JAERI-M 84-103

SUMMARY OF JENDL-2 GENERAL PURPOSE FILE

June 1984

Edited by Tsaneo NAKAGAWA

日本原子力研究所 Japan Atomic Energy Research Institute

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Edited by Tsuneo Nakagawa

Department of Physics Tokai Research Establishment, JAERI

(Received May 12, 1984)

The general purpose file of the second version of Japanese Evaluated Nuclear Data Library (JENDL-2) was released in December 1982. Recently, descriptive data were added to JENDL-2 and at the same time the first revision of numerical data was performed. JENDL-2 (Rev1) consists of the data for 89 nuclides and about 211,000 records in the ENDF/B-IV format. In this report, full listings of presently added descriptive data are given to summarize the JENDL-2 general purpose file. The 2200 m/sec and 14-MeV cross sections, resonance integrals, Maxwellian and fission spectrum averaged cross sections are given in a table. Average cross sections were also calculated in suitable energy intervals.

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Keywords :

JENDL-2. Evaluated Data File. Descriptive Data, Resonance Integral, 2200-m/sec Cross Section, 14-MeV Cross Section, JAERI - M 84 103

JENDL 2 General Purpose ファイルの概要

日本原子力研究所東海研究所物理部 (編) 中 川 庸 維

(1984年5月12日受理)

日本の評価ずみデータライブラリー第2版 (JENDL 2)のGeneral Purpose ファイルは 1982年12月に公開された。最近、JENDL-2にデータの様子を示すコメント情報を追加すると 同時に第1回目の数値データの修正を行った。このJENDL 2 (Rev I)は89 核種のデータを 含み、ENDF B 1V フォーマットで約211,000 レコードから成っている。本種告では、JENDL

2 General Purpose ファイルの概要を示すために、今回加えられたコメント情報が示される。 2200 m sec および 14 MeV 断面積、共鳴積分値、マックスエルおよび核分裂スペクトルでの 平均断面積が表に与えられる。さらに透当なエネルギー区間を用いた平均断面積も計算された。

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ブルトニウム ~ 238	204
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Preface

The general purpose file of the second version of Japanese Eval uated Nuclear Data Library (JENDL 2) was released in December 1982. However this library, called JENDL 2 (RevO), does not contain descriptive data of evaluation. Recently the descriptive data were added and some errors already encountered in numerical data were corrected. This newly revised version of JENDL 2 is called as JENDL-2 Rev1

This report gives full listings of the descriptive data and some numerical data tables in order to summarize the JENDL 2 general purpose file. Details of correction of numerical data are described in Appendix 3, for the convenience of JENDL 2 users. Full documentation of JENDL-2 is now under preparation as a JAERI report.

Descriptive Data

In order to complete descriptive data part of JENDL-2 general purpose file, each evaluator of JENDL-2 data was asked to write a brief documentation, and to send it to the JAERI Nuclear Data Center. These descriptive data gathered were modified to correctly describe the present JENDL-2 (Rev1) and inserted into MF=1 of JENDL-2. All of them are listed in this report. In the descriptive data, 2200-m/sec cross sections and resonance integrals are given for the total, elastic, capture and fission cross sections, which were calculated with RESENDD¹⁾ and INTERN⁽²⁾. This calculation of resonance cross sections with RESENDD was made by applying accuracy of 1.0 percent.

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Data of JENDL 2 Rev1

JENDL 2 = (Rev) + contains the data of 89 nuclides and consists of about 211,000 records in the FNDF B IV format¹⁰. The data are stored in six files as shown in Table 1. Any processing codes for the ENDF B IV format can be used to handle JENDL 2 -Rev1 . However, A special care is required in treating resonance parameters, because the parameters for the multilevel formula often contain resonances whose J value is unknown. Table 2 shows the resonance region and the applied formula of each nuclide. The nuclides having this problem are marked with asteriskes. For these nuclides, RESENDD should be used to calculate resonance cross sections. To avoid incorrect calculation and for the convenience of users, pointwise data were also prepared and stored in 14 files as listed in Table 3. The data in these pointwise files were calculated with RESENDD applying accuracy of 1.0 percent. Energies of cross sections calculated from resonance parameters are written in the seven digit format. The pointwise data files contain the whole data of JENDL 2 except resonance parameters.

The 2200 m sec and 14 MeV cross sections, resonance integrals, and Maxwellian and fission spectrum average cross sections were obtained from the pointwise data with INTERN. The resonance integrals were calculated by assuming a cut off energy of 0.5 eV. The Maxwellian spectrum average cross sections were calculated in the energy interval from 10^{-5} eV to 3.0 eV, and fission spectrum average cross sections from 10^{-5} eV to 20 MeV with the following Watt-type fission spectrum.

 $S(E) = \sqrt{4/(\pi a^3 b)} \times \exp(-ab/4 - E/a) \times \sinh(bE),$

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where a and b were assumed to be 0.988 MeV and 2.249-10⁶ eV⁴, respectively. The results are listed in Appendix 1.

The average cross sections were also calculated in 75 energy intervals by using CRECTJ5⁴) as follows.

$$\sigma_1 = \int_{E_1}^{E_{1+1}} \sigma_2 E_2 \sigma E_2 \sigma E_2 (E_{1+1} - E_1)).$$

The average cross sections are given in Appendix 2.

Correction of Numerical Data

Numerical data of JENDL 2 (RevO) were checked by using FIZCON5⁵⁵ and PHYCHE⁶⁹ which are data checking codes for data in the ENDF/B format. Some characteristic data such as 2200-m sec cross sections were calculated and compared with other evaluated data and recommended values in Refs. 7 and 8. Furthermore the data were carefully checked by eyes,

The errors found by above processes were corrected and some data were replaced with more reasonable data by mainly using CRECTJ5⁴⁾ which is a computer code for compilation of evaluated data in the ENDF/B format. Details of this revision work are described in Appendix 3 to let the JENDL-2 users know which parts of data have been revised.

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Table 1
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1157 1379 1997 1997 1997 1997 2645	3510 3510 3770 3417 3417 3417 1086 1126 20156 1126 3761 3761 3761 3761 3761 3761 3761	20057 20000 20057	Records	

Tape No.	No.	Nuclide	MAT number	Records
203	9 10 11 12	42 Mo- 96 42 Mo 97 42 Mo 98 42 Mo 98 42 Mo 100	2424 2425 2426 2427 Total	1862 2782 2387 2154 25509)
204	1 2 3 4 5 6 7 8 9 10 11 12	72 Hf 174 72 Hf 176 72 Hf 177 72 Hf 178 72 Hf 179 72 Hf 180 73 Ta 181 82 Pb 0 82 Pb 204 82 Pb 206 82 Pb 207 82 Pb 208	2721 2722 2723 2724 2725 2726 2731 2820 2821 2822 2823 2823 2824 (Total	2621 3512 2878 2994 2363 2815 1311 3005 3794 3634 3600 2787 35314)
205	1 2 3 4 5 6 7 8 9 10 11 12 13	90 Th 228 90 Th 230 90 Th 232 90 Th 233 90 Th 233 91 Pa 233 92 U 233 92 U 233 92 U 233 92 U 235 92 U 236 92 U 236 92 U 238 93 Np 237 93 Np 239	2301 2302 2303 2904 2905 2911 2321 2322 2923 2924 2925 2931 2932 (Total	1549 1657 4881 1802 1873 1316 3160 1184 6307 1882 3907 3024 898 33440
206	1 2 3 4 5 6 7 8 9 10 11 12 13 14	94-Pu-236 94-Pu-238 94-Pu-239 94-Pu-240 94-Pu-241 95-Am-241 95-Am-242 95-Am-242 95-Am-242 96-Cm-243 96-Cm-243 96-Cm-244 96-Cm-244 96-Cm-245	2941 2942 2943 2944 2945 2946 2951 2952 2953 2954 2961 2962 2963 2964 (Total	1012 1696 4962 2746 2683 1675 4059 1591 3039 2541 902 4129 1279 2811 35025)

Table 1 (continued)

Table 2 Resonance Region and Formula

Nuclide	MAT	Spin	Resolved Res. (Formula) Unresolved Res.
1.H - 1	2011	1/2+	no resonances
1-H - 2	2012	1+	no resonances
3-Li- 6	2031	1+	no resonances
3-Li 7	2032	3/2-	no resonances
4-Be- 9	2041	3/2-	no resonances
5-B - 10	2051	3+	no resonances
6-C 12	2061	Ō+	no resonances
9-F - 19	2091	1/2+	$10^{-5} \text{ eV} \sim 100 \text{ keV}$ (SL) not given
11-Na 23	2111	3/21	500 eV - 150 keV (ML) not given
13 A1 27	2131	5 2	3 keV - 140 keV (ML) not given
14 Si - 0	2140		no resonances
20 Ca 0	2200		10^{-5} eV - 400 keV (ML) not given
20 Ca 40	2201	0,	10^{-5} eV - 400 keV (ML) not given
20 Ca 42	2202	Č+	10^{-5} eV - 400 keV (ML) not given
20 Ca- 43	2203	7.2-	10^{-5} eV - 30 keV (ML) not given
20 Ca 44	2204	0	10 ⁻⁵ eV - 400 keV (ML.) not given
20-Ca- 46	2205	0.	no resonances
20 Ca- 48	2206	0+	$10^{-5} \text{ eV} - 400 \text{ keV}$ (ML) not given
21-Sc- 45	2211	7/2	10 ⁻⁵ eV - 90 keV (ML*) not given
23 V 51	2231	7/2-	10^{-5} eV = 100 keV (ML+) not given
24 Cr 0	2240	•, •=	10^{5} eV = 300 keV (ML) not given
24 Cr 50	2241	0+	10 ⁻⁵ eV - 300 keV (ML) not given
24 Cr - 52	2242	Õ.	10 ⁻⁵ eV - 300 keV (ML) not given
24 Cr - 53	2243	3.2-	10^{-5} eV - 120 keV (ML) not given
24 Cr - 54	2244	0+	10^{-5} eV - 300 keV (ML) not given
25-Mn- 55	2251	5.2	$10^{-5} \text{ eV} - 100 \text{ keV}$ (ML) not given
26-Fe- 0	2260	•	10 ⁻⁵ eV - 250 keV (ML+) not given
26-Fe- 54	2261	04	10 ⁻⁵ eV 250 keV (ML) not given
26 Fe- 56	2262	01	10^{-5} eV \sim 250 keV (ML) not given
26-Fe- 57	2263	1/2-	10^{-5} eV - 200 keV (ML+) not given
26-Fe- 58	2264	0+	10 ⁻⁵ eV 100 keV (ML) not given
27-Co- 59	2271	7/2-	10^{-5} eV - 100 keV (ML+) not given
28-Ni- 0	2280	• / •	$10^{-5} \text{ eV} - 600 \text{ keV}$ (ML*) not given
28-Ni- 58	2281	0+	10^{-5} eV - 600 keV (ML) not given
28-Ni- 60	2282	0+	10^{-5} eV - 600 keV (ML) not given
28-Ni- 61	2283	3/2-	10 ⁻⁵ eV - 68.5158keV(ML*) not given
28-Ni- 62	2284	Ó+	10^{-5} eV - 600 keV (ML) not given
28-Ni- 64	2285	Õ+	10^{-5} eV - 600 keV (ML) not given
29-Cu- 0	2290	-	10 ⁻⁵ eV - 35 keV (ML*) not given
29-Cu- 63	2291	3/2-	10^{-5} eV - 35 keV (ML*) not given
29-Cu- 65	2292	3/2-	10 ⁻⁵ eV - 35 keV (ML*) not given
41-Nb- 93	2411	9/2+	10 ⁻⁵ eV - 7 keV (ML*) 50 keV
42-Mo- 0	2420	-, -=	10 ⁻⁵ eV - 50 keV (ML*) 100 keV
42-Mo- 92	2421	0+	10 ⁻⁵ eV - 50 keV (ML) 100 keV
42-Mo- 94	2422	0+	10 ⁻⁵ eV - 20 keV (ML) 100 keV
42-Mo- 95	2423	5/2+	10 ⁻⁵ eV - 2 keV (ML+) 100 keV
42-Mo- 96	2424	0+	10 ⁻⁵ eV - 19 keV (ML) 100 keV
42-Mo- 97	2425	5/2+	10 ⁻⁵ eV - 1.8 keV (ML+) 100 keV
42-Mo- 98	2426	0+	10 ⁻⁵ eV - 32 keV (ML) 100 keV
42-Mo-100	2427	0+	10 ⁻⁵ eV - 26 keV (ML) 100 keV

Table 2 (continued.)

Nuclide	MAT	Spin	Resolved Res. (Formul	a)	Unresolved Res.
72-Hf-174	2721	0+	0,5 eV ~ 220 eV	(ML)	50 keV
72 11 176	2722	0+	0.5 eV 700 eV	(ML)	50 keV
72-Hf-177	2723	7/2-	0.5 eV ~ 250 eV	(ML+)	50 keV
72-Hf-178	2724	0+	0.5 eV - 1.5 keV	(ML)	50 keV
72-Hf-179	2725	9/2+	0.5 eV - 250 eV	(ML+)	50 keV
72-Hr-180	2726	0-	0.5 eV - 2.5 keV	(ML.)	50 keV
73-Ta-181	2731	7/2+	10⁻ <u>5</u> eV - 1 keV	(ML+)	50 keV
82-Pb- 0	2820		10 ⁻⁵ eV - 500 keV	(ML.)	not given
82-Pb-204	2821	0+	10 ⁻⁵ eV - 50 keV	(ML)	not given
82-Pb-206	2822	0+	10 ⁻ " eV - 200 keV	(ML)	not given
82-Pb-207	2823	1/2-	10 ⁻⁵ eV - 500 keV	(ML)	not given
82-Pb-208	2824	0+	10 ⁻⁵ eV - 800 keV	(ML)	not given
90-Th-228	2901	0+	10 ⁻ 2 eV 7.798 eV	(ML)	not given
90-Th-230	2902	0+	10 [°] eV - 564.26 eV	(ML)	not given
90-Th-232	2903	0+	10 ^{-»} eV - 3,5 keV	(ML.)	50 keV
90-Th-233	2904	1/2+	no resonances		
90-Th-234	2905	0+	no resonances		
91 - Pa-233	2911	3/2-	2.38 eV - 17 eV	(SL)	1 keV
92-1 -233	2921	5/2+	1.0 eV ~ 100 eV	(SL)	30 keV
92-11 -234	2922	0+	10 ⁻³ eV ~ 215 eV	(ML)	not given
92-1 -235	2923	'7.∕ 2 -	1.0 eV - 100 eV	(SL)	30 keV
92-11-236	2924	0+	10 ⁻⁵ eV - 1.5 keV	(ML)	not_given
92 0 238	2925	0+	10 eV 4 keV	(ML)	50 keV
93 Np-237	2931	5/2+	10°° eV - 130 eV	(SL)	30 keV
93 Np-239	2932	5,2+	no resonances	~	
94-Pu-236	2941	04	10 ⁻² eV - 6 eV	(SL)	not given
94-Pu-238	2942	0+	10 ⁻³ eV - 500 eV	(ML)	not given
94-Pu-239	2943	_1 <u>∕</u> 2+	1.0 eV - 598 eV	(ML+)	30 keV
94-Pu-240	2944	0+	10 ⁻⁴ eV 4 keV	(ML)	40 keV
94-Pu-241	2945	5/2+	1.0 ev 100 ev	(SL)	30 KeV
94-Pu-242	2946	_U+	10 ev - 1.29 kev		not given
95-Am-241	2951	D/2-	10 ° ev - 150 ev	(+سلام)	30 KeV
95-Am-242	2902	<u> </u>	no resonances		
90-Am-242m	2903	5-	10 ° eV - 3.5 eV	(SL)	not given
90-Am-243	2904	D/2-	10° eV - 213 eV	([12]_*) (Mati)	30 Kev
90-UM-242	2901		10 ° ev - 2/3 ev	(ML)	not given
90-UTT-243	2302	0/2+	$10^{-5} e^{-1} = 1 e^{-1}$	(1717)*) (MT-)	not given
90-UM-244	2303	0+ 7.0	10 ⁻⁵ eV - 1 KeV	(171L) (671.)	not given
90-UM-245	2904	1/2+	In ev - on ev	(ol) 	not given

SL : Single-level Breit-Wigner formula. ML : Multilevel Breit-Wigner formula.

RM : Reich-Moore multilevel formula.

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* : There exist resonances of which J value is unknown. The resonance cross sections should be calculated with RESENDD.

Table 3	JENDL-2	(Revision-1) Pointwise	Files
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Tape No.	No.	Nuclide	MAT number	Records
251	1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	2011 2012 2031 2032 2041 2051 2061 2091 2111 2131 2140 2200 2201 2202 2203 2204 2203 2204 2205 2206 2206 2206	249 4623 698 610 545 1856 1178 1514 2640 1328 2117 11399 6783 5116 2713 3610 629 1053 48661)
252	1 2 3 4 5 6 7	21-Sc-45 23-V 51 24-Cr 0 24-Cr-50 24-Cr-52 24-Cr-53 24-Cr-53	2211 2231 2240 2241 2242 2243 2243 2244 (Total	14497 5796 10630 7283 7520 4052 5438 55216
253	1 2 3 4 5 6 7	25 Mn 55 26 Fe 0 26 Fe 54 26 Fe 56 26 Fe 57 26 Fe 57 26 Fe 58 27 Co 59	2251 2260 2261 2262 2263 2264 2271 (Total	12420 16075 3923 10162 4921 2470 15448 65419)
254	1 2 3 4 5 6	28-Ni- 0 28-Ni- 58 28-Ni- 60 28-Ni- 61 28-Ni- 62 28-Ni- 64	2280 2281 2282 2283 2283 2284 2285 (Total	23058 15734 12653 8263 9837 7665 77210

- 9 --

	Tape No.	No.	Nuclide	MAT number	Records
_	255	1 2 3 4 5	29 Cu- 0 29-Cu- 63 29-Cu- 65 41-Nb- 93 42-Mo- 0	2290 2291 2292 2411 2420 (Total	5528 4901 3879 20915 36674 71897)
	256	1 2 3 4 5 6 7	42-Mo- 92 42-Mo- 94 42-Mo- 95 42-Mo- 96 42-Mo- 97 42-Mo- 98 42-Mo-100	2421 2422 2423 2424 2425 2426 2426 2427 (Total	10865 9020 7899 11714 9044 19043 18711 86296)
-	257	1 2 3 4 5 6 7	72-Hf-174 72-Hf-176 72-Hf-177 72-Hf-178 72-Hf-179 72-Hf-180 73-Ta-181	2721 2722 2723 2724 2725 2726 2726 2731 (Total	4172 6408 9617 6918 6637 5576 19543 58871)
	258	1 2 3 4 5	82-Pb- 0 82-Pb-204 82-Pb-206 82-Pb-207 82-Pb-208	2820 2821 2822 2823 2823 2824 (Total	14750 6589 10144 9336 4426 45245
	259	1 2 3 4 5 6	90-Th-228 90-Th-230 90-Th-232 90-Th-233 90-Th-234 91-Pa-233	290i 2902 2903 2904 2905 2911 (Total	1917 8156 60674 1802 1873 2792 77214)
	260	1 2 3 4	92-U -233 92-U -234 92-U -235 92-U -236	2921 2922 2923 2924 (Total	5934 5861 12839 22484 47118)
	261	1 2 3	92-U -238 93-Np-237 93-Np-239	2925 2931 2932 (Total	73607 17079 898 91584)

Table 3 (continued)

Table 3 (continued)

262	1 2 3	94-Pu-236	2941	1301
	2	Q4., Put., 238		1001
	3	34 ° 1 U ° 200	2942	10455
	<u> </u>	94 Pu 239	2943	23240
	4	94 Pu 240	2944	46408
			(Total	81404 >
263	1	94 -Pu-241	2945	5891
	2	94 Pu 242	2946	18636
	3	95-Am-241	2951	18743
	4	95 Am-242	2952	1591
	5	95-Am 242m	2953	3192
			(Total	48053 >
264	1	95 Am 243	2954	21267
	2	96 Cm 242	2961	3664
	3	96 Cm-243	2962	5219
	4	96 Cm 244	2963	15295
	5	96 Cm 245	2964	4475
			(Total	49920)

Descriptive Data

Full listings of descriptive data of JENDL-2 (Rev1) are given. Characters of them were converted from capital letters to a normal style of mixture of capital and small letters. To output them, the ATF (Advanced Text Formatter for science) system of the FACOM-M 380 computer was used.

