrimination techniques were applied. The detector efficiency was determined for various bias values using the Monte Carlo code NEFF4 [2].

The neutron spectra were measured in the neutron energy range from 0.4 MeV to 17 MeV. The neutron yield at 0° to the direction of the incident deuterons was obtained for deuteron energies from 9.4 MeV to 13.3 MeV. At $E_d = 12.4$ MeV and 13.3 MeV the neutron yields for angles of emission up to 40° were determined. At a distance of 80 cm from the target, the neutron fluence and the tissue kerma in air in relation to the deuteron energy were also evaluated.

In addition to this, the influence of a small water-cooling system at the target backing and the influence of the collimator system on the neutron yield and on the spectral fluence were investigated.

[°]Electrotechnical Laboratory, Ibaraki, Japan

2. Angular Distribution of the Reaction $D(d,n)^{3}$ He for $3.0 \le E_{\tilde{d}} \le 11$ MeV

H. Klein, R. Böttger, J. Suita*

Discrepancies in the calibration of the neutron detection efficiency of liquid scintillators $\begin{bmatrix} 1 \end{bmatrix}$ gave rise to an investigation of the D(d,n) reaction for deuteron energies from 3.0 MeV up to 11.0 MeV in steps of 0.5 MeV.

In the energy range 6.3 MeV $\leq E_n \leq 14$ MeV a liquid scintillator NE 213 (10.6 cm in diameter and 2.54 cm in length), was carefully calibrated in steps of 0.5 MeV with reference to a proton recoil telescope. The experimentally determined response spectra could be reproduced in absolute scale to within a standard deviation of ± 2 % by the Monte Carlo code NRESP4 [2].

On the basis of this calibration the angular distribution of the neutrons from the reaction $D(d,n)^{3}He$ were measured for emission angles $0 \leq \vartheta \underset{n}^{\text{LAB}} \leq 90^{\circ}$. The experimental yield will be compared with the expected yield as calculated by means of the MC code SINENA [3] on the basis of recently evaluated cross sections [4].

Preliminary results showed significant deviations up to 15 % increasing with the projectile energy and the emission angle. The final analysis is in progress.

*IAEA fellow from IEN, Rio de Janeiro

3. Neutrons in the Energy Range $E_n = 0.5 - 30$ keV from the Reaction ${}^{45}Sc(p,n){}^{45}Ti$

M. Cosack, H. Lesiecki , J.B. Hunt*

For the testing of radiation protection instruments with neutrons in the energy range from about 0.5 to 50 keV only a few neutron sources are available. The 45Sc(p,n)45 reaction seems to be very useful for this purpose, as by bombarding a scandium target with protons with energies of a few keV above the threshold of 290.8 keV, low energetic neutrons can be produced. In order to investigate the structure of the neutron yield a thick Sc-target was bombarded with a pulsed proton beam and the neutron energies were analyzed by means of time-offlight technique with a lithium glass scintillator. At the emission angle of 0° strong resonances at 8.2; 14.5; 16.8; 27.5; 33.6 and 36.7 keV were found with only a small neutron yield in between. The natural line width of these resonances was in most cases less than 0.5 keV. Due to the kinematics of the reaction, the energy of each resonance decreases when the emission angle increases. The yields and energies of the most prominent resonances were investigated for different emission angles. Especially the resonance of 8.2 keV at 0° emission produces neutrons of about 0.5 keV at 120° emission. The 45 Sc(p,n) was also used to produce monoenergetic neutrons, taking a thin Sctarget and bombarding it with protons whose energy was adjusted in order to produce neutrons of one of the resonances. For the 8.2 keV resonance and a proton beam of 100 μ A on a thin target a neutron flux density of $2 \cdot 10^5$ sr⁻¹ s⁻¹ was determined with a De Panger long counter. For the 27.5 keV resonance the flux density was $1.8 \cdot 10^5$ sr⁻¹ s⁻¹. These flux densities were found to be sufficient to calibrate radiation protection survey meters.

*NPL, Teddington, G.B.