1 H - 1 MAT number - 2011

1 H I Hitachi Eval Apr76 M.Yamamoto JAERI 1261 Dist Mar83 Rev1 Nov83 History 83 03 Compliled by K.Shibata Main part was carried over from JENDL 1 data evaluated by M.Yamamoto, Details are given in ref. 1. 83 11 MF 2 was added. The transformation matrix given for MT-2 of MF 4. MF 1 General Information MT 451 Descriptive Data and Dictionary MF 2 Resonance Parameters MT 151 Scattering radius only 2200 m s cross sections and calculated res. integrals, 2200 m s res. integ. elastic 20.44 b 0.332 b capture 0.149 b 20.77 Б total MF 3 Neutron Cross Sections MT 1 Total Cross Section Sum of elastic and capture cross sections S TM Elastic Scattering Cross Section Below 100 keV, calculated by using effective range and scattering length parameters of Lomon and Wilson 2. Above 100 keV, the data of Hopkins and Breit 3 were recommended. MT 102 Canture Cross Section The data of Horsley 4 were recommended MT 251 Mu bar Calculated from the data in MF-4. MF=4 Angular Distributions of Secondary Neutrons MT=2 Below 100 keV, isotropic in the center of mass system was assumed. Above 100 keV, the data of hopkins and breit/3/were recommended. References 1) Igarasi S. et al.: JAERI-1261 (1979). 2) Lomon E. and. Wilson R.: Phys. Rev. C9(1974, 1392. 3) Hopkins J.C. and Breit G.: Nucl. Data Table A9(1971) 137. 4) Horsley A.: Nucl. Data A2 1966: 243.

1 of Deuterium

1 H 2 MAT number = 2012 2 JAERI Eval Jul82 K.Shibata.T.Narita.S.Igarasi 1 H JAERI M 83 006 Dist Mar83 Rev1 Nov83 History 83 01 New evaluation for JENDL 2. Details are given in Ref. 1. Data were compiles by the authors. 83 11 MF 2 was added. MF-1 General Information MT-451 Descriptive data and dictionary MF-2 Resonance Parameters MT 151 Scattering radius only 2200 m s cross sections and calculated res. integrals. 2200 m s res. integ. 3.389 elastic b capture 0.00055 Ь 0.000286 b total 3.390 b MF 3 Neutron Cross Sections MT 1 Total Based on a least squares fit to the experimental data of 2 81. S JM Elastic Total (n.2n) Elastic Capture. MT 16 n.2nBased on a least squares fit. Data listed in 9 11 were used. MT 102 Capture Below I keV. I v form normalized to the data of Ishikawa 12 Above 1 keV, evaluated on the basis of the inverse reaction /13 . MT 251 Mu-bar Calculated from the data in MF=4. MF-4 Angular Distributions of Secondary Neutrons MT=2.16 Calculated from the three-body model based on the Faddeev equation 14/. MF=5.6 Energy and Energy-Angular Distributions of Secondary Neutrons MT=16 The three-body model calculation. References 1) Shibata, K. et al.: JAERI M 83-006 (1983). 2) Adair, R.K. et al.: Phys. Rev. 89 (1953) 1165. 3 / Seagrave, J.D. and Henkel, R.L. : Phys. Rev. 98 (1955) 666. 4) Stoler, P. et al. : Phys. Rev. C8 (1973) 1539. 5) Davis, J.C. and Barschall, H.H. : Phys. Rev. C3 (1971) 1798.

2 of Deuterium

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1 of Lithium 6

3 Li 6 MAT number 2031

3 Li 6 JAERI Eval Jul77 S.Komoda, S.Igarası JAERI M 7148 Dist Mar83 Revi Nov83 History 77 Of New evaluation for JENDL 1. Details given in Ref. 1. 81 12 Partly revised by S.Komoda 2 for JENDL 2, and compiled by T. Nakagawa. 83 11 Comment data were added. MF 1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 No resonance parameters given. Scattering radius only. 2200 m s cross sections and calculated res. integrals 2200 m s res.integ. 0.736 b elastic capture 0.028 b 0.0126 b n.alpha 936.3 b 423 h total 937.1 b MF 3 Neutron Cross Sections M1 1 Total Sum of all the partial cross sections. ML S Elestic Calculated with the Kapur Peierls theory. Details given in Ref. 1 Between 2 and 6 MeV. based on the experi mental data by Knitter et al. 5 and Lane et al. 6° . MT 4 Total melastic Sum of MT 52 and 91. MT-52 Inelastic discrete Based on the data of Presser et al. 3. MT 102 Capture The thermal cross section of Bartholemew and Campion /4/ was extrapolated as 1 v up to 20MeV. MT-107 (n.alpha) Calculated with the Kapur Peierls theory. Details given in Ref. /1/. MT=24.91.103 (n.2n alpha)p, inelastic(cont), (n.p) The data of ENDF/B-IV are recommended. Mu-bar MT=251 Calculated from the data in MF=4. MF=4 Angular Distributions of Secondary Neutrons Based on the experimental data 5 - 8 . MT-2 Isotropic in the lab system. MT-24 MT-52 Isotropic in the center of-mass system. ENDF B-IV recommended. MT=91

MF=5 Energy Distributions of Secondary Neutrons

MT-24.91 Evaporation spectrum. Values of nuclear temperature were taken from ENDE B IV.

References

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3 Li 7 MAT number = 2032 3-Li - 7 JAERI Eval-Jul82 K.Shibata Dist-Mar83 Revl-Nov83 History 82-01 New evaluation for JENDL 2 by K.Shibata. 83-11 MF-2 was added. Some Q values and transformation matrix of MT-2 in MF-4 were modified. MF-1 General Information MT-451 Descriptive data and dictionary MF 2 Resonance Parameters MT-151 Scattering radius only 2200 m s cross sections and calculated res, integrals. 2200 m s res. integ. elastic 1.049 b 0.0454 b 0.0204 b capture 1.094 b total MF-3 Neutron Cross Sections MT-1 Total Below 1 keV, Total 1.04894 + Capture (b). Above 1 keV, data listed in 21 6 were used. MT-2 Elastic Elastic Total Reaction. Total inelastic MT=4 Sum of MT-51 and 91. n.2n Li 6 MT-16 ENDF B IV recommended. MT-24 n.2n alpha d ENDF B-IV recommended. MT-51 n.n` Leading to the 1st level (0.47748 MeV) in Li-7. Data listed in 7, 8 were used. MT--91 (n.n' alpha-t Data listed in .9 14 were used. MT=102 Capture 1/v form normalized to the data of Jurney /15/. MT=104 (n.d)He-6 ENDF B-IV recommended. MT=251 Mu-bar Calculated from the data in File4. MF=4 Angular Distributions of Secondary Neutrons MT=2 1.0E-5 eV to 40 keV : Isotropic. 50 keV to 14 MeV : Data listed in /16/-/18/ used. 14 MeV to 20 MeV : Optical model calculation with parameters of /19/.MT=16,24,91 Isotropic in the laboratory system.

MT=51 Isotropic in the center of mass system. MF-5 Energy Distributions of Secondary Neutrons MT=16.24.91 Evaporation spectrum. Values of nuclear temperature taken from ENDF B-IV. References 1) Hibdon, C.T. and Langsdorf, Jr., A. ; ANL-5171, p.7 (1954). 2) Hibdon, C.T. and Mooring, F.P. : '68 Washington Conf. 3) Meadows, J.W. and Whalen, J.F. : Nucl. Sci. Eng. 41 (1970)351. 4) Foster, Jr., D.G. and Glasgow, D.W. : Phys. Rev. C3(1971)576. 5: Goulding, C.A. et al. : private communication (1972). 6 Lamaze, G.P. et al. : Bull. Am. Phys. Soc. 24 1979 862. 7 Benveniste et al. : Nucl. Phys. 38 1962 300. 8 Presser, G. and Bass, R. : Nucl. Phys. A182 (1972) 321. 9 Wyman, M.E. and Thorpe, M.M. : LA 2235 1958 10 Brown, F. et al. : J. Nucl. Energy Parts A B 17 (1963) 137. 11 Hopkins, J.C. et al. : Nucl. Phys. A107 1968 139. 12 Lisowski, P.W. et al. : LA 8342 (1980). 13 Smith, D.L. et al. : Nucl. Sci. Eng. 78 (1981) 359. 14: Liskien, H. and Paulsen, A. : INDC(EUR) 014/G, p.14 (1980). 15) Jurney, E.T. : USNDC=9, p.109 (1973). 16) Lane, R.O. et al. : Ann. Phys. 12 (1961) 135. 17) Hogue, H.H. et al. : Nucl. Sci. Eng. 69 (1979) 22.

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4 Be 9 MAT number 2041 4 Be 9 JAERI I Eval Sep82 K.Sibata,K.Ioki (MAPL) Dist Mar83 Rev1 Nov83 History 82 09 New evaluation wad made by K.Sibata/JAERI / and K.Ioki (MAPI) 83 11 Comment was added. MT 1 General Information MT 451 Descriptive data MT 2 Resonance Parameters MT 151 Scattering radius only 2200 m sec cross sections and resonance integrals. 2200 m sec res integ. elastic 6.000 b 0.0076 b 3.42 milli b capture total 6.0076 h MT 3 Neutron Cross Sections MT 1 Total Below 1 keV. Total 6.0 (Capture (b). Above 1 keV. data listed in 1 10 were used. S IM Elastic Elastic Total Reaction. MT 16 n.2n Data listed in 11 15 were used. MT 102 Capture 1 v form normalized to the data of Jurney 16. MT 103 .n.p Evaluated on the basis of the data of Augustson and Menlove 17 by taking account of the branching ratio of 35 percent for Li9 => Be9+ => 2a + n. MT-104 (n.d) Based on the data of Scoebel 18/. MT=105 (n.t) Based on the data of Biro et al. /19/ and Qaim and Wolfle (20 /. MT-107 n.alpha Data listed in 21 - 25 were used. Only the transition to the ground state in He6 is given. MT-251 Mu-bar Calculated from the data in File4.

MT=4 Angular Distributions of Secondary Neutrons MT=2 1.0E 5 eV tO 40 keV : Isotropic in the center of mass sys. 50 keV to 14 MeV Data listed in 26 31 used. 15 MeV to 20 MeV : Optical model calculation with parameters of Ref. 32. MT 16 Isotropic in the laboratory system. MC 5 Energy Distributions of Secondary Neutrons MT 16 Evaporation spectrum. References 1 Adair, R.K. et al. : Phys. Rev. 75 (1949) 1124. 2 Bockelman, C.K. : Phys. Rev. 80 1950 1011. Bockelman, C.K. et al. : Phys. Rev 84 1951 69. 3 4 Hibdon, C.T. and Langsdorf, Jr., A. : Phys. Rev. 98 (1955) 223 5. Fowler, J.L. and Cohn, H.O. : Bull. Am. Phys. Soc. 4 (1959)385. 6 Bilpuch, E.G. et al. : private communication, 1962, 7) Schwartz, R.B. et al. ; Bull. Am. Phys. Soc. 16 (1971) 495. 8) Foster, Jr., D.G. and Glasgow, D.W. : Phys. Rev. C3(1971)576. 9 Cabe, J. and Cance, M : CEA-R-4524, 1973. 10 / Auchampaugh, G.F. et al. : Nucl. Sci. Eng. 69 (1979) 30. 11: Ashby, V.J. et al. : Phys. Rev. 111 (1958) 616. 12) Catron, H.C. et al. : Phys. Rev. 123 (1961) 218. 13) Holmberg, M. and Hansen, J. : Nucl. Phys. A129 (1969) 305. 14 Bloser, M. : Atomkernenergie 20 (1972) 309. 15) Drake, D.M. et al. : Nucl. Sci. Eng. 63 (1977) 401. 16 Jurney, E.T. : USNDC-11, p.149, 1974. 17 Augustson, R.H. and Menlove, H.O : Nucl. Sci. Eng. 54(1974)190 13 Scoebel, W. : Z. Naturforsch, A24 (1969) 289. 19 Biro, T. et al.: J. Inorg. Nucl. Chem. 37 (1975) 1583. 20 Qaim, S.M. and Wolfle, R. : Nucl. Phys. A295 (1978) 150. 21 Battat, M.E. and Ribe, F.L. : Nucl. Phys. 89 (1953) 80. 22 Stelson, P.H. and Campbell, E.C. : Nucl. Phys. 106(1957)1252. 23 Bass, R. et al. : Nucl. Phys. 23 (1961) 122. 24) Paic. G. et al. : Nucl. Phys. A96 (1967) 476. 25) Perroud, J.P. and Sellem, CH. : Nucl. Phys. A227 (1974) 330. 26) Lane, R.O. et al. : Phys. Rev. 133B (1964) 409. 27) Lane, R.O. et al. : Ann. Phys. 12 (1961) 135. 28) Levin, J.S. and Cranberg, L. : private communication, 1960. 29) Phillips, D.D. : private communication, 1961. 30) Marion, J.B. et al. : Phys. Rev. 114 (1959) 1584. 31) Hogue, H.H. et al. : Nucl. Sci. Eng. 68 (1978) 38. 32) Agee, F.P. and Rosen, L. : LA-3538-MS, 1966.

5-8 10 MAT number = 2051 A CONTRACTOR AND AND A CONTRACTOR AND AN 5 B 10 Hitachi Eval Apr76 M. Yamamoto JAERI -1261 Dist Mar83 Rev1 Nov83 History 76-01 Evaluation was made for JENDL 1 by M.Yamamoto. Details are given in Ref. 1 83 03 JENDL 2 data were taken from JENDL 1. 83 11 Comment was added. MF-1 General Information MT 451 Comments and Dictionary MF 2 Resonance Parameters MT 151 Scattering radius only 2200 m sec cross sections and resonance integrals 2200 m sec res integ. 2.173 b elastic 0.503 b 3836, b 0.0643 b capture 1720. b n.alpha/ n.t 2alpha 0.566 milli b 0.234 b total 3839. b MF 3 Neutron Cross Sections MI I Total Sum of partial cross sections MT 2 Elastic scattering Below 580 keV, calculated from the resonance parameters in BNL 325, 3rd ed. 2. Above 580 keV, ENDF B-IV recommended. MT 4 Total inelastic Sum of MT 51 to 91. MT 16 n.2n A straight line passing through the data of Mather and Pain .3 was assumed. MT=51-61,91,103,104,113 inelastic. (n,p), (n,d) and (n,t2a) ENDF/B-IV recommended. MT=102 Capture 1/v curve normalized to the recommended value of Ref./2/. MT=107,780,781 (n.a), (n.a0) and (n.a1) Below 235 keV, based on the measured data of Sowerby et al. /4/. Above 235 keV, ENDF/B-IV recommended. MT=113 (n,t 2a) ENDF/B-IV recommended. MT=251 Mu-bar ENDF B-IV recommended. MF=4 Angular Distributions of Secondary Neutrons MT=2,16,51-61.91 ENDF B-IV recommended. MF=5 Energy Distributions of Secondary Neutrons MT=16.91

ENDF/B-IV recommended.

References

- 1) Igarasi S. et al.: JAERI 1261 (1979).
- 2 Mughabghab S.F. and Garber D.I.: BNL 325. 3rd Edition (1973). 3 Mather D.S. and Pain L.F.: AWRE 047 69 (1969).
- 4 > Sowerby M.G. et al.: AWRE 6316 (1970).

1 of Carbon 12

6 C 12 MAT number = 2061 6 C 12 Hitachi Eval-Apr76 M.Yamamoto **JAERI 1261** Dist Mar83 History 76 04 Recommendation was made by M.Yamamoto (Hitachi for JENDL 1. Details given in Ref. 1. 83-03 Main part of JENDL 2 data was carried over from JENDL 1. 83 11 Comment was added. MF 1 General Information MT 451 Comments and dictionary MF 2 Resonance Parameters MT 151 Scattering radius only 2200 m sec cross sections and resonance integrals 2200 m sec res integ. 4.699 b elastic capture 0.0034 b 0.00153 Ь 4.702 b total MF-3 Neutron Cross Sections MT 1 Total Sum of partial cross sections. MI 2 Elastic scattering Below 2 MeV, based on the evaluated data of Nishimura et al. 2. Avobe 2 MeV. ENDF B IV recommended. Total inelastic MT 4 sum of MT 51 and 91. MT-51.91.107 Inelastic scattering and n.a ENDF B IV recommended. MT 102 Capture 1 v curve normalized to the recommended value of BNL 325 3rd edition 3. MT:251 Mu bar ENDF B-IV recommended. MF=4 Angular Distributions of Secondary Neutrons MT=2.51 ENDF/B-IV recommended. MT=91 Isotropic distributions in the center-of-mass system were assumed, and transformed into the laboratory system. MF=5 Energy Distributions of Secondary Neutrons MT=91 ENDF B IV recommended. References 1) Igarasi S. et al.: JAERI-1261 (1979). 2) Nishimura K. et al.: JAERI-1218 (1971). 3) Mughabghab S.F. and Garber D.I.: BNL-325, 3rd Edition (1973).

- 24 -

9 F - 19 MAT number - 2091

9 F 19 JAERI Eval Apr77 T.Sugi and K.Nishimura JAERI-M 7253 (1977) Dist Mar83 Revi Nov83 History 77 O4 Data above 100 keV were evaluated by T.Sugi and K.Nishimura (JAERI). Details are given in Ref. 1. 83 Ol Resonance parameters were evaluated by T.Sugi. 83 11 Modification of angular distributions was made. Comment data were added in MF 1. MF 1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved resonance parameters : 1.0E 5 eV 100 keV The single level Breit Wigner formula was used. Res. energis and Gam n : The first three levels were based on Johnson et al. 2. The 4th level was based on Mughabghab and Garber 3/. Gam g : The first three levels were based on Macklin and Winters 74%. The 4th level was taken from the mean value of Macklin and Winters 44, and Gabbard et al. 57. J : Based on Mughabghab and Garber 3. Calculated 2200 m s cross sections and res. integrals. 2200 m/s res. integ. 3.641 b elastic 9.6 milli b 19.6 milli b capture. total 3.651 b MF 3 Neutron Cross Sections Below 100 keV Background data are given. Above 100 keV MT 1 Total cross section Between 100 keV and 5.0 keV, evaluated based on the following experimental data. Energy range Experiments 100keV - 300keV Hibdon 7/ 300keV - 500keV Whalen et al. 8 Cabe 9 500keV - 1.15MeV 1.15MeV - 2.2MeV Elwyn 10 2.2MeV - 5.0 MeV Foster and Glasgov (11) 5.0MeV - 8.5MeV : Determined from the least squares fitting of the experimental data. See Ref. 1. MeV 20MeV : Calculated with the optical model by using 8.5MeV the code ELIESE 3 12 The optical potential parameters were determined with the code TOTALCS 13 so that the calculated cross section curve is fitted to the experimental data from 3.1 MeV to 20 MeV. Optical potential parameters V = 45.54 - 0.733 E (MeV).

₩s	15.53 (der	.Woods	Saxon > (MeV
Vso	10.0		MeV 1.
r0	rs rso	1.31	- £m 🤖
a	aso 0.66		of mark.
b	0.47		(fm).

S TM Elastic scattering cross section Derived by subtracting the nonelastic from the total cross section.

MT 3

Nonelastic (not given in file) 15 - 6MeV : Sum of all cross sections other than elastic 1.0E 5 scattering.

6MeV 20MeV : Calculated with the Hauser Feshbach method ELIESE 3 12 .

MT 4 Total inelastic scattering cross section

Up to 1 MeV : Sum of the inelastic scattering cross sections for the lowest two levels of 110 keV and 197 keV.

- 1MeV 5.5MeV Calculated with the Hauser Feshbach method ELIESE 3 12 . The optical potential parameters are V
 - 51 56 MeV . 1.792 E

Ws. 11.82 der. Woods Saxon - MeV .

The other parameters are the same as those for the total cross section. These parameters were determined so that the calculated total cross section curve is fitted to the experimental data from 0.61 MeV to 20 MeV.

= 20MeV : Derived by subtracting the (n.alpha). 5 5MeV n.p., (n.d., (n.t.), (n.n'alpha), (n.alpha n'), (n.n'p), n.p.n', n.2n) and capture cross sections from the nonelastic.

MT 16 (n,2n) cross section Calculated by fitting the Pearlstein's function /14/ to the experimental data.

MT 22 n.n' alpha and n.alpha n' cross sections Calculated with a statistical model by using Pearlstein's empirical formula.

- n.n'p and n.p.n' cross sections MT 28 Calculated with a statistical model by using Pearlstein's empirical formula.
- MT 51 56 Inelastic for the lowest six excited states Up to 1MeV : Based on the experimental data of Broder et al. /16
 - 1MeV = 5.5MeV : Calculated with the Hauser-Feshbach method ELIESE-3 (12) taking into account (n.alpha) and (n.p) as competing processes. The level scheme of F-19, N-16 0.19 were taken from Ajzenberg-Selove /17/./18/.
 - 5.5MeV 20MeV : Ratio of the inelastic scattering cross sections for the lowest six excited states and for the continuum to the total inelastic scattering cross section were calculated with the Hauser-Feshbach method (ELIESE-3 (12/). The level density parameter of 3.609 1/MeV 19 and pairing energy of 2.52 MeV /20/ were used.

Inelastic to continuum MT=91

Calculated with the same method as MT=51-56, 5.5 MeV -20 MeV.

MT=102 Capture cross section 100keV - 1.87MeV : Based on the experimental data of

3 of Fluorine 19

Gabbard et al. 5. 1.87MeV - 20MeV : Assumed to decrease with I v law. MT-103 (n,p) cross section Up to SMeV : Based on the experimental data of Bass et al. SI 9MeV - 20MeV : Calculated with the statistical model by using Pearlstein' empirical formula. MT 104 n.d) cross section Calculated with the Pearlstein's empirical formula /15/. The cross section was normalized to 39.5 milli barns at 14.4 MeV. MT 105 n.t cross section Calculated with the Pearlstein's empirical formula 15. The cross section was normalized to 15.0 milli barns at 14.4 MeV. MT 107 n.alpha cross section Below 9 MeV. Based on the following experimental data: Up to 4MeV Davis et al 22. Smith et al. 23. 4MeV 5.5MeV 9Me-V 5.5MeV Bass et al. 21. Above 9 MeV, Calculated with the Pearlstein's formula, MT 251 Average cosine in the laboratory system Derived from the angular distributions. MF 4 Angular Distributions of Secondary Neutrons S 1M Calculated with optical model .CASTHY 24 -MT 16.22.28 Assumed to be isotropic in the laboratory system. MT 51 56 Assumed to be isotropic in the center of mass system. MT 91 Assumed to be isotropic in the center of mass system and transformed into the laboratory system. MF 5 Energy Distributions of Secondary Neutrons MT 16.22.28.91 Evaporation spectra were given. References 1 Sugi T. and Nishimura K.: JAERI M 7253 (1977). english translation : ORNL TR 4605. 2: Johnson C.H. et al.: ORNL 5025 (1975). 3) Mughabghab S.F. and Garger D.I.: BNL 325, 3rd Ed. Vol.1 (1973) 4) Macklin R.L. and Winters R.R.: Phys. Rev. C7, 1766 (1973). 5) Gabbard F. et al.: Phys. Rev. 114, 201 (1959). 6) Mughabghab S.F. et al.: Neutron Cross Sections, Vol.1, Part A. Z=1-60. Academic Press (1981). 7: Hibdon C.T.: Phys. Rev. 133B, 353 1964 . 8' Whalen J. et al.: communication from the NEA Data Bank (1967). 9 ; Cabe J. : CEA R 3279 1967 10 Elwyn A.J. et al.: Nucl. Phys. 59 113 1964 .. 11) Foster D.G.Jr. and Glasgev D.M.: Phys. Rev. C3, 576 (1971). 12) Igarasi S.: JAERI 1224 (1972). 13) Igarasi S.: private communication. 14) Pearlstein S.: Nucl. Sci. Eng. 23, 238 (1965).
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11-Na- 23 MAT number = 2111

11 Na 23 Hitachi Eval Mar75 M.Yamamoto JAERI 1261 Dist Mar83 Revi Nov83 14ERI 1261 History 75 03 Evaluated by M.Yamamoto and compiled by C.G. for JENDL 1 with some modifications. Detailes are given in Ref. 1. 83 03 JENDL 1 data were taken for JENDL 2, and MF-5 was modified. 83 11 Q values and MF=4 were corrected. MF 1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved resonance parameters Evaluation was made on the basis of recommended values by Mughabghab and Garber 2. Paik and Pitterle 3. and the experimental data by Hockenbury et al. 41. Resonance cross sections can be obtained with MLBW formula in the energy region from 500 eV to 150 keV. 2200 m s cross sections and calculated res. integrals. 2200 m/s res. integ. 3.170 b elastic 0.530 b 0.329 b capture 3.700 b total MF-3 Neutron Cross Sections Thermal energy region from 1.0E 5 eV to 500 eV -The total and capture cross sections of 3.7 and 0.53 barns were taken from BNL 325 3rd ed. 2 . and the form of 1 v was assumed for the capture cross section. Resonance energy region (from 500 eV to 150 keV) Small background cross sections were given for the total and capture cross sections to connect smoothly resonance cross sections with those for below and above resonance region. Above 150 keV MT=1 Total Based on the following experimental data. 150 keV - 550 keV : Stelson and Preston /5/ 550 keV - 10 MeV : Cierjacks et al. 6/ Above 10 MeV : Glasgov and Foster 7 Elastic scattering MT-2 Obtained by subtructing partial cross sections from the total

MT=4 Total inelastic scattering Below 4 MeV, summation of partial inelastic scattering cross sections (MT=51 - 57). Above 4 MeV, based on the evaluated data by Schmidt /8/ and the experimental data by Martin and

cross section.

Stewart 9/ and Sukhanov and Rakavishnikov 10. MT=51 Inelastic scattering to the first level Based on the experimental data by Towle and Gilboy 11. Chien and Smith $\sqrt{12}$. Lind and Day $\sqrt{13}$ and Shipley et al. 14. MT 52-57 Inelastic scattering to the second to 7th levels Based on the data by Freeman and Montague [15], Lind and Day 13. Towle and Ovens /16 and the evaluated data by Schmidt 8. MT-91 Inelastic scattering to continuum level Determined from the total and partial inelastic scattering cross sections. MT 16 (n,2n)Based on the data by Menlove et al. 17. MT 102 Capture Based on the data by le Rigoleur et al. 18. MT 103 n, pBased on the data by Williamson 19, Bass and Saleh 20. and Picard and Williamson 21 MT 107 n.alpha Based on the data by Williamson 19 and Woelfer and Bormann 22 MT 251 Mu bar Obtained from the angular distributions (MF 4.MT=2). MF 4 Angular Distributions of Secondary Neutrons S IM Based on the compilation work by Garber et al. /23/. and evaluation work by Moorhead /24 . MT-16 Isotropic in the lab system. MT 51-57 Isotropic in the center of mass system. MT 91 Isotropic distribution in the center of mass system was transformed into the lab system. MF 5 Energy Distributions of Secondary Neutrons MT-16.91 Evapolation spectrum References 1) Igarasi S. et al.: JAERI 1261 (1979). 2) Mughabghab S.F. and Garber D.I.: BNL 325, 3rd Edition Vol.1 1973 3) Paik N.C. and Pitterle T.A.: WARD-4181-2 (1971). 4) Hockenbury R.W. et al.: Phys. Rev. 178, 1746 (1969). 5) Stelson P.H. and Preston W.M.: Phys. Rev. 88, 1354 (1952). 6) Cierjacks S. et al.: 1968 Washington Conf., 743 (1968), also KfK-1000 (1968). 7) Glasgow D.W. and Foster D.G.Jr.: HW-73116, 51 (1962). 8) Schmidt J.J.: KfK-120 (1966). 9) Martin P.W. and Stewart D.T.: J. Nucl. Energy, A/B19, 447 (1965). Martin P.W. and Stewart D.T.: Can. J. Phys., 46, 1657 (1968). 10) Sukhanov B.I. and Rukavishnikov B.G.: Atom. Energ., 11, 398 (1961), also Sov. Atom. Energy, 11, 1044 (1961). 11) Towle J.H. and Gilboy W.B.: Nucl. Phys., 32, 610 (1962). 12) Chien J.P. and Smith A.B.: Nucl. Sci. Eng., 26, 500 (1966). 13) Lind D.A. and Day R.B.: Ann. Phys., 12, 485 (1961).

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1 of Aluminium-27

13-A1 27 MAT number - 2131

13 Al 27 JAERI Eval Mar75 JENDL C.G.

JAERI 1261 Dist Mar83 Rev1 Nov83

History

- 75 03 Evaluated for JENDL 1 by JENDL Compilation Group. Details are given in Ref. 1.
- 83 03 JENDL 1 data were taken for JENDL 2.

83 11 Angular distributions were modified.

- MF-1 General Information MT-451 Descriptive data and dictionary
- MF 2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW formula The resonance parameters were taken from BNL-325 3rd edition 2 and some modifications were made so that the calculated total cross section might satisfactorily reproduce the experimental data. A negative resonance was added.

2200 m s cross sections and calculated res. integrals.

	CCUU m s	res. integ.
elastic	1,500 b	
capture	0,230 b	0.147 Б
total	1.730 Б	

MF 3 Neutron Cross Sections

Below 3 keV

The 2200 m s cross sections recommended in Ref. 2 were adopted. The capture cross section was assumed to have the the form of 1 v below 500 eV and was estimated by an eye guide between 500 eV and 3 keV. The elastic scattering was assumed to be constant

Resonance energy region from 3 keV to 140 keV) Background cross sections were given to reproduce the resonance structure around 10 keV.

Above 140 keV

MT=1 Total

Obtained with an eye-guide method.

MT=2 Elastic scattering

Obtained by subtructing partial cross sections from the total cross section.

MT=4.51-58.91 Inelastic scattering cross sections Calculated with optical and statistical models by using the code CASTHY 3. The following optical potential parameters were determined to reproduce well the total cross section The capture. n.p. n.alpha and n.2n) reactions were taken into account as the competing processes.

Optical potential parameters

2 of Aluminium-27

۷	42	42.39 ~ 0.18+E,	Ws≈ 4,5,	Vso 5.8	(MeV)
r	24	rso = 1,235,	rs= 1.137		(fm)
а	~ 5	aso = 0,65 .	as 0,6		' £m⇒

Level scheme

1

No.	Energy (MeV)	Spin Parity
g.s.	0.0	5,2+
1	0.842	12+
2	1.013	3.2 +
3	1.65	52
4	1.83	5/2 +
5	2.21	7/2 +
6	2.73	5,2+
7	2.98	32
8	3.00	92.

Levels above 3.7 MeV were assumed to be overlapping.

MT 16 n.2n

Calculated with Pearlstein's method 4 .

MT-102 Capture

Obtained with an eye guide method.

MT 103 n.p)

Evaluated data by Asami 252 were adopted.

- ME-107 = (n,alpha) Evaluated data by Kanda and Nakasima =67 were adopted.
- MT-251 Mulbar

Calculated with optical model.

MF 4 Angular Distributions of Secondary Neutrons

MT 2 Calulated with optical model.

- MT=16 Isotropic in the lab system.
- MT-51-58 Isotropic in the center of mass system.
- MT 91 Isotropic distribution in the center of mass system was transformed into the lab system.
- MF-5 Energy Distributions of Secondary Neutrons MT-16.91 Evaporation spectrum

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- 2) Mughabghab S.F. and Garber D.I.: BNL 325, 3rd Edition, Vol.1 (1973).
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- 5) Asami T.: 1976 Consultants' Meeting at Vienna, IAEA-208 Vol.2, 395 (1978).
- 6) Kanda Y. and Nakasima R.: JAERI 1207 (1972).

14 Si 0 MAT number = 2140

14 St O JAERI Eval Mar78 T. Asami JAERI 1261 Dist Mar83 Rev1 Nov83 History 76 03 The evaluation was made by T.Asami (JAERI) for JENDL 1. Details are given in Ref. 1. 83 03 JENDL 1 data were adopted for JENDL 2 and extended to 20 MeV. 83 1' Small correction was made. Comment data were added. MF 1 General Information MT 451 Descriptive data and dictionary MF 7 Resonance Parameters MI 151 No resonance parameters are given. .200 m sec cross sections and calculated resonance integrals. 2200 m sec Res. Integ. 2.200 b elastic capture 0.156 b 0.079 b 2.356 b total MF 3 Neutron Cross Sections Energy region below 1 keV. MT 1 Total Sum of the elastic scattering and capture cross sections. - S TM Elastic scattering The constant value of 2.2 barns was taken from Ref. 2/. MT-102 Capture The cross section of 0.156 barns at 0.0253 eV was obtained for natural silicon from the following experimental data. For Si 28 and Si 29' Spitz and Boer 3. : Koehler and Knopf '4/, Ryves /5/. For S1 30 The shape of cross section was assumed to be the form of 1/v. Energy region above 1 keV. MT=1 Total Obtained by averaging the following experimental data. Below 200 keV: Fields and Walt /6/. Above 200 keV: Cierjacks et al. /7/, Schwartz et al. /8/, Perey et al. /9/, Cabe and Cance /10/. MT=2 Elastic scattering Obtained by subtracting partial cross sections from the total cross section. MT=4.51-83.91 Inelastic scattering Calculated with a statistical and optical model code CASTHY 3117. The optical potential parameters of Bhat et al. /12/ was used. The level schemes are listed below. The capture, (n,p), (n,alpha, and (n,2n) reactions were taken into account as the competing processes.

	Level	scher	es for S	i isoto	pes	
	Si-28		Si-29		Si 30	
No.	Energy	(MeV)	Energy	(MeV)	Energy	MeV 👘
g.s.	0.0	01	0,0	1.2	0.0	0
1	1.799	21	1.273	3/21	2,235	21
2	4.617	4+	2,028	5/21	3,498	2.
3	4.975	0+	2,425	3.2	3.770	1+
4	6.267	3+	3.067	5.24	3 788	0+
5	6.691	0+	3.623	7.2	4,809	S+
6	6.878	3	4.380	7 2+	4.830	31
7	6.889	4+	4.741	9/2+	5.230	3+
8	7,381	1+	4.839	1 2+	5.280	4+
9	7.416	2₁	4,895	5 2+	5.372	0+
10	7.798	3-	4,933	3.2	5.487	3
11	7,935	2			5.612	2
12					5.951	4+

Overlapping levels were assumed above 8.25~MeV for Si-28, 5.2 MeV for Si 29 and 6.5 MeV for Si 30.

The Q values of inelastic levels of natural silicon were adjusted to keep the threshold energy of each level.

MT≈ !6 ___n.2n⇒

Constructed from the isotope data evaluated as follows. For Si 28, evaluation was based on the experimental data. For others, the cross section was calculated with Pearlstein's method -13%.

MT 102 Capture

For the energy region from 1 keV to 1.6 MeV, the cross section was calculated from the resonance parameters of Boldeman et al. -14^{+} and Kenney et al. -15^{+} , and averaged. Then it was renormalized at 1 keV to the 1.9 cross section mentioned above. Above 1.6 MeV, the cross section was estimated so as to increase and pass through the value of 0.56 milli barns at 14.1 MeV which was obtained from the experimental data by Cvelber et al. 16 and Rigaud et al. 17.

Constructed from the isotope data evaluated as follows.

- Si 28: From 5.0 to 7.8 MeV, the cross section was obtained by eye-guiding the experimental data of Marion et al. 18/ and Jeronymo et al. /19/. The shape of the data of Bass et al. /20/ was adopted, and renormalized by the factor of 1.4 in the energy range from 7.8 MeV to 9.0 MeV. Above 9.0 MeV, the curve was drawn by an eye guide.
- Si-29: The shape of the Si-28(n,p) cross section was normalized to 0.120 barns at 14.5 MeV /21,22,23/.

Si-30: Ignored. MT=107 (n.alpha)

Constructed from the isotope data evaluated as follows.

- Si-28: Based on the data of Birk et al. 24. Mainsbridge et al. 25 and Singh 23
- Si-29: Based on the data of Konijr and Lauber /26/, Birk et al. /24 and Singh /23/.
- Si-30: The same shape as the Si-28(n,alpha) was adopted and normalized to 0.070 barns at 14.5 MeV /22,23/.

3 of Natural Silicon

- MT=251 Mu-bar Calculated with CASTHY /11/.
- MF-4 Angular Distributions of Secondary Neutrons
 - MT=2 Calculated with CASTHY 11.
 - MT=16 Isotropic in the laboratory system.
 - MT-51-88 Isotropic in the center of mass system.
 - MI-91 Isotropic distribution in the center of mass system was transformed into the laboratory system.
- MF 5 Energy Distributions of Secondary Neutrons MT-16.91 Evaporation spectra.

- 1 / Igarasi S. et al.: JAERI 1261 (1979).
- 2 Mughabghab S.F. and Garber D.I.: BNL 325, 3rd Ed., Vol. 1 1973.
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20-Ca- 0 MAT number = 2200

20-Ca- 0 Mitsui E.S.Eval-Apr80 M.Hatchya Dist-Mar83 Rev1-Feb84 History 80-04 New evaluation was made by M.Hatchya (Mitsui). 83-11 Ang. Dist. was modified, and comment data were added. 84-02 The total and elastic scattering cross sections were reevaluated on the basis of experimental data. MF=1 General Information MT 451 Descriptive data and dictionary MF-2 Resonance Parameters MT-151 Resolved resonance parameters for MLBW formula Parameters were evaluated on the basis of the following data Ca-40 : BNL 325 3rd ed. /1/, Musgrove et al. /2/, Singh et al. /3/. Ca-42.43.44 : Musgrove et al. /4/. : no resonances were given. Ca-46 : BNL 325 3rd ed. /1/. Ca 48 Resonance energy region of each isotope is Ca 40.42.44 : below 400 keV. : below 30 keV. Ca-43 : below 500 keV. Ca 48 Calculated 2200 m/sec cross sections and resonance integrals. 2200 m sec Res. Integ. 2.963 b elastic 0.4307 b 0.0024 b capture 0.226 b 0.349 b n.alpha total 3,396 b MF=3 Neutron Cross Sections Above resonance region. MT=1 Total cross section Between 400 keV and 5.0 MeV, the cross section was determined from the experimental data by Perev et al. /5/ and Cierjacks /6/. Above 5.0 MeV, the optical model calculation with CASTHY /7/ was adopted. Optical potential parameters for Ca-40, 43, 44, 46 and 48 taken from Ref. /8/. V = 46.72(MeV). Ws = 9.13(MeV). Vso= 5.37 MeV . r0 = rso = 1.26fm. $z_{\rm S} = 1.39$ ⊡fm . fm). a = aso= 0.76 b = 0.40(fm).

Optical potential parameters for Ca-42 obtained so as to

2 of Natural Calcium

reproduce the Ca-42 total cross section $V = 52.06 \pm 0.023 + e^{-(MeV)}$, $Ws = 5.57 \pm (MeV)$, $Vso = 5.37 \pm (MeV)$, Others are the same as above parameters.

MT-2 Elastic scattering cross section Derived by subtracting partial cross sections from the total cross section.

MT-4.51 88.91 Inelastic scattering cross sections Calculated with optical and statistical model code CASTHY.7.1

Level scheme

Level scheme for Ca 40 was taken from Ref. 8° and for other isotopes from Table of Isotopes 9° , Q values of natural calcium data were re-calculated from threshold energies.

MT	′Q keV	Ca	40	Ca 42	Ca	43	Ca 🚽	14	Ca	46	Ca⊷48
51	372.2				373	52					. , , , , , , , , , , , , , , , , , , ,
52	592.4				593	3.2			· · ·		
53	988.7				990	3.21					4
-54	1154.5						1157	24			
-55	1342.8								1347	2+	4.11
56	1392.3				1395	521					
57	1522.9			1525 2+							
-58	1835.3			1837 0+							<u>.</u> .
59	1879.4						1884	0+			
60	2278.2						2283	4+			_
61	2420.9			2424 2							-
62	2650.8						2657	2+			
63	2749.3			2752 4							
64	3014.5								3024	21	<i></i>
65	3185.8			3189 6							
66	3441.2			3445 3							
67	3601.7								3613	3	-
68	3737.3	3737	3								
69	3816.3	-									3832 2
70	3904.3	3904	2+	-	~		-		-		
71	4484.9	-									4503 4+
72	4488.7	-		-			-				4507 3-
73	4492.3	4492	5-	-	-						
74	4593.3				***		-		-		4612 3+
75	5249.4	5249	2+						-		
76	5348.0		_	-					-		5370 3-
77	5627.4	5627	2+	-							
78	6285.5	6285	3-	-			~*				-
79	6585.5	6585	3-	-	-		-		-		-
80	6910.5	6910	2+								
81	6932.5	6932	3-						~		
82	7860.6	7860	2-	-	-		-		-		-
83	7930.6	7930	4+	-	-				-		
84	8090.6	8090	2+	-			_		-		
85	8280.6	8280	2+						-		-

- 38 -

86 8371.6 8371 4+ -91 1928.2 8900 c 3884 c 1931 c 3285 c 4463 c 6614 c easily and the manufacture and the second Level density parameters (Gilbert and Cameron (10/) a constant constant as a constant constant constant and a second second second second second second second seco isotope 40 41 42 43 44 45 46 47 48 49 isotope a 1 MeV. 7.135 7.075 6.84 7.20 S C 1 SQRT.MeV: 3.03 3.08 3.07 3.19 Delta.MeV. 9.37 1.83 3.13 1.83 Fw. MeV. 9.121 7.52 9.26 7.20 9.131 7.522 8.76 7.39 Ex (MeV) A THE CONSIST OF A STREET AND A STREET $M\Gamma$ 16 (n.2n) cross section Based on available data. M¹22.28 (n.n'alpha) and (n.n'p) cross sections Derived from the calculation by Fu /8/ and available data. MT-102 Capture cross section Calculated with CASTHY /7/. MT=103.107 (n.p) and (n.alpha) cross sections Statistical and pre-equilibrium model calculations using the optical potential parameters and the level density parameters given above. Fitted to available data. MT-111 n.2p cross section This cross section was given only for Ca 40 by adopting the calculated values by Fu 8. MT=251 Mu bar Calculated with optical model. MF=4 Angular Distributions of Secondary Neutrons MT=2 Optical model calculation MT=51-88 Isotropic in the center-of-mass system. MT=16.22.28 Isotropic in the laboratory system. MT=91 Isotropic distribution in the center-of-mass system was transformed into the laboratory system. MF=5 Energy Distributions of Secondary Neutrons MT=16.22.28.91 Evaporation spectra.