References

- R. Böttger, H.J. Brede, M. Cosack, G. Dietze, R. Jahr,
 H. Klein, H. Schölermann and B.R.L. Siebert
 Proc. of the Int. Conf. on Nuclear Data for Science
 and Technology, Antwerp, 1982, ed. K.H. Böckhoff,
 Reidel Publ. Comp. Dordrecht (1983) p. 836
- [2] G. Dietze and H. Klein
 PTB-report ND-22, Braunschweig, Oct. 1982
- [3] B.R.L. Siebert, H.J. Brede and H. Lesiecki PTB-report ND-23, Braunschweig, Nov. 1982
- [4] M. Drosg Nucl. Sci. Eng. 67 (1978) 190

FACHINFORMATIONSZENTRUM ENERGIE PHYSIK MATHEMATIK

Status Report

H. Behrens, G. Ebel, H.W. Müller

1. Cooperation between the Fachinformationszentrum and Chemical Abstracts Service

The American Chemical Abstracts Service (CAS) and the Fachinformationszentrum are joining their online computer services. The computers of CAS in Columbus and of FIZ at Karlsruhe will be linked and the same computer software will be used in both facilities. The new network established is named "STN International". The bibliographic and numerical data bases implemented at both locations can be accessed by the user via both nodes of the network.

2. New Data compilations

The following new issues in the series Physics Data were published during the period of this report:

4-3 (1983) Compilation of Coupling Constants and Low-Energy Parameters.
1983 Edition.
U. Dumbrajs, R. Koch, H. Pilkuhn, G.C. Oades, H. Behrens,
J.J. de Swart, P. Kroll

- 5-22 (1983) Gases and Carbon in Metals (Thermodynamics, Kinetics and Properties).
 Part XXII: Group Ib Metals (2): Silver, Gold.
 H. Jehn W. Hehn, E. Fromm, G. Hörz
- 20-1 (1984) Experimental Values of Critical Expoenents and Amplitude Ratios at Magnetic Phase Transitions. K. Stierstadt, R. Anders, W. von Hörsten.

27-1 (1983) International Directory of Certified Radioactive Sources.
 G. Grosse, W. Bambynek

3. Bibliography of Existing Data Compilations

The bibliographic database "Physcomp" which covers data compilations in physics on a worldwide basis has been updated. A printed version will be published in the Physics Data series.

4. The Evaluated Nuclear Structure Data File (ENSDF)

The mass chain A = 98 has passed review procedure and is now in print. The mass chains A = 82, 34, 97 are nearly finished and will enter the review procedure in the next weeks. The work on the mass chains A = 81, 93 and 99 is going on.

The online retrievable databases ENSDF, MEDLIST, and NSR (Nuclear Structure Reference File) have been updated. In ENSDF, 62 mass chains have been replaced by their new versions. In the MEDLIST database, 533 radioactive decay data sets have been updated.

The bibliographic database "Nuclear Structure References" has been updated monthly. In all, app. 3800 new nuclear structure relevant documents have been added to the file.

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APPENDIX I

Addresses of Contributing Laboratories

Institut für Kernphysik II Director: Prof. Dr. A. Citron Senior reporter: Dr. S. Cierjacks Kernforschungszentrum Karlsruhe Postfach 3640 7500 Karlsruhe

Institut für Kernphysik III Director: Prof. Dr. G. Schatz Senior reporter: Dr. F. Käppeler Kernforschungszentrum Karlsruhe Postfach 3640 7500 Karlsruhe

Institut für Neutronenphysik und Reaktortechnik Director: Prof. Dr. G. Kessler Senior reporter: Dr. F.H. Fröhner Kernforschungszentrum Karlsruhe Postfach 3640 7500 Karlsruhe

Institut für Chemie (1): Nuklearchemie Director: Prof. Dr. G. Stöcklin Senior reporter: Dr. S.M. Qaim Kernforschungsanlage Jülich Postfach 1913 5170 Jülich

Institut für Reine und Angewandte Kernphysik Director: Prof. Dr. K.O. Thielheim Senior reporter: Dr. H.-G. Priesmeyer Universität Kiel, Geesthacht Reaktorstr. 1 2054 Geesthacht/Tesperhude

Institut für Physik Director: Prof. Dr. W. Michaelis Senior reporter: Dr. R. Pepelnik GKSS-Forschungszentrum Geesthacht 2054 <u>Geesthacht</u>

Institut für Kernchemie Senior reporter: Dr. R. Michel Universität zu Köln Zülpicher Str. 47 5000 <u>Köln</u> Universität Mainz Fritz-Strassmann-Weg 2 6500 <u>Mainz</u>

Institut für Strahlenphysik Director: Prof. Dr. K.W. Hoffmann Senior reporter: J.W. Hammer Universität Stuttgart Allmandring 3 7000 <u>Stuttgart 80</u>

Fachbereich Physik der Technischen Universität München Abteilung E14, Forschungsreaktor Head and senior reporter: Prof. Dr. L. Köster 8046 Garching/München

Institut für Kernchemie Senior reporter: Prof. Dr. P. Patzelt Philipps-Universität Marburg Lahnberge 3550 Marburg/Lahn