- 1) Mughabghab S.F. and Garber D.I.: BNL-325, 3rd Edition, Vol.1 (1973).
- 2) Musgrove A.R.de L. et al.: Nucl. Phys. A259, 365 (1976).
- 3) Singh U.N. et al.: Phys. Rev. C10. 2143 (1974)
- 4) Musgrove A.R.de L. et al.: Nucl. Phys. A279, 317 (1977).
- 5) Cierjacks S.: KfK-1000 (1968).
- 6) Perey F.G. et al.: ORNL-4823 (1972).
- 7) Igarasi S.: J. Nucl. Sci. Theonol., 12, 67 (197E).
- 8) Fu C.Y.: Atomic Data and Nuclear Data Table 17, 127 (1976).
- 9) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed., Wiley-Interscience (1978).
- 10: Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).

1 of Calcium-40

20-Ca- 40 MAT number = 2201 -----'n 20 Ca 40 Mitsui E.S.Eval-Apr80 M.Hatchya Dist-Mar83 Rev1-Feb84 History 80 04 New evaluation was made by M.Hatchya (Mitsui). 83-11 Ang. dist. was modified. 84 02 Comment vas added. MF-1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved resonance parameters for MLBW formula Resonance energy region 1.0E 5 eV to 400 keV. Parameters were evaluated on the basis of the following data. BNL 325 3rd ed. 11. Musgrove et al. 21. Singh et al. 3 Calculated 2200 m sec cross sections and resonance integrals 2200 m/sec Res. Integ. 3.010 b elastic 0.216 b 0.410 b capture 0.355 b (n.alpha) 0.0025 Ь 3 423 b total MF 3 Neutron Cross Sections Above resonance region. MT 1 Total cross section The optical model calculation with CASTHY 4 was adopted. Optical potential parameters taken from Ref. 54. V 46.72 MeV . Ws 9.13 (MeV). Vso- 5.37 (MeV). r0 = rso = 1.26 (fm). rs = 1.39(fm). a ≕ aso≈ 0.76 (fm), b = 0.40 (**fm**). MT=2 Elastic scattering cross section Derived by subtracting partial cross sections from the total cross section. MT=4.51-66.91 Inelastic scattering cross sections Calculated with optical and statistical model code CASTHY 4. Level scheme Taken from Ref. 6 No. Energy (MeV) Spin-Paity

g.s. 0.0 0 + 1 3.737 3 --2 3,904 2 + 3 4.492 5 2 + 4 5.249 5 5.627 2 1 6 6.285 3 7 3 6,585 8 21 6.910 9 6.932 3 10 2.4 7.860 7.930 11 4 + 15 8,090 2 4 13 2 + 8.280 8.271 14 4 . 15 8.540 . 5 16 8.553 Continuum levels were assumed above 8,99 MeV. Level density parameters (Gilbert and Cameron (7.5)isotope 40 41 a (1/MeV) 5,43 6,00 MT=16 (n.2n) cross section Based on available data. MT=22.28 (n.n'alpha) and (n.n'p) cross sections Derived from the calculation by Fu 5. MT=102 Capture cross section Calculated with CASTHY 4 . MT=103.107 n.p: and n.alpha cross sections Statistical and pre-equilibrium model calculations using the optical potential parameters and the level density parameters given above. Fitted to available data. MT=11! (n.2p) cross section The calculated values by Fu /5/ were adopted. MT=251 Mu-bar Calculated with optical model. MF=4 Angular Distributions of Secondary Neutrons MT=2,51-66,91 Optical model calculation MT=16,22,28 Isotropic in the laboratory system. MF=5 Energy Distributions of Secondary Neutrons MT=16.22.28.91 Evaporation spectra. References 1) Hughabghab S.F. and Garber D.I.: BNL-325, 3rd Edition, Vol.1 (1973).

- 2) Musgrove A.R.de L. et al.: Nucl. Phys. A259, 365 (1976).
- 3) Singh U.N. et al.: Phys. Rev. C10, 2143 (1974)
 4) Igarasi S.: J. Nucl. Sci. Theonol., 12, 67 (1975).
- 5) Fu C.Y.: Atomic Data and Nuclear Data Table 17, 127 (1976).
- 6) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed., Wiley-Interscience (1978).
- 7) Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).

1 of Calcium-42

20-Ca- 42 MAT number = 2202 -----20 Ca 42 Mitsui E.S. Eval Apr80 M. Hatchva Dist Mar83 Revi Feb84 History 80-04 New evaluation was made by M.Hatchya (Mitsui). 83-11 Ang. dist. was modified. 84-02 Comment was added. MF-1 General Information MT-451 Descriptive data and dictionary MF-2 Resonance Parameters MT-151 Resolved resonance parameters for MLBW formula Resonance energy region 1.0E 5 eV to 400 keV. Parameters were evaluated on the basis of the experimental data by Musgrove et al. 1. Calculated 2200-m/sec cross sections and resonance integrals 2200-m/sec Res.Integ. 1.230 b elastic 0.680 b 0.384 b capture 1.910 b total MF:3 Neutron Cross Sections Above resonance region. MT 1 Total cross section The optical model calculation with CASTHY /2/ was adopted. Optical potential parameters were taken from Ref. /3/, and modified so as to reproduce the Ca-42 total cross section 52.06 0.C23 E MeV . v Ws - 5.57 (MeV), Vso= 5.37 (MeV), r0 = rso = 1.26 (fm), rs = 1.39 (fm). a = aso = 0.76(fm), b = 0.40(fm). Elastic scattering cross section MT=2 Derived by subtracting partial cross sections from the total cross section. MT=4,51-56,91 Inelastic scattering cross sections Calculated with optical and statistical model code CASTHY /2 . Level schere Taken from Table of Isotopes /4/. (lo. Energy (MeV) Spin-Parity

g.s.	0.0	0 +
1	1.5246	2 +
2	1.8373	0 +
3	2.4236	2 .
4	2.7523	4 +
5	3.1893	6 +
6	3.445	3

Levels above 3.884 MeV were assumed to be overlapping.

Level density parameters (Gilbert and Cameron (5/)

isotope	42	43
a (1-MeV)	6.58	7.00
S C 1.SQRT(MeV))	2.75	2.88
Delta(MeV)	3.47	1.83
Ex MeV	9.54	7.82

- MT 16 n.2n cross section Based on available data.
- MT-102 Capture cross section Calculated with CASTHY /2/.
- MT-107 (n.alpha) cross section Statistical and pre-equilibrium model calculations using the optical potential parameters and the level density parameters given above. Fitted to available data.
- MT=251 Mu-bar Calculated with optical model.

MF=4 Angular Distributions of Secondary Neutrons MT=2.51-56.91

Optical model calculation

MT=16

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Isotropic in the laboratory system.

MF=5 Energy Distributions of Secondary Neutrons MT=16.91

Evaporation spectra.

- 1) Musgrove A.R.de L. et al.: Nucl. Phys. A279, 317 (1977).
- 2) Igarasi S.: J. Nucl. Sci. Theonol., 12, 67 (1975).
- 3) Fu C.Y.: Atomic Data and Nuclear Data Table 17, 127 (1976).
- 4) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed., Wiley-Interscience (1978).
- 5) Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965),

20-Ca 43 MAT number = 2203

20 Ca 43 Mitsui E.S.Eval Apr80 M.Hatchya Dist Mar83 Revi-Feb84

History

80-04 New evaluation was made by M.Hatchya (Mitsui). 83-11 Ang. dist. was modified.

84-02 Comment was added.

MF-1 General Information MT-451 Descriptive data and dictionary

MF-2 Resonance Parameters

MT-151 Resolved resonance parameters for MLBW formula Resonance energy region 1.0E 5 eV to 30 keV. Parameters were evaluated on the basis of the data of Musgrove et al. 1.

Calculated 2200 m sec cross sections and resonance integrals

elastic	2.997 1	
capture	6.200 b	3.20 b
total	9.197 b)

MF-3 Neutron Cross Sections

Below 30 keV, background cross sections were given to reproduce the thermal capture cross section of 6.2 barns \mathbb{Z}^2 and reasonable elastic scattering cross section. Above resonance region, data were evaluated as follows.

MT-1 Total cross section

The optical model calculation with CASTHY '3. was adopted.

Optical potential parameters were taken from Ref. 74%.

V = 46.72	(MeV).
Ws = 9.13	(MeV).
Vso= 5.37	(MeV),
r0 = rso = 1.26	(fm),
rs = 1.39	(fm),
a = aso= 0.76	(fm),
b = 0.40	(fm).

MI=2 Elastic scattering cross section Derived by subtracting partial cross sections from the total cross section.

 $\rm MT=4,51-54.91$ Inelastic scattering cross sections Calculated with optical and statistical model code CASTHY 3 .

Level scheme

Level scheme was taken from Table of Isotopes /5/.

2 of Calcium-43

No.	Energy (MeV)	Spin-Parity
g.s.	0,0	7/2
1	0.3728	5/2
2	0.5934	3/2
3	0.9903	3/2 +
4	1.3946	5/2 +

Levels above 1.9314 MeV were assumed to be overlapping.

Level density parameters (Gilbert and Cameron (6/)

isotope	43	44
a (1/MeV)	7.00	7.10
S C(1 SQRT(MeV))	2.88	2.95
Delta MeV	1.83	3.27
Ex MeV	7.82	9.18

MT 16 n.2n cross section Based on available data.

MT 102 Capture cross section Calculated with CASTHY /3/.

- MF 103,107 (n.p) and (n.alpha) cross sections Statistical and pre equilibrium model calculations using the optical potential parameters and the level density parameters given above. Fitted to available data.
- MT 251 Mu bar Calculated with optical model.
- MF-4 Angular Distributions of Secondary Neutrons
 - MT 2.51 54.91

Optical model calculation

MT-16

Isotropic in the laboratory system.

MF-5 Energy Distributions of Secondary Neutrons MT=16.91 Evaporation spectra.

- 1) Musgrove A.R.de L. et al.: Nucl. Phys. A279, 317 (1977).
- 2) Mughabghab S.F. and Garber D.I.: BNL-325, 3rd Edition, Vol.1 (1973).
- 3) Igarasi S.: J. Nucl. Sci. Theonol., 12, 67 (1975).
- 4) Fu C.Y.: Atomic Data and Nuclear Data Table 17, 127 (1976).
- 5) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed., Wiley-Interscience (1978).
- 6) Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).

1 of Calcium-44

20-Ca- 44 MAT number = 2204

20-Ca- 44 Mitsui E.S.Eval-Apr80 M.Hatchya Dist-Mar83 Rev: Feb84 History 80-04 New evaluation was made by M.Hatchya (Mitsui). 83-11 Ang. dist. was modified. 84-02 Comment was added.

MF-1 General Information MT=451 Descriptive data and dictionary

MF-2 Resonance Parameters

MT-151 Resolved resonance parameters for MLBW formula Resonance energy region 1.0E-5 eV to 400 keV. Parameters were evaluated on the basis of the data of Musgrove et al. 1.

Calculated 2200 m (see cross sections and resonance integrals 2200-m/sec Res.Integ. elastic 1.323 b

elastic	1.363	D		
capture	0.8799	b	0.429	b
total	2.203	b		

MF-3 Neutron Cross Sections

Below 400 keV, background cross sections were given to reproduce the thermal capture cross section of 0.88 barns $>2^{2}$. Above resonance region, data were evaluated as follows.

MT 1 Total cross section The optical model calculation with CASTHY 3 was adopted.

Optical potential parameters were taken from Ref. 747.

V 46.72	:MeV ,
Ws = 9.13	(MeV),
Vso- 5.37	(MeV),
r0 = rso = 1.26	(fm),
rs = 1.39	(fm),
a = aso= 0.76	(fm),
b = 0.40	(fm).

MT=2 Elastic scattering cross section Derived by subtracting partial cross sections from the total cross section. MT=4,51-54,91 Inelastic scattering cross sections

Calculated with optical and statistical model code CASTHY

Level scheme

Level scheme was taken from Table of Isotopes /5/.

No. Energy (MeV) Spin-Parity

g.s.	0.0	0 +
1	1.157	2 +
2	1.8835	0 +
з	2.2831	4 +
4	2.6565	2 +

Levels above 3.2849 MeV were assumed to be overlapping.

Level density parameters (Gilbert and Cameron (6/))

isotope	44	45
a 1 MeV)	7.10	7.12
S-C(1/SQRT(MeV))	2.95	3.00
Delta(MeV)	3.27	1.83
Ex MeV	9.18	7.66

- MT-16 n.2n cross section Based on available data.
- MT 102 Capture cross section Calculated with CASTHY 23/.
- MT-103.107 (n,p) and (n,alpha) cross sections Statistical and pre-equilibrium model calculations using the optical potential parameters and the level density parameters given above. Fitted to available data.
- MT-251 Mu-bar Calculated with optical model.

MF-4 Angular Distributions of Secondary Neutrons MT-2.51 54.91

- Optical model calculation
- MT 16 Isotropic in the laboratory system.

MF-5 Energy Distributions of Secondary Neutrons MT:16.91 Evaporation spectra.

- 1) Musgrove A.R.de L. et al.: Nucl. Phys. A279, 317 (1977).
- 2) Mughabghab S.F. and Garber D.I.: BNL-325, 3rd Edition, Vol.1 (1973).
- 3) Igarasi S.: J. Nucl. Sci. Theonol., 12, 67 (1975).
- 4) Fu C.Y.: Atomic Data and Nuclear Data Table 17, 127 (1976).
- 5) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed., Wiley-Interscience (1978).
- 6) Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).

1 of Calcium-46

20 -Ca - 46 MAT number ~ 2205 20 Ca 46 Mitsui E.S.Eval-Apr80 M.Hatchya Dist-Mar83 Rev1-Feb84 History 80-04 New evaluation was made by M.Hatchya (Mitsui). 83-11 Ang. dist. was modified. 84-02 Comment was added. MF-1 General Information MT=451 Descriptive data and dictionary MF-2 Resonance Parameters MT 151 No resonance parameters 2200 m sec cross sections and calculated resonance integrals 2200 m sec Res. Integ. 2.900 b elastic 0.7400 b 0.339 b capture total **3.640** b MF 3 Neutron Cross Sections Thermal region was assumed to be below 1.0 keV. The capture and elastic scattering cross sections were assumed to be 0.74 barns 1 and 2.9 barns at 0.0253 eV, respectively. The total cross section was calculated as a sum of these two. Above 1.0 keV. data were evaluated as follows. MT 1 Total cross section The optical model calculation with CASTHY 2 was adopted. Optical potential parameters were taken from Ref. 3/. V 46.72 MeV . Ws 9.13 MeV . Vso- 5.37 rs = 1.39 (**fm**). a = aso= 0.76 (**fm**). b ≈ 0.40 (fm). MT=2 Elastic scattering cross section Derived by subtracting partial cross sections from the total cross section. MT=4.51-53.91 Inelastic scattering cross sections Calculated with optical and statistical model code CASTHY ·2 . Level scheme Level scheme was taken from Table of Isotopes /4/. No. Energy (MeV) Spin-Parity

- 50 -

0 ÷

0.0

g.s.

	1 2 3	1.347 3.024 3.613	, 2 2 3	-1. +	
	Levels abo	ove 4.463 I	1eV were	assumed to be	overlapping.
	Level density	parameters	Gilber	t and Cameron	2 5 70)
	isotope	46	47		
	a (1.MeV) S-C(1.SQRT(Me Delta(MeV) Ex (MeV)	7.135 9) 3.03 3.37 9.131	7.075 3.08 1.83 7.522		
MT 1 MT 1 MT 2 MF 4 MT 2 MT 1 MF 5 MT 1	6 n.2n cros Based on availa 02 Capture cro Calculated with 03.107 n.p. a Statistical and the optical pot parameters give 51 mu bar Calculated with Angular Distribu 51 53.91 Optical model ca 6 Isotropic in the Energy Distribut 5.91	s section ble data. ss section CASTHY 2 nd n.alph pre equil ential par n above. optical m utions of alculation e laborato tions of S stra.	a: cross ibrium m ameters Fitted t odel. Secondar ry system econdary	sections odel calculatio and the level o available dat y Neutrons ». Neutrons	ons using density ta.
Referen 1 : Mu 2) Iga 3) Fu 4) Leo Wi	nces ghabghab S.F. et PB1:, arasi S.: J. Nuc C.Y.: Atomic Da derer C.M. and S leve-Interscience	al.: Neu al. Sci. Th ta and Nuc Shirley V.S (1978)	tron Cros necnol., clear Dat S.: Table	s Sections. Vo 12, 67 (1975). a Table 17, 12 e of Isotopes.	01. 1, Part A 77 (1976). 7th Ed.,

5) Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).

20 Ca 48 MAT number - 2206

20 Ca 48 Mitsui E.S.Eval Apr80 M.Hatchya Dist Mar83 Rev1 Feb84 History 80 04 New evaluation was made by M.Hatchva (Mitsur). 83-11 Ang. dist. was modified. 84 02 Comment was added. MF-1 General Information MT 451 Descriptive data and dictionary MF-2 Resonance Parameters MT 151 Resolved resonance parameters for MLBW formula Resonance energy region 1.0E 5 eV to 400 keV. No s wave resonances were given. P wave resonance pareme ters were evaluated on the basis of BNL 325 3rd ed. 71., Calculated 2200 m sec cross sections and resonance integrals 2200 m/sec Res. Integ. elastic 2,900 b 1.090 Ъ 0.492 b capture 3.990 Б total MF 3 Neutron Cross Sections Below 400 keV, background cross sections were given. Above resonance region, data were evaluated as follows. MT 1 Total cross section The optical model calculation with CASIHY 2 was adopted. Optical potential parameters were taken from Ref. 31, ... V 49.72 MeV . MeV . Ws. -9.13Vso 5.37 (MeV). $r0 \approx rso \approx 1.26$ (**fm**). rs - 1.39 (fm). $a \approx aso \approx 0.76$ (fm), b = 0.40 (Cm). Elastic scattering cross section MT-2

M-2 Elastic scattering cross section
 Derived by subtracting partial cross sections from the total cross section.
 MT=4.51-55.91 Inelastic scattering cross sections

Calculated with optical and statistical model code CASTHY 2

Level scheare

Level scheme was taken from Table of Isotopes /4/.

No. Energy(MeV) Spin-Parity g.s. 0.0 0 +

1	3.8317	2 +
2	4.503	4 1
3	4,5069	3
4	4.6119	3 +
5	5.3696	3
	L O BLOP M	

Levels above 6.6137 MeV were assumed to be overlapping.

Level density parameters (Gilbert and Cameron (5/)

isotope	48	49
a (1.MeV)	6.84	7.20
S.C.(1.SQRT(MeV))	3.07	3.19
Delta(MeV)	3.13	1.83
Ex. MeV	8.76	7.39

- MT 16 n.2n cross section Based on available data.
- MT 102 Capture cross section Calculated with CASTHY 2.
- MT 251 Mu bar Calculated with optical model.
- MF-4 Angular Distributions of Secondary Neutrons MT-2.51-55.91 Optical model calculation
 - Optical model calculation
 - MT 16

Isotropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons MT 16.91

Evaporation spectra.

- 1) Mughabghab S.F. and Garber D.I.: BNL 325, 3rd Edition, Vol.1 (1973).
- 2) Igarasi S.: J. Nucl. Sci. Theonol., 12, 67 (1975).
- 3) Fu C.Y.: Atomic Data and Nuclear Data Table 17. 127 (1976).
- 4) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed., Wiley-Interscience (1978).
- 5 Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).

1 of Scandium 45

21-Sc- 45 MAT number = 2211 - - ----21 Sc. 45 UTOK, JAERI Eval Feb82 Y. Oka, T. Nakagawa, Y. Kikuchi JAERI M 9981 Dist Mar83 Rev1 Nov83 History 82 02 Evaluation was maded by Y.Oka (Tokyo University). T.Nakagawa and Y.Kikuchi (JAERI). Details are given in Ref . 1 83 11 Angular dist, was modified and comment was added. MF-2.MT-151 Resonance parameters for MLBW formula Resolved resonance regin : 1.0E 5 eV - 90 keV On the basis of the data of Liou- 2 and Kenny- 3. Adjusted to reproduce 1 total cross section minimum at 2 keV : 0.23 b 4. thermal cross sections of BNL 325.3rd ed. Calculated thermal cross sections and resonance integrals: 2200 m/s value res.int. total 51.05 b elastic 25.03 b 26.02 b 11.3 b capture MF 3 Cross Sections MT 1.2.4.51 61.91.102 Sig t.Sig el.Sig in.Sig c Calculated with optical and statistical models. Optical potential parameters were obtained by fitting the data of Foster+ 5 and Barnard+ 6 : v 56,15 0.2189 En Ws 8,698 .Vso 6.874 (MeV) (fm) r 1.16 .rs 1.288 .rso 1.185 .aso 0.76 a 0.677 0.310 (fm) , b Statistical model calculation with CASTHY code .74. Competing processes : n.p., n.alpha . n.2n). Level fluctuation considered. The level scheme taken from Ref. 8 : No Energy (MeV) Spin-Parity g.s. 0 7/2 -3/2 + 0.0124 1 2 0.3764 3/2 --З 5/2 + 0.5429 5/2 + 4 0.7202 5 0.9391 1/2 +6 0.9745 7/2 + 7 1.2364 11/2 -8 1.3032 3/2 + 9 1.4334 9/2 + 10 1.6615 92 -1.8006 5.2 + 11 Continuum levels assumed above 1.9 MeV. The level density parameters : Gilbert and Cameron /9/. The gamma-ray strength function was determined so that Sig-c = 32 mb at 100 keV / 3/.

MT-16 (n.2n) Evaluated on the basis of the data of Holube 10 . MT-103.107 (n.p). (n.alpha) Taken from compilation by Alley and Lessler 7117. MT 251 Mu bar Calculated with optical model. MF 4 Angular Distributions of Secondary Neutrons Calculated with optical model. S TM MT-16 Isotropic in laboratory system. MT-51-61 90 deg. symmetry in center of mass system. MT-91 90 deg. symmetry in laboratory system. MF 5 Energy Distributions of Secondary Neutrons MT 16.91 Evaporation spectrum. References 1 Oka Y., Nakagawa T., Kikuchi Y. : JAERI M9981 (1982). 2 Liou H.I. et al. : Nucl. Sci. Eng. .67.326 (1980). 3 Kenny M.J. et al. : Aust. J. Phys. 30,605 (1977). 4) Fujita Y. : J. Nucl. Sci. Technol., 20, 191 (1983). 5 Foster G.D. Glasgow D.W. : Phys. Rev. C3,576 (1971). 6) Barnard E. et al. : Z Phys., 245,36 (1971), 7) Igarasi S. : J. Nuct. Sci. Technol., 12,67 (1975). 8 Lederer C.M. Shirley V.S. : Table of Isotopes .7th Ed. (1978). 9) Gilbert A. Cameron A.G.W. : Can. J. Phys. 43, 1446 (1965).

- 10) Holub E., Cindro N. ; Z. Phys. A289.421 (1979).
- 11) Alley W.E. Lessler R.M. : Nucl. Data Tables. A11.648 (1973).

23 V - 51 MAT number - 2231

23 V - 51 JAERI Eval Oct82 S. Tanaka

JAERI - M 82-151 Dist Mar83 Revi Feb84

History

- 82-10 Evaluation was made by S. Tanaka (JAERI). Details are given in Ref. 31.
- 83-11 Q value and threshold energy of 2nd level was corrected. Comment data were added.
- 84 02 Background cross sections were replaced with correct data.
- MF-1 General Information
 - MT-451 Descriptive data and dictionary

MF 2. MT 151 Resonance parameters

Resolved resonances for MLBW formula : 1.0E 5 eV 100 keV Parameters were taken from BNL 325 3rd edition /2/, and modified to reproduce experimental total cross section. Cross sections calculated with these parameters are to be corrected by adding MF 3, MT 1. 2 and 102 data.

Calculated 2200 m sec cross sections and resonance integrals 2200 m sec Res. Integ.

elastic	4.805 0	
capture	4.900 b	2.53 b
total	9.705 b	

MF-3 Neutron Cross Sections

Below 100 keV.

Background cross sections are given.

500 keV. 100

MT 1 Total

Based on the following experimental data.

- 220 keV. data of Rohr and Friedland 3. 100
 - 220 360 keV, data of Smith et al. 4/ with energy shift.
- 360 500 keV, data of Cierjacks et al. $\sqrt{5}/$.
- Elastic scattering MT 2

Obtained by subtracting the sum of MT=102 and 51 from MT=1.

MT=51 Inelastic scattering to the 1st level

Hauser-Feshbach calculation mentioned below.

MT=102 capture

Based on the following experimental data.

100 - 210 keV, data of Winter et al. .67. 210 - 500 keV, data of Dudey et al. .77.

Above 0.5 MeV.

MT=1.2.4.51-76.91.102 Total, Elastic, Inelastic and Capture Calculated with optical and statistical models except for MT=51 and 52 data in 4.5 - 20 MeV, which follow the data of Perey and Kinney /8/ and extrapolation of their data above 8.5 MeV. The effect of this exception and threshold reactions are reflected in MT=2 data.

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The spherical	optical potent	ial parameters:
V = 49.50 -	0.33+En , Vso	= 7.0 (MeV)
₩s ∾ 4,6 + 0.	.344En . Wv	O (MeV)
r 🖷 rs 🗇 rsc) ⊨ 1.23	(fm)
a – aso – 0,	.65 . b	⇒ 0.48 (£m.),
Statistical mode	el calculation	with CASTHY code /9/ was
perfomed, MT=10	12 data were no:	rmalized to the
experimental dat	a of Dudey et i	al. 7 at 0.5 MeV.
The level sche	me taken from [Ref. 10 :
No	Energy MeV	Spin Parity
2.S.	0	72
3	0.320	52
2	0.929	32
3	1 609	11.2
á	1.813	9.2
5	2 109	32
Č	2 545	12
7	2.075	3 3
, L	2.000 S.600	16.9
с О	2.088	15 2
10	2.750	592
10	0.004	
11	3.190	
12	3.210	32
13	3.262	52
14	3.230	52(1)
15	3,381	32
16	3.383	9.2
17	3,386	13.2
18	3,396	13.2
19	3.412	· 92 · · ·
20	3.452	92
21	3.515	13 2
22	3.569	13.2
23	3.576	32
24	3,614	11 2
25	3.631	12
26	3.674	3.2
arbitraril	y assigned.	
Continuum levels	assumed above a	3.68 MeV. The level
density parameter	's of Dilg et a	1. /11/ were used.
MT=16 (n.2n)		
Guided by experim	ental data of 1	Frehaut et al. /12/.
MT=22 (n.n'alpha)		
Based on the asse	rtion of Hillma	an /13/.
MT=28 (n.n')		
Given by subtract	iong the (n,p)	cross section (MT=103)
from the (n,xp) c	ross section ca	alculated by Kitazawa
and Isogai 14		•
MT=103 (n.p.)		
Kitazawa and Isog	ai's calculatio	on 14 normalized to
Bormann's experim	ental data 15	at 13.2 MeV.
MT=104 (n,d)		
Calculation of Gu	erther et al	16 .
MT=105 (n.t.)		••, •
Roughly the same :	shape as the (r	(d) cross section was
adopted by normal	izing to 0.001	barns at 14 MeV
adopted by normal		build the it int.

MT-107 (n.alpha) Follows experimental data of Paulsen et al. 177. MT 251 Mu-ber Calculated with optical model.

- MF 4 Angular Distributions of Secondary Neutrons S TM Calculated with optical model. MI 51 76,91 Calculated with Hauser-Feshbach formula. MT-16.22.28 Isotropic in the laboratory system.
- MF=5 Energy Distributions of Secondary Neutrons MT 16.22,28.91 Evaporation spectra.

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24-Cr- 0 MAT number = 2240

24-Cr- 0 JAERI Eval-Aug82 T.Asami Dist-Mar83 Rev1 Nov83 History 82-08 New evaluation was made by T. Asami (JAERI) for JENDL 2. 83-11 Modification for MF-4. MF=1 General Information MT=451 Descriptive data and dictionary MF-2 Resonance Parameters MT-151 Resolved resonance parameters for MLBW formula Resolved resonance region : 1 OE 5 eV to 300 keV for Cr 50. Cr 52 and Cr 54. 1.0E 5 eV to 120 keV for Cr 53. Resonance parameters were evaluated from the following data: Cr-50 : Stieglitz(71 1, Beer(74 2, Allen(77 3 and Kenny+77 4 Cr 52 : same as above. Cr 53 : same as above and Mueller (71 5 . Cr.54 : Stieglitz(71 /1/, BNL/325 /6/, Allen(77 /3/ and Kenny+77 /4 . For unknown radiative widths, assumed were average values of known widths. Effective scattering radii of Cr-50, Cr-53 and Cr 54 were taken from BNL 325 6 and radius of Cr-52 was detrmined to get a good fit to the experimental total cross sections. calculated 2200 m s cross sections and res. integrals. 2200 m s res. integ. elastic З.830 Б 3.070 b 1.60 b capture 6.900 b total MF-3 Neutron Cross Sections Below 300 keV. Background cross sections were applied to reproduce the 2200-m/s capture cross section of 3.07 barns and elastic scattering of 3.8 barns. Contribution from Cr-53 in the energy range from 120 to 300 keV were also considered as background cross sections. Above 300 keV. MT=1 Total Cross sections in the energy range of 300 keV to 7 MeV were estimated from the experimental data of Cierjacks+68 /7/. the data above 7 MeV were calculated by using optical model code CASTHY 8. Potential parameters 9 were obtained by fitting average total cross section of natural chromium. V = 50.05 - 0.262 + E, Ws = 4.87 + 0.352 + E, Vso = 7.0 (MeV) r0=1.24, rs = 1.4rso = 1.24(fm)

, b 0.4

2 of Natural Chromium

a0 0.48

, aso 0.48∈fm⇒

MT 2 Elastic scattering

Total - All other partial cross sections

MI 4.51 90.91 Inelastic scattering

Calculated with statistical and optical model code CASTHY 9/ for four stable isotopes, and constructed taking into account of their isotope abundances. In the calculation, optical potential parameters given above and level density parameters by Yoshida 10 were used. Level schemes were taken from Ref. 11. Level energies and corresponding isotopes are as follows:

Energy MeV	Isotope	MT	Energy (MeV)	Isotope
0.5638	53	71	2.7071	53
0.7839	50	72	2,7678	52
0.8343	54	73	2.7711	53
1.0060	53	74	2.6256	53
1 2891	53	75	2.8275	54
1.4341	192 - E	76	2.9268	50
1.5361	53	77	2.9649	52
1.8225	54	78	2,9920	53
1.8829	50	79	3.0718	54
1.9729	53	-80	3.0831	53
2.1717	53	-81	3.0920	53
2.2332	53	82	3.1139	52
2.3200	53	83	3,1369	53
2.3697	52	84	3,1578	54
2.4523	53	85	3.1618	52
2.6177	54	86	3,1636	50
2.6471	52	87	3,1666	50
2.6561	53	88	3.1782	53
2.6686	53	69	3,2203	54
2.7056	53	90	3.2428	53
	Energy MeV 0.5638 0.7839 0.8343 1.0050 1.2991 1.4341 1.5361 1.8225 1.6829 1.9729 2.1717 2.2832 2.3200 2.3697 2.4523 2.6177 2.6561 2.6565 2.7056	Emergy MeV Isotope 0.5638 53 0.7839 50 0.8343 54 1.0030 53 1.4341 52 1.4341 52 1.4341 52 1.8225 54 1.8829 50 2.1717 53 2.3200 53 2.3200 53 2.3697 52 2.4523 53 2.6177 54 2.6686 53 2.6686 53 2.6686 53 2.7056 53	Emergy MeV Isotope MI 0.5638 53 71 0.7839 50 72 0.8343 54 73 1.0030 53 74 1.2691 59 75 1.4341 52 76 1.5861 53 74 1.8225 54 78 1.8829 50 79 1.9729 53 80 2.1717 53 81 2.832 53 82 2.3200 53 83 2.3697 52 84 2.4523 53 85 2.6177 54 86 2.6631 53 88 2.6635 53 69 2.7056 53 90	Energy MeV Isotope MI Energy (MeV) 0.5638 53 71 2.7071 0.7839 50 72 2.7678 0.8343 54 73 2.7711 1.0050 53 74 2.6256 1.2901 53 75 2.8275 1.4341 52 76 2.9268 1.5361 53 77 2.9649 1.8225 54 78 2.9920 1.8829 50 79 3.0718 1.9729 53 80 3.0831 2.1717 53 81 3.0920 2.832 53 82 3.1139 2.3200 53 83 3.1369 2.3697 52 84 3.1578 2.4523 53 85 3.1618 2.6177 54 86 3.1636 2.6265 53 89 3.2203 2.66561 53 89 3.2203 </td

Contributions from the levels above 3,260 MeV were put together into continuum MT 91.

MT 16 n.2n

Constructed from the evaluated .n.2n. data for CR 50, 52, 53 and 54. Data for Cr 50 and Cr 52 were estimated from the experimental data. Data for Cr 53 and 54 were obtained from calculations with evaporation-model code GROGI /12/, and adjusted to the experimental data of Frehaut+80 /13/ for natural chrominm in the energy range below 15 MeV.

MT=28 (n,n'p)

Constructed from the evaluated (n,n'p) data for Cr-50, 52, 53 and 54 which were calculated with evaporation-model code GROGI 12, and normalized to the experimental data.

MT=102 Capture

Calculated with statistical and optical model code CASTHY/8/. and normalized to reproduce 10 milli-barns at 50 keV. MT=103 - (n, n)

Constructed from the evaluated on.p data for Cr-50, 52, 53 and 54 which were calculated with evaporation-model code GROGI 12, and normalized to the experimental data.

MT=107 (n.alpha)

Constructed from the evaluated (n.aLpha) data for Cr-50, 52, 53 and 54 which were calculated with evaporation-model code

3 of Natural Chromium

GROGI /12/, and normalized to the experimental data. Data for Cr-52 were modified in the energies below 10 MeV to repoduce the experimental data of Paulsen-80 4 for natural chromium

MI 251 Mu bar

Calculated with optical model.

MF 4 Angular Distributions of Secondary Neutrons

- S TM Calculated with optical model.
- MT 51 90 Isotoropic in the center of mass system.
- MT 16,28,91 Isotoropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

MT 16.28.91 Evaporation spectra were given.

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24-Cr 50 MAT number = 2241

24 Cr 50 JAERI Evol Aug82 T.Asami Dist Mar83 Rev1 Nov83

History

82,08 New evaluation was made by T. Asami (JAERL) for JENDL 2. B3-11 Modification for MF-4.

MF-1 General Information MT-451 Descriptive Data and Dictionary

MF-2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW Resolved resonance region : 1.0E 5 eV 300 keV Evaluation based on the experimental data by Stieglitz+71 1. Beer 74 2. Allen:77 3 and Kenny:77 4. assumed gamma width 1.81 eV for s wave and 0.92 eV for p wave resonances. Effective scattering radius 5.4 fm 5.

Calculated 2200 m s cross sections and res. integrals. 2200 m s res. integ. elastic 8.581 b capture 15.90 b 7.77 b

total 24.48 b

MF 3 Neutron Cross Sections

Resonance region from 1.0E 5 eV to 300 keV Background cross sections were applied to reproduce the 2200 m s capture cross section of 15.9 - 0.2 barns 5, and to modify the elastic scattering cross section in the lower energy region.

Above 300 keV

MT-1 Total

Calculated with optical model. Potential parameters $\frac{6}{}$ were obtained by fitting Cr natural average total cross section. V = 50.05 = 0.262 F. Ws: 4.87 ± 0.352 F. Vsc= 7.0 (MeV)

	00.00	0.202.2	10	4.01		100	1.0 (1)01 /
r0=	1.24		rs≕	1.4		rso=	1.24(fm)
a0=	0.48	٦	b ==	0.4	•	aso=	0.48(fm)

MT=2 Elastic scattering

(Total - (All other partial cross sections -

MT=4.51-70.91 Inelastic scattering

Calculated with statistical and optical model code CASTHY/7/.

Level scheme taken from Ref. 8. No. Energy MeV Spint-Parity g.s. 0.0 0 + 1 0.7833 2 + 2 1.8814 4 +

 Continuum levels assumed above 4.066 MeV. Level density parameters of Yoshida 9 were used. MT 16 n.2n. Evaluated mainly on the basis of the experimental data of Bormann25 10. MT 28 n.n'p: Calculated with evaporation model code GROGI /11/, and normalized to 0.393 barns at 14.7 MeV which was estimated from the experimental data of Qaim+82 12 and Grimes+79/13/. MT 102 Capture Calculated with statistical and optical model code CASTHY/7/. MT 103 n.p: Calculated with evaporation model code GROGI /11/, and normalized to 0.437 barns at 14.7 MeV which was estimated based on the experimental data of Grimes-79 13. MT 107 n.alpha Calculated with evaporation model code GROGI /11 , and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes-79 13 and Dolja+73 .14. MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=16.28 Assumed to be isotropic in the lab system. MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfX 2053 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20	2.9245 3.1611 3.1641 3.3247 3.5946 3.6940 3.6978 3.6978 3.6978 3.6978 3.8261 3.8261 3.8261 3.8261 3.8261 3.8263 3.8752 3.8953 3.8953 3.8983 3.8983 3.9377 4.0517	2 + 1 + + + + + + + + + + + + + + + + +	
 MT 16 - n.2n.⁵ Evaluated mainly on the basis of the experimental data of Bormann65 10. MT 28 - n.n⁵p. Calculated with evaporation model code GROGI /11/, and normalized to 0.393 barns at 14.7 MeV which was estimated from the experimental data of Qaim(82 12 and Grimes+79/13/. MT 102 Capture Calculated with statistical and optical model code CASTHY/7/. MT 103 n.p. Calculated with evaporation model code GROGI _11/, and normalized to 0.437 barns at 14.7 MeV which was estimated based on the experimental data of Grimes 79 13. MT 107 n.alpha Calculated with evaporation model code GROGI _11 . and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes 79 13 and Dolja 73 .14 MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=6.28 Assumed to the lab system. MT=16.28 Assumed to be isotropic in the lab system. MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfX 2053 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgreve A.R.De L.: Neutron Data of Structural 	Contin Level de	nuum levels assu	med above 4.0 s of Yoshida	66 MeV. 9 were used
 Evaluated mainly on the basis of the experimental data of Bormann65 10. MT 28 — n.n'p. Calculated with evaporation model code GROGI /11/, and normalized to 0.333 barns at 14.7 MeV which was estimated from the experimental data of Qaim(82 12 and Grimes+79/13/. MT 102 Capture Calculated with statistical and optical model code CASTHY/7/. MT 103 n.p. Calculated with evaporation model code GROGI /11/, and normalized to 0.437 barns at 14.7 MeV which was estimated based on the experimental data of Grimes-79 13. MT 107 n.alpha Calculated with evaporation model code GROGI 11 , and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes-79 13 and Dolja(73 .14 MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=91 Calculated with CASTHY /7/. MT=16.28 Assumed to be isotropic in the lab system. MT=16.28.91 Evaporation spectra were given. References 1) Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). 2) Beer H. and Spencer R.P.: KfX 2033 (1974), also Nucl. Phys. A240, 29 (1975). 3) Allen B.J. and Musgreve A.R.De L.: Neutron Data of Structural 	MT 16 n.	2n		
 MT 102 Capture Calculated with statistical and optical model code CASTHY/7/. MT 103 n.p. Calculated with evaporation model code GROGI [117], and normalized to 0.437 barns at 14.7 MeV which was estimated based on the experimental data of Grimes-79 13. MT 107 n.alpha Calculated with evaporation model code GROGI 11. and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes-79 13 and Dolja+73 [14]. MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=9i Calculated with CASTHY /7/. MT=16.28 Assumed to be isotropic in the lab system. MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	Evaluate Bormann6 MT 28 n. Calculat normaliz from the	d mainly on the 5 10 . n'p+ ed with evapora ed to 0.393 bar experimental d	basis of the tion model council ns at 14.7 Me ³ ata of Qaim+84	experimental data of de GROGI /11/, and / which was estimated 2 12 and Grimes+79/13/.
 MT 103 n.p. Calculated with evaporation model code GROGI [117], and normalized to 0.437 barns at 14.7 MeV which was estimated based on the experimental data of Grimes 79 13. MT 107 n.alpha Calculated with evaporation model code GROGI 11. and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes 79 13 and Dolja 73 .14 MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2,51-70 Calculated with CASTHY /7/. MT=91 Calculated with CASTHY /7/. MT=16.28 Assumed to be isotropic in the lab system. MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	MI 102 Cap	ture od with statist	ical and optic	al model and CASTRY /7/
 normalized to 0.437 barns at 14.7 MeV which was estimated based on the experimental data of Grimes-79 13. MT 107 in alpha Calculated with evaporation model code GROGI 11 , and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes-79 13 and Dolja+73 .14 MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=91 Calculated with CASTHY /7/. MT=16.28 Assumed to be isotropic in the lab system. MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 Evaporation spectra were given. References 1.) Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). 2) Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). 3) Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	MT 103 n. Calculat	ed with statist p ed with evapora	tion model cod	de GROGI /11/, and
 MI 107 n.alpha Calculated with evaporation model code GROGI 11 . and normalized to 0.108 barns at 14.8 MeV which is an average value of the experimental data of Grimes.79 13 and Dolja+73 .14 MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=91 Calculated with CASTHY /7/. MT=91 Calculated with CASTHY /7/. MT=16.28 Assumed to be isotropic in the lab system. MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	normaliz based on	ed to 0.437 bari the experimenta	ns at 14.7 MeV al data of Gra	/ which was estimated mes-79 13 .
 MT-251 Mu bar Calculated with optical model MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=9i Calculated with CASTHY /7/. MT=9i Calculated with CASTHY, and the same distribution was assumed in the lab system. MT=16.28 Assumed to be isotropic in the lab system. MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK-2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	MI 107 n. Calculate normalize Value of 14	alpha ed with evaporat ed to 0.108 barn the experimenta	tion model coo ns at 14.8 MeV al data of Gri	le GROGI 11 , and 'which is an average mes:79 13 and Dolja+ 73
 MF=4 Angular Distributions of Secondary Neutrons MT=2.51-70 Calculated with CASTHY /7/. MT=9i Calculated with CASTHY /7/. MT=9i Calculated with CASTHY and the same distribution was assumed in the lab system. MT=16.28 Assumed to be isotropic in the lab system. MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	MT-251 Mu I Calculate	bar ed with optical	model	
 MF=16.28 Assumed to be isotropic in the Tab system. MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	MF=4 Angular MT=2,51-70 MT=91	Distributions of Calculated with Calculated with was assumed in	of Secondary N CASTHY /7/. CASTHY, and the lab syste	eutrons the same distribution m.
 MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 Evaporation spectra were given. References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	11-10,20	Assumed to be 1	Sociopic in t	ne lad system.
 References Stieglitz R.G. et al.: Nucl. Phys. A163, 592 (1971). Beer H. and Spencer R.P.: KfK 2063 (1974), also Nucl. Phys. A240, 29 (1975). Allen B.J. and Musgrove A.R.De L.: Neutron Data of Structural 	MF=5 Energy [MT=16.28.91)istributions of Evaporation spe	Secondary Ne ctra were giv	utrons en.
	References 1) Stieglitz 2) Beer H. an A240, 29 (3) Allen B.J.	R.G. et al.: Nu d Spencer R.P.: 1975). and Musgrove A	cl. Phys. A16 KfK-2063 19 R.De L.: Neu	3, 592 (1971). 74), also Nucl. Phys. tron_Data of Structural
(1979).