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Physikalisch-Technische Bundesanstalt Abteilung 6, Atomphysik Director: Prof. Dr. S. Wagner Bundesallee 100 3300 Braunschweig

Fachinformationszentrum Energie, Physik, Mathematik Directors: Drs. W. Rittberger, E.-O. Schulze Senior reporter: Dr. H. Behrens Kernforschungszentrum 7514 Eggenstein-Leopoldshafen 2

APPENDIX II

CINDA Type Index

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Prepared by H. Behrens and G. Schust FIZ Energie, Physik, Mathematik, Karlsruhe

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REF VOL FAGE I | LA
NATE | r Cornents |
|--------|--------------|--------------|------------|-------------------|---------------------------------|------------|--|
| - | | | | | | | |
| YP | 774 | N, PAOTON | EXPT-PROG | 15+7 | NEANDC(L)-252U | 684 KIC | VOL.5.P.29.PEFELNIK+ TBL |
| Υņ | 176 | TOTAL | EXFT-PROG | 15+7 | NEANDC (E)-252U | 684 KIG | VOL.5.F.29.PEPELNIK+ TBL |
| t p | | POLARIZATION | EXPT-PROG | 78+6 | NEANDC(C)-252U | 684 TH | 5 VOL.5.P.43.9ULSKI+ ANALYZ POWER,NDG |
| 4 | | POLAFIZATION | EXPT-PF OG | 78+6 | NEANDC(E)-252U | 684 TH S | S VOL.5.P.43.BULSKI+ ANALYZ POWER,NDG |
| ų | 183 | TOTAL | EXPT-PPCG | 15+7 | NE4NOC(E)-252U | 634 KIG | VOL.S.P.29.PEPELNIK+ TBL |
| • | 154 | N,24 | EXPT-PR'JG | 15+7 | NEANDC (E)-252U | 684 KIQ | 5 VOL.5.F.29.PEPELNIK+ TBL |
| 25 | 190 | TOTAL | EXPT+PPCC | 15+7 | NEANDC(L)-252U | 684 KIG | 5 VOL.S.P.29.PEPELNIK+ TBL |
| ٥s | 192 | TATON | EXPT-PROG | 15+7 | NEANDC(E)-252U | 634 KIC | VGL.5.P.29.PEPELNIF+ TPL |
| IR | 191 | TOTAL | EXPT-PROG | 15+7 | NEANDC(E)-252U | 684 KIC | VGL.5.P.29.PEPELNIK+ TRL |
| AU | | N, GA#FA | EXPT-PROG | ND G | NEANDC (C)-252U | 684 K F# | . VCL.5.F.9.WISSHAK+ SELF-ARS CORR,NDG |
| AU | | TOTAL | EXPT-P+0G | 24+4 | NE ANDC (E.)-252U | 604 KI | ; VOL.S.P.25.PRIESMEYER+ TOL |
| ÞÞ | | TOTAL | EXPT-PROG | 24+4 | NEANDC(E)-252U | 684 KIG | VOL.5.P.25.PRIESMEYER+ TOL |
| ₽₽ | 206 | DIFF ELASTIC | EXPT-PROG | 78 +6 | NE ANDC (E)-252U | 684 THS | VCL.5.P.43.BULSKI+ ANGDIST,GRAPH |
| øв | 206 | POLARIZATION | EXPT-PROG | 78 +6 | NEANDC (E)-25 ZU | 684 THS | VCL.5.P.43.6ULSKI+ANALYZ POWER,GRAPH |
| PH | 207 | TOTAL | EXPT-PROG | 15+7 | NE ANDC (C) -252U | 684 KIG | VOL.S.P.29.PEPELNIK+ TBL |
| Pił | 208 | 4,2N | EXPT-PROG | 15+7 | NEANDC(E)-252U | 604 KIG | VOL.5.P.29.PEPELNIK+ TBL |
| Pfi | 208 | POLADIZATION | EXPT-PROG | 78+6 | NEANDC(E)-252U | 684 TKS | VOL.5.F.43.BULSKI+ ANALYZ POVER,NDG |
| TH | | THERMAL SCAT | 5.8PT-P90G | 13+0 52+0 | NE ANDC (E) -2520 | 684 MUR | VOL.5.P.46.KOESTER+ SCAT LENGTH+SIG |
| ТH | 232 | POLARIZATION | EXPT-PROG | 78+6 | NEANDC(E)-252U | 684 THS | VOL.S.P.43.BULSKI+ ANALYZ POWER,NDG |
| U | 235 | FRAG CHARGE | EXPT-PROG | PILE | NEANDC(E)-252U | 684 MNZ | VOL.5.P.38.DENSCHLAG+VIELD A=130-147 |
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