- 4) Kenny M.J. et al.: AAEC/E-400 (1977).
- 5) Mughabghab S.F. and Garber D.L.: BNL-325 3rd Ed., Vol.1 (1973)
- 6) Kawai M.: unpublished.
- 7) Igarasi S.; J. Nucl. Sci. Theonol., 12, 67 (1975).
 8) Lederer C.M. and Shirley V.S.; Table of Isotopes, 7th Ed., (1978).
- 9) Yoshida T.: unpublished.
- 10) Bormann M.: data in EXFOR file (1965).
- 11) Gilat J.: BNL-50246(T-580) (1970).
- 12) Qaim S.M. et al.: Nucl. Phys., A382, 255 (1982).
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- 14) Dolja G.P. et al.: 1973 Kiev Conf., Vol.3, 131 (1973).

24-Cr- 52 MAT number = 2242

24 Cr 52 JAERI Eval-Aug82 T.Asami Dist-Mar83 Rev1 Nov83

History

82-08 New evaluation was made by T. Asami (JAERI) for JENDL-2. 83-11 Modification for MF-4.

- MF=1 General Information MT=451 Descriptive data and dictionary
- MF-2 Resonance Parameters
 - MT 151 Resolved resonance parameters for MLBW formula Resolved resonance region : 1.0E 5 eV - 300 keV Evaluation based on the experimental data by Stieglitz+71 1 . Beer 74 2 . Allen 77 3 and Kenny 77 4 . Assumed gamma width = 1.86 eV for s wave and 0.57 eV for p wave resonances. Effective scattering radius = 5.1 fm.

Calculated 2200 m/s cross sections and res. integrals.

	2200 -m∕s	res. integ.
elastic	2. 223 b	5.5
capture	0.7599b	0.493 b
total	2.983 b	

MF-3 Neutron Cross Sections

Resonance region (from 1.0E 5 eV to 300 keV)

Background cross sections were applied to reproduce the 2200-m s capture cross section of 0.76 \cdots 0.06 barns 757, and to modify the elastic scattering cross section in the lower energy region.

Above 300 keV

MT-1 Total

Calculated with optical model. Potential parameters /6/ were obtained by fitting Cr-natural average total cross section.

V 🕾	50.05	- 0.262+E,	₩s≕ 4.87	+ 0.352+E,	Vso≕	7.0 (MeV)
r0∷	1.24		rs≃ 1.4		rso≃	1.24(fm)
a0=	0.48	,	b = 0.4		aso=	0.48(fm)

MT=2 Elastic scattering

(Total)= (All other partial cross sections)

MT=4.51-73.91 Inelastic scattering

Calculated with statistical and optical model code CASTHY/7/. Calculated values for MT=51 and 52 were modified using the experimental ones of Kinney+74 21/ in the energies above 5.0 MeV.

Level scheme taken from Ref. /8/. No. Energy(MeV) Spint-Parity

5.3 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 Conti	s. 0.0 1.4341 2.3696 2.6470 2.7677 2.9648 3.1138 3.1617 3.4152 3.4152 3.4152 3.4722 3.6158 3.7000 3.7717 3.9460 3.9512 4.0154 4.0380 4.58500 4.58500 4.58500 4.58500 4.58500 4.58500 4.58500 4.58500 4.58500 4.585000 4.585000 4.585000 4.58500000000000000000000000000000000000	0 + 2 + 4 + 0 + 4 + 2 + 6 + 3 + 5 + 2 + 4 + 1 - 5 + 2 + 4 + 1 - 5 + 2 + 4 + 4 + 2 + 6 + 4 + 2 + 6 + 1 - 1 - 5 + 2 + 4 + 2 + 6 + 1 - 1 - 1 - 1 - 1 - 1 - 1 - 1 -
Level d	ensity parameters	s of Yoshida /9/ were u sed .
Evaluati	.∠n) ed mainly on the	basis of the experimental data of
Wenusch	62 10 Bormann	168 /11/, Maslov+72 /12/, Qaim72/13/,
Sailer (77 14 and Molla	.81 15 .
Mi-28 n Calculat	a p) ted with evaporat	ion model code (ROOI 16: and
normali;	zed to 0.085 barn	as at 14.8 MeV which was estimated
from the	e experimental da	ta of Grimes 79 17 .
MT 102 Car	oture tog with statisti	and and optional model code CASTHY 7
and norr	alized to reprod	uce 10 milli barns at 50 keV for
Cr natu	ral	
MT=103 (n.	(p)	in madel and OPDOT (10 (and
Calculat	ed with evaporation evaporation and the constant of the second second second second second second second second	ion-model code GNUGI /16/, and s at 14 7 MeV which is an average
value of	the experimenta	1 data. Calculated data below 9 MeV
were mod	lified using the	experimental data of Smith+79 /18/.
MI=107 (n, Calculat	alpha) ed with evanorat	ion-model code CROGI /16/
Calculat	ed values were n	ormalized to 0.038 barns at 14.8 MeV
which is	an average valu	e of the experimental data of
Grimes+/	9 /17/ and Dolja rimental values (+73 /19/, and modified to reproduce
in the e	nergy region belo	ow 10 MeV.
MT=251 Mu-	bar	
Calculat	ed with optical i	nodel.
MF=4 Angular	Distributions of	f Secondary Neutrons
MT=2.51-70	Calculated with	CASTHY /7/.
MT=91	Calculated with	CASIHY, and the same distribution

was assumed in the lab system. MT=16.28 Assumed to be isotropic in the lab system.

MF=5 Energy Distributions of Secondary Neutrons MT=16,28,91 evaporation spectra were given.

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24-Cr- 53 MAT number = 2243

24 Cr. 53 JAREI Eval-Aug82 T.Asami Dist Mar83 Reviewov83

History

82,08 New evaluation was made by T. Asami (JAERI) for JENDL-2, 83,11 Modification for MF-4.

MF-1 General Information MT-451 Descriptive data and dictionary

MF-2 Resonance Parameters

MT-151 Resolved resonance parameters for MLBW formula Resolved resonance region : 1.0E 5 eV 120 keV Evaluation mainly based on the experimental data by Stieglitz -71 1 . Peer 74 2 . Allen 77 3 . Kenny 77 4 and Mueller 71 5 . Assumed gamma width 2.21 eV for s wave and 0.61 eV for p wave resonances. Effective scattering radius - 6.9 fm 767.
Calculated 2200 m s cross sections and res. integrals. 2200 m/s res. integr.

	CEUVINS	res, integ,
elastic	16.31 b	
capture	18.20 b	8.86 5
total	34.51 b	

MF-3 Neutron Cross Sections

Resonance region from 1.0E 5 eV to 120 keV -Background cross sections were applied to reproduce the 2200 m s capture cross section of 18.2 - 1.5 barns /6/. and to modify the elastic scattering cross section in the lower energy region.

Above 120 keV MT 1 Total Calculated with optical model. Potential parameters /7/ were obtained by fitting Cr-natural average total cross section. V = 50.05 - 0.262 + E, $W_S = 4.87 + 0.352 + E$, $V_{SO} = 7.0$ (MeV) r0= 1.24 . rs= 1.4 . rso= 1.24(fm) , b = 0.4 , aso= 0.48(fm) a0= 0.48 MT=2 Elastic scattering (Total - (All other partial cross sections) MT=4.51-72.91 Inelastic scattering Calculated with statistical and optical model code CASTHY/8/ Level scheme taken from Ref. 94. No. Energy MeV Spint-Parity 0.0 3/2 g.s. 1 0.5640 1/2 -

234 567 8910 1112 1314 1516 1718 1920 21	1.0063 1.2995 1.5366 1.9736 2.1724 2.2330 2.3208 2.4531 2.6595 2.7065 2.7065 2.7080 2.7720 2.8266 2.9930 3.0841 3.0930 3.1390 3.1390 3.1793	5/2 = 7/2 = 7/2 = 7/2 = 5/2 = 11/2 = 9/2 = 3/2 = 3/2 = 11/2 = 13/2 = 11/2 = 13/2 = 11/2 =	
22	3,2439	5.2 ·	
Contin	uum levels assum	ed above 3.435 MeV	
Level de	nsity parameters	of Yoshida /10/ ver	e used.
MT-16 (n.1	2n)		
Calculate	ed with evalpora	tion-model code GROG	I /11/, and
normalize	ed to the evaluation	ted value of Boedy+7	3 /18/ (0.89 b at
14.8 MeV	and modified t	o reproduce the expe	rimental values
OI UT≃na 15 MoV	tural (n.Zn) by	rrenau+eu /19/ in ch	e range of 9.0 to
MT∴28 (n.)	1 ' n)		
Calculate	ed with evaporat	ion-model code GROCI	/11/, and
normaliza	ed to 7.2 milli	barns at 14.8 MeV wh	ich is an average
value of	the experimenta	1 data by Husain+67	12/ and Webber+
68 13 .			
MI-102 Capt	ure doubt statistic	aal and antianl made	Leade Cherry Q
and norma	With statisti	cal and optical mode	t 50 keV for
Cr-natura		dee to milli on no a	
MT=103 (n.p	• •		
Calculate	d with evaporat	ion-model code GROGI	/11/. and
normalize	d to 0.0416 bar	ns at 14.8 MeV (aver	age value of the
experimer Valleener?	tal data by Hus	ain+6/ /12/. Prasad+	(1 /14/,
WE-107 (n.e.	o /15/ and yaim	+// /10/).	
	d with evaporat	ion-model code GROGI	/11/. and
normalize	d to 0.0451 bar	ns at 14.7 MeV by Do	lja+73 /14/.
MT=251 Mu-b	ar		
Calculate	d with optical s	nodel.	
ME-A Angular	Distributions of	Secondary Neutrons	
MT=2 Ca	leulated with C	ASTHY 7.	
MT=16,28 As	sumed to be iso	tropic in the lab sys	stem.
MT=51-72 As	sumed to be iso	ropic in the center-	of-mass system.
MT=91 Ca	lculated with C	STHY, and the same o	listribution
Wa	s assumed in the	e lad system.	

MF=5 Energy Distributions of Secondary Neutrons MT=16,28,91 Evaporation spectra were given.

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1 of Chromium-54

24-Cr- 54 MAT number = 2244

24 Cr 54 JAERI Eval-Aug82 T.Asami Dist-Mar83 Revi-Nov83

History

82-08 New evaluation was made by T. Asami (JAERI) for JENDL-2. 83-11 Modification for MF-4.

MF-1 General Information

MT=451 Descriptive data and dictionary

MF-2 Resonance Parameters

MT-151 Resolved resonance parameters for MLBW formula Resclved resonance region : 1.0E-5 eV - 300 keV Evaluation based on the experimental data by Stieglitz+71 1 . Allen 77 2 and Kenny 77 3 . and the recommended data in BNL 325 3rd edition 4. Assumed gamma width = 2.5 eV for s-wave and 0.28 eV for p-wave resonances. Effective scattering radius = 4.8 fm /5/.

Calculated 2200 m/s cross sections and res. integrals. 2200-m/s res. integ. . 1

erastic	1,914 0	-
capture	0.360 b	0.193 b
total	2.274 b	here.

MF-3 Neutron Cross Sections

Resonance region (from 1.0E 5 eV to 300 keV) Background cross sections were applied to reproduce the 2200 m s capture cross section of 0.36 - 0.04 barns /5/. and to modify the elastic scattering cross section in the lower energy region.

Above 300 keV

MT=1 Total Calculated with optical model. Potential parameters /5/ were obtained by fitting Cr-natural average total cross section. V = 50.05 - 0.262 (E, Ws = 4.87 + 0.352 (E, Vso = 7.0) (MeV)rO= 1.24 , rs= 1.4 , rso= 1.24(fm) . aso= 0.48(fm) a0= 0.48 , b = 0.4MT=2 Elastic scattering (Total)- (All other partial cross sections) MT=4.51-71.91 Inelastic scattering

Calculated with statistical and optical model code CASTHY/6/.

Level scheme taken from Ref. 70 No. Energy (MeV) Spint-Parity 0.0 0+ g.s. 1 0.8349 2 +

- 71 -

Z 10 12 15

2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20	1.8237 2.6195 2.8294 3.0739 3.1600 3.2225 3.3920 3.4366 3.4680 3.5140 3.6552 3.7198 3.7658 3.7989 3.8640 3.9340 3.9340 3.9340 4.0160 4.0160	4 + 2 + 0 + 2 + 2 + 6 + 1 + 2 + 1 + 2 + 4 + 2 + 4 + 2 + 3 + 1 - 3 - 6 - 6 -
21	4.0832	4
Contin	uum levels assur	red above 4.088 MeV.
Level de	ensity parameters	s of Yoshida /8/ were used.
'MT≂16 (n.	.2n)	
Calculat	ed with evaporat	ion-model code GROGI /9/, and
normalia	ed to 1.12 barns	at 14.8 MeV which was evaluated by
Boedy 70	3 ∵15 ′.	·
MT=102 Car	oture	
Calculat	ed with statisti	cal and optical model code CASTHY/6/,
and norm	nalized to reprod	uce 10 milli-barns at 50 keV for
Cr-natur	al.	
MT=103 (n.	p 1	
Calculat normaliz experime Qaim+77	ed with evaporat ed to 0.016 barn ntal data by Hus 12 .	ion-model code GROGI /9/, and s at 14.7 MeV (average value of the ain:67 10 . Valkonen76 /11/ and
MT 107 n.	alpha	
Calculat normaliz value of and Sail	ed with evaporat. ed to 0.014 barns the experimental er+77 /14/.	ion model code GROGI <u>/9</u> /, and s at 14.7 MeV which is an average 1 data by Husain+67 /10/, Qaim74 /13/
MT-251 Mu-	bar	
Calculat	ed with optical r	nodel.
MF=4 Angular	Distributions of	f Secondary Neutrons
MI=2,51-70	Calculated with	CASTHY /6/.
MI=91	Calculated with	CASIHY, and the same distribution
	was assumed in t	the lab system.
MI=16	Assumed to be 1s	sotropic in the lab system.
MF=5 Energy MT=16.91	Distributions of Evaporation spec	Secondary Neutrons ctra were given.
Pofessona		
1) Stieglitz	R.G. et al.: Nuc	1. Phys. A163, 592 (1971).

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1 of Manganese-55

25-Mn- 55 MAT number = 2251

25 Mn 55 rBEC Eval Dec82 T.Hojuyama Dist Mar83 Rev1-Nov83

History

- 82-12 Data were evaluated by T. Hojuyama (FBEC) for JENDL-2. 83 11 Comment was added.
- MF-1 General Information

MT-451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT-151 Resolved resonance parameters for MLBW formula Energy range from 1.0E 5 to 1.0E+5 eV. Resonance parameters were taken from PNL 325 4th ed. Ref.1. Effective scattering radius var determined to 5.3 fermi.

Calculated 2200 m s cross sections and resonance integrals 2200 m sec res. integ.

elastic	2.184	b	
capture	13.32	b	14.6 b
total	15.50	b	

MF 3 Neutron Cross Sections

MT 1 Total cross section

Obtained with optical model calculation. Optical potential parameters were taken from Kawai's evaluation (Ref.2). Optical potential parameters

urca	r porei	161da (M	ram	eters		
۷.	47.856	0.032 E	'n		MeV	5
Ws	5.288	0.342-E	n		MeV)
Vso	6.90				MeV	į
r0	rso	1.24 .	::s	1.41	िमा ः	
380	8	0.522.	b	0.392	- fm -	

MT- 2 Elastic scattering cross section

Obtained by statistical model calculations. $MI-4 \ \%$ from 51 to 91 Inelastic scattering cross sections

Obtained by statistical and optical model calculations. Level scheme taken from N.D.S. (Ref.3)

NO.	Energy (MeV)	Spin-Parity
g.S.	0.0	5/2
1	0.1260	7/2 -
2	0,9843	92 -
3	1.2900	1/2 -
4	1.2922	11/2 -
5	1.5289	3 2 -
6	1.8853	72-
7	2,1985	7/2
8	2.2153	52
9	2,2533	3,2
:0	2.2694	1/2 +
11	2.3118	13/2 -
12	2,3636	5/2 -
13	2,3990	5/2 -

14 2.4286 1.2 + 15 2.5648 3/2 Continuum levels assumed above 2.58 MeV. MT 16 (n.2n) cross section Besed on the following exp. data : Thr 14.7 MeV Paulsen(Ref.4) Auchampaugh (Ref. 5) : 14.7 - 20.0 MeV Normalization : : 14.7 MeV Energy Cross section : 787 mb : Barrall (Ref.6) & Auchampaugh (Ref.5) Data-base MT 22 & 28 $(n,n'a) \otimes (n,n'p)$ cross sections Determined by statistical model calculation with preequilibrium effect corrections (Ref.7). Normalization : Method Cross section systematics : 14.7 MeV -Energy Cross section 5 mb(n,n'a & 11 mb(n,n'p) MT 102 Capture cress section Based on following exp. data ; Garg Ref .8 > : 0.7MeV Dovbenko (Ref .9) : 1.0MeV Dovbenko & Menlove (Ref. 10) : 3.5MeV Menlove : 10 MeV Schwerer (Ref. 11) & Budnar (Ref. 12) : > 10 MOV MT 103 (n.p) cross section Determined statistical model calculation(Ref.7) by Normalization : Energy : 14.7 MeV Cross-section : 44.7 mb Data base : Prasad (Ref. 13) & Allan (Ref. 14) MT 107 (n.a: cross section Based on the statistical model calculation (Ref.7 / / 13MeV. and exp. data Zupranska Ref.15 13MeV. Normalization : : 14.7 MeV Energy Cross section 28,5 mb Data base : Dresler Ref. 16 & Zupranska MT 251 Mu bar Calculated with optical model. MF-4 Angular Distributions of Secondary Neutrons MT 2 Elastic scattering based on statistical & optical model calculation. MT= 16,22.28 & 91 The angular distributions of the secondary neutrons from (n.2n), (n.n'a), (n.n'p) & inelastic processes to the continuum levels assumed to be isotropic in the laboratory system. MT= from 51 to 65 The angular distributions of the secondary neutrons from inelastic processes assumed to be isotropic in the c.o.m. system.

MF=5 Energy Distributions of Secondary Neutrons

MT= 16.22.28 and 9: The evaporation spectrum assumed for the secondary neutrons from (n.2n/, (n.n'a), (n.n'p) & in- elastic processes.

3 of Manganese-55

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1 of Natural Iron

28-Fe- 0 MAT number = 2260

26 Fe 0 JNDC Eval-Oct78 S.Iijima,H.Yamakoshi Dist Mar83 Rev1 Nov83

History

 78-10 New evaluation for JENDL 2. Details given in Ref. /1/.
 83-03 Resonance region was revised by T.Asami and T.Narita. The file structure was modified and background data were

evaluated to reproduce the same cross sections as before. 83-11 The Q values were adjusted to threshold energies. Other small corrections were made.

Natural iron data constructed from Fe isotopes. Fe 58 was ignored because of small abundance.

MF 1 General Information

MT 451 Descriptive data and dictionary

MF-2 Resonance Parameters

MT=151 Resolved resonances

Resonance region = 1.0E-5 eV to 250.0 keV The multilevel Breit-Wigner formula was used. Parameters were adopted from the following sources.

Fe-54 : Pandey /2 for 0 = 680 keV, R-5.6 fm from Ref.1

Fe-56 : Perey+/3/ for -2.0 - 400 keV. R=5.4 fm from fitting to total cross section below 60 keV.

Fe 57 : Allen+/4. for s-wave resonances, and Beer+/5/ for p-wave resonances in 0 - 185 keV.

For Fe 56, a negative level was added at -3.75 keV with neutron width of 100 eV and gamma width of 1.0 eV. Neutron width of 27.67 keV resonance was taken as 1420 eV.

Calculated 2200 m s cross sections and res. integrals.

	2200 11	S	res, integ.
elastic	12.44	b	
capture	2.514	b	1.349 b
total	14.95	Ь	-

- MF-3 Neutron Cross Sections Below 250 keV, background cross sections were given.
 - MT-1 Total

For energies 250 keV - 20 meV, fine resolution data were taken by eye-guide using interactive display of NDES (Neutron Data Evaluation System) developed by T.Nakagawa at the Nuclear Data Center, JAERI. Below 4 MeV, data of Carlson+6/ were adopted. Above 4 MeV, data of Cierjacks+ '7/ were adopted. Spherical optical model calculation was also made (but not adopted as the total cross section for JENDL 2. Parameters are as follows. For Fe-56

V = 52.644-0.002+E-0.006+E+12, r0=1.166, aC=0.371Ws = 2.869+0.289+E , rs=1.450, as=0.48(

2 of Natural Iron

Vso: 6.138	, rso 1.166.aso 0.371
V = 50,136-0,150+E	$r0^{-1}.240$, $a0^{-0}.500$
Vso 7,00	, rso=1,240,aso=0,500

Energies in MeV unit, lengths in fm unit.

MT 2 Elastic scattering

Given as total minus other cross sections

MT=16 (n.2n)

Calculated using essentially the method of Pearlstein. Normalized to experimental data at 14 MeV.

MT 4.51 91 Inelastic scattering

Level excitation cross sections were calculated for each isotope with the optical and statistical model code CASTHY -8, and the results were modified to obtain a better fit to experimental data. The modifications are :

Fe 54 : Calculation was multiplied by a factor of 1.5.

- Fe 55 : Calculation was multiplied by a factor of 1.17. The 1st level excitation below 2.12 MeV was replaced with fine resolution data of Kinney+ .9.
- Fe 57 : CASTHY calculation for 1st level (14.4 keV) Excitation was replaced in the energy from 14.65 keV to 200 keV by calculated cross sections from resonance parameters.

For natural iron, isotopic level excitation cross sections with about equal threshold energies were condensed together. Isotopic level schemes recommended by R.Nakasima and condensed level scheme are tabulated below. Q values of natural iron levels were calculated from their threshold energies.

		Fe 54	Fe 56	Fe-57
No.	Q val MeV	E MeV J P	E-MeV J P	E (MeV / J-P
1	0.0144			0.0144 3 2
2	0.1366			0.1366 5/2~
3	0 3666			0 3667 3/2-
1	0 7064			0 7067 5/2
5	0.1004		0 9469 2	0.1001 0/2
5	0.0400		0.0400 2.	1 000 7.0
6	1.0076		**	1.008 1/2-
7	1.1976	6.31	-	1.198 9/2-
8	1.2647		6a.	1.2651 1/2-
9	1.3563			1.3568(1/2-)
10	1.4091	1.4082 2+		-
11	1.6271	-	***	1.6277 3/2-
12	1.7251			1.7257 3/2-
13	1.9743	-		1.975 (1/2-)
				1.9894 9.2-
14	2.0850		2.0851 4+	·
15	2.1162			2,117 5/2-
16	2.5397	2.5382 4+	+	
17	2.5629	2.5613 0+		
18	2.6574		2.6576 2+	
19	2 9416	2 9499 6+	2 9417 0+	
	C.0710	2 2590 2+	2 9600 2-	
20	2 1100	2.000 2.	3 1200/14	_
20	0.1139	_	3.1200(17)	—
			3. 12CH 4+	

21 22	3.1680 3.2972	3,1661 2+ 3,2952 4+		
23	3.3470	3,3450 3	3,3702 2	
			3,3884 6+	
24	3,4451	+1	3.4454 3	-19
			3,4493 1+	
20	3.6007		3,6009 24	
			3.6019 2+	
17.00	40. 478 a 114 als		3.6070 0	
26	3.7478		3.7480 24	
			3,7558 6	
27	3.8318	3.8338 4+	3.832 2+	
			3.8565 3+	
28	4.0354	4.033 4+	4,0940(3+)	
		4.047 4		
		4.072 3		
29	4.1001		4.1003/3	
			4.1200 4	
30	4.2656	4.263 4	4.2982 4	
_		4.292 0+	4.302 (0+)	
31	4.3948		4,3950 3	92 G
			4.401 (2+)	
32	4.4582	•	4,4584 3+	100
33	4.0098	a	4.5100 3	8.0

Continuum level excition including each isotopic continuum level and 15, 16, 17, 18, 19, 20th levels of Fe-54.

Level density parameters for modified Gilbert-Cameron formula (as proposed in FPND Conf., Petten (1977) /10,11/) are as follows. For nortation, see Ref. /11/.

Isotope	a(i/MeV)	T (MeV)	C(L/MeV)	EX (MeV)
Fe 54	6,19	1.45	0.532	12.0
Fe 55	6.90	1.30	1.274	9.0
Fe 56	7,58	1.27	0.746	10.0
Fe 57	8.27	1.14	1.694	7.70
Fe 58	3.75	1.16	0.742	10.0

MT-102 Capture

Background cross section was given below 3 keV. Between 3 and 800 keV, recommended cross section by Ribon /12/ was adopted. Above 800 keV, CASTHY calculation was used adjusting S-gamma to the experimental cross sections of 10 mb at 200 keV for Fe-54 and 5.5 mb at 200 keV for Fe-56. obtained effective S-gamma values are 0.34E-4 for Fe-54 and 0.18E-4 for Fe-56. The S-gamma of 3.53E-4 was used for Fe-57 without adjustment. Giant dipole resonance parameters were taken from systematics of compilation by Berman /13/.

MT=103.107 (n.p) and (n.alpha) Adopted from experimental data. For Fe-56 (n.alpha),

the same cross section as Fe-54 was adopted.

MT=251 Mu-bar

Calculated with CASTHY 8 .

MF=4 Angular Distributions of Secondary Neutrons MT=2 : CASTHY-code calculation MT=51-88 : Isotropic in the center-of-mass system MT-16,91 : Isotropic in the laboratry system

MF-5 Energy Distributions of Secondary Neutrons MT-16,9! : Evaporation spectrum

References

- 1) Iijima S. et al.: Proc. NEANDC Topical Discussions in 1979. Kawai M.: ibid. Yoshida T.: ibid. Yamakoshi H. et al.: J. Nucl. Sci. Technol., 17, 477 (1980). Iijima S. et al.: to be published as JAERI M report.
- 2 Pandey M.S. et al.: Proc. Conf. Nuclear Cross Sections and Technology, Washington D.C., (1975), p.748.
- 3 Perey F.G. et al.: Proc. Specialist Meeting on Neutron Data of Structural Materials for Fast Reactors, Geel. 1977. p.530.
- 4 Allen B.J. et al.: ibid. Loc.Cit. , p.476.
- 5 Beer H. and Spencer R.R.: KfK 2063 1974 .
- 6 Carlson A.D. and Cerbone R.J.: Nucl. Sci. Eng., 42, 28 (1970).
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- 8) Igarasi S. : J. Nucl. Sci. Technol., 12, 67 (1975).
- 9) Kinney W.E. and Perey F.G.: Nucl. Sci. Eng., 63, 418 (1977).
- 10 Gruppelaar H.: ECN-13 (1977),
- 11: Iijima S.: IAEA 213, Proc. IAEA Second Advisory Group Meeting on Fission Product Nuclear Data, Petten. (1977), p.279.
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26-Fe- 54 MAT number - 2261

26 Fe 54 JNDC Eval Oct78 S. Lijima H. Yamakoshi Dist Mar83 Revi Feb84

History

- 78-10 New evaluation for JENDL 2. Details given in Ref. 1.
- 83-11 Small corrections were made.
- 84-02 Effective scattering radius was corrected. Comment was added.
- MF-! General Information

MT 451 Descriptive data and dictionary

- MF-2 Resonance Parameters
 - MT 151 Reselved resonances

Resonance region 1.0E 5 eV to 250.0 keV

The multilevel Breit Wigner formula was used. Parameters were adopted mainly from Pandey 2 by assuming the average radiative width to be 2.5 eV $_{23}$. R=5.6 fm was taken from Ref. 4 .

Calculated 2200 m/s cross sections and res. integrals.

	2200 m 's	Res. Integ	Į.
elastic	0,4929 b		
capture	2.156 b	1.33 b	
total	2.649 b		

MF-3 Neutron Cross Sections

Below 250 keV, background cross sections were given for the total and elastic scattering cross sections on the upper side of the first resonance. Above 250 keV, the cross sections were evaluated as follows.

MT I Total

Spherical optical model calculation was made by using code CASIHY 5. Optical potential parameters, 1 are as follows.

- Vso 7.00 , rso=1.240, aso=0,500

(energies in MeV, lengths in fm)

MT-2 Elastic scattering

Given as total minus other cross sections

 $MT{=}16 \qquad (n,2n)$

Calculated using essentially the method of Pearlstein /6/.and normalized to experimental data at 14 MeV.

M-4.51-91 Inelastic scattering

Level excitation cross sections were calculated with the optical and statistical model code CASTHY 5, and the results were multiplied by a factor of 1.5 to obtain a better fit to experimental data. Level scheme was recommended by R.Nakasima 7 as follows.

No.	Energy (MeV)	Spin-Parity
g.s.	0.0	0+

1 2 3 4 5 6 7 8 9 10 11 12	1.4082 2.5382 2.5613 2.9499 2.9590 3.1661 3.2952 3.3450 3.8338 4.033 4.033 4.047 4.072	240622434443			
13	4.2916	0 +			
15	4.578	2 -			
10	4,605	6 -			
18	4.700	ġ			
19	4.760	3			
20 Continuu	4.949	accumed at		MoV	
Level der mula as as follow isotor Fe 50 MT 102 Captu CASTHY of experium effectiv paramete Berman MT 103,107 Adopted f MT 251 Mo ba Calculate	sity paramete proposed in F is. For norta ic a (1 MeV) 6 6.19 5 6.90 me valculation wa ental cross se vers were taken 10 . n.p and n.a rom experimen r d with CASTHY	ers for mod PND Conf tion, see T(MeV) C 1.45 1.30 as used adj stions of ue is 0.34 (from syst lpha tal data. .5., of Secondal	lified Gi Petten Ref. 9 0.532 1.274 usting S 10 mb at E 4 Gi comatics	lbert Came (1977) /8. Ex(MeV) 12.0 9.0 Comma to 200 keV, ant dipole of compila	the Obtained resonance tion by
MT=4 Angular Distributions of Secondary Neutrons MT-2 : CASTHY-code calculation MT=51-70 : Isotropic in the center-of-mass system MT=16.91 : Isotropic in the laboratry system					
MF=5 Energy Di MT=16.91 . Ev	stributions of aporation spec	f <mark>Secondary</mark> ctrum	y Neutron	ns	
References 1) Iijima S. e Kawai M.: 1 Yoshida T.: Yamakoshi H Iijima S. e 2) Pandey M.S. Technology,	t al.: Proc. M bid. ibid. . et al.: J. M t al.: to be p et al.: Proc. Washington D.	VEANDC Topi Vucl. Sci. Dublished a Conf. Nuc C., (1975)	ical Disc Technol. as JAERI- clear Cro), p.748.	ussions in . 17, 477 M report. Dss Section	1979. (1980). Is and

- 3) Spencer R.R. and Beer H.: KfK 2046, 79 (1975).
- 4) Mughabghab S.F. and Garber D.I.: BNL 325, 3rd Ed. Vol. 1 (1973).
- 5) Igarasi S. : J. Nucl. Sci. Technol., 12, 67 (1975). 6) Pearlstein S.: Nucl. Sci. Eng., 23, 238 (1965).
- 7) Nakasima R.: private communication.
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- 9) Iijima s.: IAEA 213. Proc. IAEA Second Advisory Group Meeting on Fission Product Nuclear Data. Petten. (1977), p.279.
- 10) Berman B.L.: Atom. Data and Nucl. Data Tables. 15, No.4 (1975)

1 of Iron-56

26-Fe- 56 MAT number -= 2262

26-Fe-56 JNDC Eval-Oct78 S.Iijima.H.Yamakoshi Dist-Mar83 Rev1-Feb84

History

78-10 New evaluation for JENDL-2. Details given in Ref. 17.

83 11 Small corrections were made.

84-02 Background data were modified. Comment data were added.

MF-1 General Information

MT-451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT 151 Resolved resonances Resonance region 1.0E 5 eV to 250.0 keV The multilevel Breit Wigner formula was used. Parameters were adopted from the experimental data by Perey: .27, R 6.5 fm was selected to reproduce the 24 keV window cross

section. Neutron width of **27.67 keV resonance was taken as** 1420 eV.

Calculated 2200 m s cross sections and res. integrals.

	2200 m/s	Res. Integ.
elastic	12.46 b	
capture	2.813 b	1.44 b
total	15.18 b	

MF 3 Neutron Cross Sections

Below 250 keV, background cross sections were given to reproduce the thermal elastic scattering cross section of 12.46 barns 3. Above 250 keV, cross sections were evaluated as follows.

MT 4 Total

Spherical optical model calculation was made by using CASTHY code 4. Parameters /1, are as follows, $V = 52,644, 0.002\pm E \cdot 0.006\pm E\pm 12, r0\pm 1.166, a0\pm 0.371$

Ws 2.869+0.289+E rs-1.450, as=0.480 Vso= 6.138 rso=1.166.aso=0.371 (energies in MeV, lengths in fm).

MT-2 Elastic scattering

Given as total minus other cross sections. Fine structure in the MeV region is due to the inelastic scattering cross section.

MT=16 (n.2n)

Calculated using essentially the method of Pearlstein $^{\prime}5^{\prime}$. Normalized to experimental data at 14 MeV.

MT=4.51-77.91 Inelastic scattering

Level excitation cross sections were calculated with the optical and statistical model code CASTHY 4. The result was multiplied by a factor of 1.17. The 1st level excitation below 2.12 MeV was replaced with fine resolution data of Kinney+ /6/. Level scheme was recommended by R.Nakasima /7/.

No.	Energy (MeV)	Spin Parity
g.s	0.0	0 +
1	0.8468	S +
2	2.0851	4 .
3	2.6576	2 .
4	2.9417	0 +
5	2,9600	<u>2</u> +
6	3.1200	1 1 1
7	3.1229	4 +
В	3.3702	2 1
9	3.3884	<u>ē</u> ,
10	3.4454	3 -
11	3.4493	Ĩ.
12	3.6009	2.
13	3.6019	2 -
14	3 6070	O ·
15	3 7480	2.
16	3 7.52	6
17	3.332	S •
18	3,8565	3 🔹
19	4,0940	3 ()
20	4.1003	(3 →)
21	4.1200	(4 +)
22	4.2982	4 +
23	4.302	<0 → 1
24	4.3950	3 -
25	4.401	(2) (c)
26	4 4584	3 +
27	4.5100	3

Continuum levels were assumed above 4.6 MeV.

Level density parameters for modified Gilbert Cameron for mula as proposed in FPND conf., Petten 1977 8.9are as follows. For nortation, see Ref. 9. a MeV T MeV C : MeV Ex MeV isotene 7.58 Fe 56 1.27 0.746 10.0 Fe 57 7.70 8.27 1.14 1.694

MT 102 Capture

CASTHY calculation was used adjusting S-gamma to the experimental cross sections of 5.5 mb at 200 keV. Obtained effective S gamma value is 0.18E 4. Giant dipole resonance parameters were taken from systematics of compilation by Berman 10 .

Mf = 103, 107 (n,p) and (n,alpha)

Adopted were the same cross section as Fe 54 which was evaluated from experimental data.

MT=251 Mulbar

Calculated with CASTHY 4

MF-4 Angular Distributions of Secondary Neutrons

MT=2 : CASTHY code calculation

- MT-51-77 : Isolropic in the center of mass system MT=16.91 : Isolropic in the laboratry system

MF=5 Energy Distributions of Secondary Neutrons MT=16.91 : Evaporation spectrum

References

- 1 · Iijima S. et al.: Proc. NEANDC Topical Discussions in 1979. Kawai M.: ibid. Yoshida T.: ibid. Yamakoshi H. et al.: J. Nucl. Sci. Technol., 17, 477 (1980). Iijima S. et al.: to be published as JAERI M report.
- 2 Percy F.O. et al.: Proc. Specialist Meeting on Neutron Data of Structural Materials for Fast Reactors. Geel. 1977. p.630.
- 3 Mughabghab S.F. et al.: Neutron Cross Sections, Vol. 1, Part A 1981.
- 4 Igarası S. S. J. Nucl. Sci. Technol., 12, 67 (1975).
- 5 Pearlstein S : Nucl. Sei, Eng., 23, 238 1965 .
- 6 Kinney W.F. and Percy F.G. Nucl. Sci. Eng. 63, 418 1977 .
- 7 Nakasima R private communication
- 8 Gruppelaar H.: ECV 13 1977
- 9 Lijima S.: IAEA 213, Proc. IAEA Second Advisory Group Meeting on Fission Product Nuclear Data, Petten, 1977, p.279.
- 10 Berman B.L., Atom. Data and Nucl. Data Tables, 15, No.4 (1975)

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1 of Iron-57

26-Fe-57 MAT number = 2263

26 Fe 57 JNDC Eval-Oct78 S. Iijima H. Yamakoshi Dist Mar83 Rev1 Feb84 History 78-10 New evaluation for JENDL 2. Details given in Ref. 17. 83 11 Small corrections were made. 84 02 Comment was added. MF 1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved resonances Resonance region 1.0E 5 eV to 200.0 keV The muluilevel Breit Wigner formula was used. Parameters were adopted from Allen- 2 for s wave resonances, and Beer- 3 for p wave resonances in 0 - 185 keV. Calculated 2200 m/s cross sections and res. integrals. 2200 m /s Real Integ. 0.2021 b elastic 2.462 b capture 1.45 b total 2.664 b MF-3 Neutron Cross Sections Below 200 keV, background cross section was given for the capture cross section to reproduce the value of (2.48 ± 0.3) barns 4 at 0.0253 eV. The background data for the total cross section contain algo contributions from the inelastic scattering cross section. Above 200 keV, the data were evaluated as follows. MT 1 Total Spherical optical model calculation was made with CASTHY code /5'. Parameters /1/ are as follows. V == 50,136 0,150(E r0=1,240, a0 0.500 Ws = 4,600+0.340+E, rs=1.400. as=0,400 Vso= 7.00 . rso-1.240.aso-0.500 (energies in MeV unit, lengths in fm unit) MT=2 Elastic scattering Given as total minus other cross sections MT=16 (n.2n) Calculated using essentially the method of Pearlstein /6/, and normalized to experimental data at 14 MeV. MT=4.51-64.91 Inelastic scattering Level excitation cross sections were calculated with the optical and statistical model code CASTAY 757. The calculation for 1st level (14.4 keV) excitation was replaced in the energy from 14.65 keV to 200 keV by calculated cross sections from resonance parameters. Level scheme was recommended by R.Nakasima /7/.

2 of Iron 57

No,	Energy MeV	Spin Parity
g.s.	0.0	15
1	0.0144	3.2
2	0.1366	5 2
3	0.3667	3.2
4	0.7067	5 2
15	1.008	7.2
6	1.198	9.2
7	1.2651	1.2
8	1.3568	1.2
9	1.6277	3.2
10	1.7257	3.2
11	1 975	1.2
12	1.9894	9.2
18	2 117	5.2
14	2 207	5 2
	1	a must distant of Q K

Continuum levels were assumed above 2.3 MeV.

Level density parameters for modified Gilbert Cameron for mula as proposed in FTND cont. Petter 1977 8.9 are as follows. For nortation, see Ref. 9. isotope a 1 MeV T MeV C 1 MeV Ex(MeV)Fe 57 8.27 1 14 1.694 7.70

Fe 58	8.45	1.16	0 742	10.0

MT 102 Capture

Calculated with CASTHY adopting the S gamma of **3.53E 4**. Giant dipole resonance parameters were taken from systematics of compilation by Berman 10

MT 251 Mu bar Calculated with CASTHY 5

MF 4 Angular Distributions of Secondary Neutrons MT 2 CASTHY code calculation

MT 51 64 1 isotropic on the center of mass system

MT 16.91 . Isotropic in the laborativ system

MF 5 Energy Distributions of Secondary Neutrons MT 16.91 : Evaporation spectrum

References

- 1 Iijima S. et al.: Proc. NEANDC Topical Discussions in 1979. Kawai M.: ibid. Yoshida T.: ibid.
 - Yamakoshi H. et al.: J. Nucl. Sci. Technol., 17, 477 (1980).
 - lijima S. et al.: to be published as JAERI M report.
- 2) Allen B.J. et al.: ibid. (loc.cit.), p.476.
- 3) Beer H. and Spencer R.R.: KfK 2063 (1974).
- 4 Mughabghab S.F. et al.: Neutron Cross Sections. Vol. 1, Part A 1981
- 5: Igarasi S. : J. Nucl. Sci. Technol., 12, 67 (1975).
- 6. Pearlstein S.: Nucl. Sci. Eng., 23, 238 (1965).
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- 8 / Gruppelaar H.: ECN-13 (1977).
- Iijima S.: IAEA-213, Proc. IAEA Second Advisory Group Meeting on Fission Product Nuclear Data, Petten, (1977), p.279.
- 10 Berman B.L.: Atom. Data and Nucl. Data Tables, 15, No.4 (1975)

1 of Iron 58

26 Fe 58 MAT number 2264

26 Fe 58 JAERI

Eval Mar76 JENDL CG Dist Mar83 Rev1 Jan84

History

- 76 03 Evaluation for JENDL 1 1 was made by JENDL 1 Compilation Group.
- 83-03 Resonance region was revised by T.Asami (JAERI). MF=5 was also revised.
- 84 01 Comment was added.
- MF 1 General Information MT 451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT 151 Resolved resonances Resonance region 1.0E 5 eV to 100.0 keV The multilevel Breit Wigner formula was used. Parameters were adopted from the recommended values by Mughabghab et al. 27.

Calculated 2200 m s cross sections and res. integrals.

	2200 m s	res. integ.
elastic	2,821 b	
capture	1.280 b	1.83 b
total	4.101 b	,

MF 3 Neutron Cross Sections

Below 100 keV, background cross sections were given to reproduce the thermal cross section recommended in Ref. 2. Above 100 keV, the data were evaluated as follows.

MT	1.2.4.51 9	58.91.102 Tot	al. Elastic, Inelastic and Capture
	Calculate	ed with optica	1 and statistical model code CASTHY
	3. Opt	lical potentia	1 parameters were determined by
	Yamakoshi	i 1 to repro	duce the smoothed total cross section.
	V 5.4	16.0-0.25+En (MeV). Wi = 0.125+En-0.0004+En++2 (MeV)
	Ws 🖘 1	4.0-0.2+En (MeV), Vso= 6.0 (MeV)
	Gau	ussian form)	
	R ⇒ 1	.16+a++1,3+0.	4821 (fm), a0 = 0.62 (fm)
	Ri 🗉 1	.16+a++1/3+0.	(fm), ai = 0.62 (fm)
	Rs ≈ 1	.16+a++1/3+0.	8799 (fm), as = 0.7 (fm)
	Rso≃ 1	.16+a++1/3-0.	3443 (fm), aso= 0.62 (fm)
	The level	scheme was t	aken from Refs. /4,5/ as follows.
	No.	Energy (MeV) Spin-Parity
	g.s.	0.0	0 +
	1	0.8106	2 +
	2	1.6745	2 -
	3	2.133	3 -
	4	2.257	0 ·
	5	2.596	4 +
	6	2,782	1 +
	7	2.876	2 +

8 3.0842 . Levels above 3.15 MeV were assumed to be overlapping. The (n.2n). (n.p) and (n.alpha) reactions were considered as competing processes. MT 16 (n,2n)Calculated with Pearlstein's method S, and normalized to 1.06 barns /7 at 14.5 MeV. MT 103 (n.p) Adopted from the evaluated data of Alley and Lessler 9. MT 107 (n.alpha) The shape of cross section was taken from Schmit's evaluation 9 and normalized to the data of Gardner and Yu-Wen 10 MT 251 Mu bar Calculated with CASTHY 3 . MF 4 Angular Distributions of Secondary Neutrons MT 2 CASTHY code calculation MT 51 58 1 Isotropic in the center of mass system MT [6,9] : Isotropic in the laboratry system MF 5 Energy Distributions of Secondary Neutrons MT 16.91 : Evaporation spectrum References 1 Igarasi S. et al.: JAERI 1261 (1979). 2 Mughabghab S.F. et al.: "Neutron Cross Sections, Vol. 1, Part A', Academic Press (1981). 3: Igarasi S. : J. Nucl. Sci. Technol., 12, 67 (1975). 4: Lederer C.M. et al.: Table of Isotopes. Sixth Edition. John Wiley and Sons Inc. (1967). 5 Raman S.: Nucl. Data Sheets, 3, 145 (1970). 6 Pearlstein S.: Nucl. Sci. Eng., 23, 238 1965 / 7 Boedy Z.T.: INDC HUN + 10 1973 . 8 Alley W.E. and Lessler R.M.: Nucl. Data Tables, 11, 622 1973 9 Schmit J.J: KfK 120 - 1966 . 10 Gardner L. and Yu Wen Yu.: Nucl. Phys., 60, 49 (1964).

1 of Cobalt-59

27 Co 59 MAT number 2271

27 Co 59 FUJI: Eval Nov82 T.Aoki,T.Asami Dist Mar83 Revi Jan84

History

82-11 Evaluation was completed by T.Aoki (FUJI) and T.Asami (JAERI). 84-01 Resonance parameters were revised and comment was added.

MF 1 General Information

MT 451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW formula Energy range : from 1.0E 5 eV to 100 keV. Resonance energies, neutron and radiative widths were taken from recommended values of Mughabghab et al. 1. Radiative widths assumed for resonances whose widths were not recommended were 0.56 eV and 0.7 eV for s wave and p-wave resonances, respectively. Two negative resonances were adopted also from Ref. /1/. The effective scattering radius and parameters of the 500 keV resonance were adjusted to reproduce the thermal cross sections recommended in Ref. /1/.

Calculated 2200 m sec cross sections and resonance integrals.

	2200 m	sec	res. integ.
elastic	6.001	b	
capture	37.18	b	75.6 b
total	43.18	b	

MF 3 Neutron Cross Sections

Below 100 keV: Resonance region. Background cross sections are zero.

Above 100 keV: Data were evaluated for the following MT's.

- MT 1 total cross section
- MT 2 elastic scattering cross section
- MT= 4 total inelastic scattering cross section
- MT = 16 (n.2n) cross section
- MT=51 57 inelastic scattering cross sections to discrete levels
- MT= 91 inelastic scattering cross section to continuum levels
- MT=102 capture cross section
- MT=103 (n,p) cross section
- MT=107 (n.alpha) cross section
- MT-251 mu-bar

Cross sections were calculated with optical and statistical model code CASTHY 2. The optical potential parameters were selected by Kawai 3 to reproduce average total cross section in the high energy region.

The	inelastic	scattering	cros	s secti	ons were	given for	seven
а	≈ 0.541	,	b =	0.400	,	aso=0.541	(fm)
r	- 1.240	•	rs -	1.400	,	rso=1.240	(fm)
v	49.69	0.135 En.	Ws -	4.231-0	.198 En.	Vso=7.00	(MeV)

discrete levels of which level scheme was taken from Ref. /4 .

No.	Energy (MeV)	Spin Parity
g.s.	0.0	72
1	1.099	32
2	1.292	32
3	1,434	12
4	1.482	52+
5	1.745	72
6	2.063	72
7	2.088	9.2

Levels above 2.154 MeV were assumed to be continuum. The level density parameters of Gilbert and Cameron (5) were used. The (n,2n), (n,p) and (n,a) cross sections were taken into account as competing processes. The evaluation of these cross sections were made as follows.

- n-2n based on the experimental data of Paulsen and Liskien 6 .
- n.p calculated with GNASH 7, and normalized to the experimental data of Smith and Meadows 8.9 at 5 MeV.
- (n,alpha) evaluated from the experimental data of Santry and Butler ≥ 10 .

The mu bar was also calculated with CASTHY /2/,

MF 4 Angular Distributions of Secondary Neutrons

- MT 2.51 57 : Calculated with CASTHY 2
- MT 16 : Assumed to be isotropic in the laboratory system.
- MT 91 Calculated with CASTHY 2
- MF 5 Energy Distributions of Secondary Neutrons MT 16.91 : Evaporation spectra.

References

- 1 Mughabghab S.F. et al., "Neutron Cross Sections, Vol. 1, Part A., Academic Press, 1981.".
- 2 Igarasi S.: J. Nucl. Sci. Technol., 61, 477 (1976).
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- 9) Smith D.L. and Meadows J.W.: Nucl. Sci. Eng., 60, 187 (1976).
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28 Ni 0 MAT number 2280

O JAERI Eval Nov79 Y.Kikuchi and N.Sekine 28 Ni Dist Dec79 Rev1 Nov83 History 79 11 New evaluation for JENDL 2. MF's 2, 3 and 4 were reliesed as JENDL 2B 1 83 03 MF 5 was added as final JENDL 2 data. 83 11 Comment was added. MF 1.MT 451 Comments and dictionary MF 2.MT 151 Resolved resonance parameters : 1.0E 5 eV 600 keV Evaluation based on the following data. Ni 58 | Trans. | Perey 2 . Syme 3 .Farrel+ 5 Capt. : Perey 2 . Freehner 4 .Hockenbury:6. Ni 60 : Trans. : Syme 3 .Stieglitz: 7 .Farrel:/5. Capt. Froehner 4 .Stienlitz(7).Hockenbury+/6/ N1 61 : Trans, : Cho+ 8 Capt. : Froehner/4/, Hockenbury+/5/ Ni 62 | Trans. | Beer (-97 .Farrel+-57 Capt. : Beer 9 Ni 64 | Trans. | Beers 9 Farrel+/5 Capt. : Beer . 9 A negative resonance added for Ni 58. Assumed Gameg : 2 eV for s wave res. and 1 eV for p-wave. Calculated 2200 m s values and resonance integrals (barn): 2200 m s value res.int. 21.20 total 6.77 elastic 4.429 2.22 capture MF 3 Neutron Cross Sections Background cross sections (BCCS) applied to resonance region. MT 1.2 : Positive or negative BCCS to compensate errors due to constant R and resonance truncation /10/. MT-102 : Smooth positive BGCS above 10 keV. Cross sections above resonance region evaluated as follows : MT=1 : Total cross section Calculated with optical model. Potential parameters obtained by fitting nat-Ni data /11/: V =51.33 - 0.331+En .Ws=8.068 + 0.112+En .Vso=7.0 (MeV) r0=rso-1.24 .rs-1.40 (fm) a0-aso-0.541 .b =0.4 (fm) MT 2 : Elastic scattering Total All the other partial cross sections). MT = 16 : (n, 2n) Ni-58

Data of JENDL 1, evaluated on the basis of plenty experimen tal data, extended up to 20 MeV. Ni 60, 61, 62, 64 Calculated with evaporation model. MT 17 : (n.3n) given only for Ni 64. Calculated with evaporation model. MT 22 : (n.na) given only for Ni 58. Evaluated on the analogy of Cu 65 (n.na) cross section. MT 28 : (n.np) Calculated with evaporation model.

Inelastic scattering to the 1st level for even A isotopes evaluated on the basis of experimental data up to 7 MeV. Direct reaction calculated with DWBA and added above 7 MeV

The level scheme for each isotope taken from Ref./13/. Ni 58 No Energy(MeV) Spin Parity No Energy(MeV) Spin Parity

g.s.	0	0 +	12	3 6206	4 -
1	1.4545	2 + S	13	3.7757	4 +
2	2,4595	4 +	14	3,8989	2 +
3	2,7757	2 +	15	4.1080	2 +
4	2.9026	1 +	16	4.2900	3 (
5	2.9428	0 ·	17	4.3430	6 -
6	3,0383	2 ·	18	4.3490	4 +
17	3.2645	2 ·	19	4.3800	5 -
8	3,4208	3 .	20	4.4010	4 +
9	3,5240	4	21	4.4498	0 +
10	3,5313	C ·	22	4.4720	3 -
11	3.5942	1 +			

continuum levels assumed above 4.517 MeV.

Ni	60	No	Energy MeV	Spin-Parity	No	Energy (MeV)	Spin-Parity
		g.s.	0	0 +	12	3.3183	0 +
		1	1.3325	2 +	13	3,3810	4 +
		2	2.1589	2 +	14	3.3936	2 +
		3	2.2848	0 +	15	3.5300	0 +
		4	2.5058	4 +	16	3,5890	3 +
		5	2,6262	3+	17	3.6197	3+
		6	3.1190	4 +	18	3.6710	4 +
		7	3,1241	2 +	19	3.7290	3 ÷
		8	3,1300	4 ·	20	3.7355	1 +
		9	3.1864	з.	21	3.7410	0 +
		10	3.1941	1 •	22	3.8714	2 +
		11	3.2694	2 -			
			a			0.000 14.1	•

Continuum levels assumed above 3.895 MeV.

Ni-61 No Energy (MeV) Spin-Parity No Energy (MeV) Spin-Parity

g.s.	0	32	11	1 7298	3.2
1	0.0674	5 2	12	1 8080	7.2
2	0,2830	1 2	13	1 9780	9.20
з	0.6560	3 2	1.4	1 9370	3.2
4	0,9088	5.2	15	2.0030	7.2
5	1.0160	7.2	16	2 0190	3.7
6	1.1000	3.2	17	2.1140	9.2 -
7	1.1323	5 2	18	2 1230	12
- 8	1.1857	3.8	19	2.4100	52
9	1.4580	7-2	50	2.4660	72
10	1 6100	52			
	** •			0 100 1	

Continuum levels assumed above 2 528 MeV.

Nı	62	No	Energy MeV	Spin Parity	No	Energy MeV	Spin Parity
		g. s	0	0	11	3 2699	2 ·
		1	1 1729	2	-12	3.2774	4
		2	2 0486	0	-13	3.3703	1 -
		13	2 3018	2	14	- 4620	4 .
		.4	2 3364	1	15	3,4860	O ·
		5	2 8912	0 · ·	-16	3.5185	2 -
		G	3 0582	2.0	17	3,5229	3 -
		- 7	3,1580	2 ·	18	3.7570	3
		8	3.1765	4 +	-19	3.8493	1 4
		- 9	3.2577	12 1	20	3.8530	2.
		10	3.2620	4 -	21	3,8600	5 +
			Continuum	levels assume	d at	ove 3.967 M	eV.

Nı	-64	No	Energy MeV	Spin Parity	No	Energy MeV.	Spin Parit	У
		\mathbf{g} .s	0	0 .	11	3.393	3 (
		1	1.3459	2 ·	-12	3.459	1 •	
		2	2.2750	0 .	3	3.483	4	
		3	2.6080	4	1.1	3.560	3	
		4	2.7500	2	-15	3 647	2.	
		-5	2.8650	0	1G	3,748	4 .	
		6	2,8850	P	17	3.795	1 .	
		7	2 9710	2	18	3 808	3	
		8	3 0280	Ο ·	-19	3,848	5	
		9	3.1650	4.	20	3.965	4 .	
		10	3.2730	2 ·				
			Continuum	levels assume	d at	ove 4.084 M	ΞV.	

The inelastic levels of each isotope are grouped in natural Nickel file as follows :

MT	Q:MeV:	Ni 58	Ni 60	Vi 61	Ni 62	Ni 64
51	0.0674			51		
52	0.2828			52		
53	0.6556			53	-	
54	0.9082			54		
55	1.0144			55		
56	1.0993			56.57		
57	1.1719			58	51	
58	1.3320		51			51
59	1.4549	51		59.60		
				61.62		
60	1.9768	-	-	63,64	52	-
				65,66		

					67,68		
	61	2.1582		52			52
	6?	2 2840		53	689	183.154	
	- 63	2,4601	152		70		
	64	2.5049		54			583
	65	2 6253		55			64
	66	2 7703	53			55	55.56
	67	2.9033	54				
	68	2.9435	55				57.58
	69	3.0390	56			56	
	70	3 1179		56.57		57.58	59
				58			
	71	3.1853		59.60		59,60	
	72	3 2652	57	61		61.62	60
	73	3 3172		62		63	
	7.1	3 3798		GR GJ			61
	ч,	3 426	$\frac{1}{2}$			64.65	62.63
						FB 67	
	<u>4</u>	法法律规	50.075	4 J y			64
	,-	is bevas	151	2.2			65
	25	计科理	637	67			
	79	3 (237		{ 34			
	BO	3 6467		69.70		68	66
				71			
	81	3 7761	63			69,70 71	67.68 69
	82	3 8701		72			~~
	123	3 8968	634				70
	84	4 1089	65				
	85	4 2910	67				
	EG	4 3440	68.69				
	87	1.58.0	10				
	993	1 3020	1				
	64.3	1.15.43	62				
		4 4740	19				
	e pi	2.4442	61	541	01	91	91
Th	$e^{-1}e^{i}$	el densi	tv param	eters ev	Insted	by Yoshi	da 17
	Niii	satope	57	58	59	60	61
	a	1 MeV	5.00	6.45	6.97	7.55	8.14
	Delt	a MeV	1.20	2.47	1.20	2.47	1.20
	Fx	MeV.	6.33	7 30	8 00	10.00	7 00
	Te	MeV	1 44	1 49	1.35	1 26	1 17
	••	1101		1.10	1.00		
	N1 1	sotope	62	63	64	65	
	а	1 MeV	8.77	9.37	9.98	10.57	
	Delt	a MeV	2.60	1,20	2.70	1.20	
	Ex	MeV	9,00	3.00	4,32	4,00	
	Tc	MeV	1.08	1.36	1.15	0.947	
						0.011	
MT-10	2 : C	apture					
Ca	lcula	ted with	the stat	tistical	model co	de CASTI-	Y 12
	Comp	eting pro	cesses	n.2n	n.3n .	n.np).(n	na .
	2 je .					and the second sec	

(n.p., (n.a).

Level fluctuation considered.

The gamma ray strength function obtained from the capture cross section of natural Ni : 9.6 mb at 450 keV. Ni 58 : 4,62E 5 . Ni 60 : 2.77E 5 Ni 61 : 1.94E 4 . Ni 62 : 9,52E 6 N1 64 : 5.67E 6 МГ 103 : п.р.: Ni 58,Ni 60 Data of JENDL 1. evaluated on the basis of plenty experimen tal data, extended up to 20 MeV. N1 61, 62, 64 Evaluated on the analogy of Ni 60 on.p cross section. MT 107 : n.a Evaluated on the analogy of Co 59 n.a cross section. MT 251 : Mu bar Calculated with CASTHY 12 . MF 4 Angular Distributions of Secondary Neutrons S TM : Calculated with optical model. MT 16.17.22.28.91 : Isotropic in laboratory system. MT 51 90 : lsotropic in center of mass system. MF 5 Energy Distributions of Secondary Neutrons MT 16.17.22.28.91 : Evaporation spectrum, References 1 Kikuchi Y. et al.: J.Nucl.Sci.Technol., 17,567(1980). 2 Perev F.G et al.: Neutron Data of Structural Materials for FBR ,1977 Geel Meet, p.503, Pergamon Press (1979 ... 3 Syme D.B. et al.: ibid.p.703. 4 Froehner F. : ibid.p.138. 5 Farrell J.A. et al.: Ann.Phys..37.367 1966 . 6 Hockenbury R.W. et al.: Phys.Rev. 178.1746 (1969). 7 Stieglitz R.G. et al.: Nucl. Phys. A163.592 (1971). B Cho et al.: Nuclear Data for Reactors, Proc. 1970 Helsinki Conf., p.619, IAEA (1970). 9) Beer H. and Spencer R.R.: Nucl. Phys. A240,29 (1975). 10) Kikuchi Y. and Sekine N. : to be published as JAERI M report. 11) Kavai M. : unpublished. 12) Igarasi S.: J.Nucl.Sci.Technol., 12,67 (1975). 13) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Edition, Wiley-Interscience (1978).

14 / Yoshida T. : unpublished.

MAT number 2281 28 Ni 58 28 Ni 58 JAERI Eval Aug82 Y.Kikuchi and N.Sekine Dist Mar83 Rev1 Nov83 History 82 08 New evaluation for JENDL 2 was made by Y.Kikuchi - JAERI - and N.Sekine HEC 83 11 Comment was added. MF 1.MT 451 Comments and dictionary MF 2.MT 151 Resolved resonance parameters : 1.6E 5 eV 660 keV Evaluation based on the following data. Transmission (Perey 1 , Syme 2 ,Farrel 4 , Capture (Perey 1 , Freehner 3 , Hockenbury 5 , Two negative resonances added. Assumed Gam $g \in 2$ eV for s wave res. and 1 eV for p wave. Calculated 2200 m 's values and resonance integrals (barn): 2200 m s value res. int. 30.62 total elastic 20.02 4.605 15.5 capture MF 3 Neutron Cross Sections Background cross sections (BOCS: applied to resonance region. MT 1.2 : Many point BOCS to compensate errors due to constant R and resonance truncation 6 . MT 102 : Smooth BGCS to correct b wave level missing above 200 keV. Cross sections above 600 keV evaluated as follows : MT 1 . Total cross section Calculated with optical model. Potential parameters obtained by fitting nat-Ni data /7/: V 51.33 - 0.331+En ,Ws=8.068 + 0.112+En ,Vso=7.0 (MeV) r0=rso=1.24 (fm) .rs∘1.40 a0 aso 0.541 .b -0.4 (fm) MT=2 : Elastic scattering (Total) - (All the other partial cross sections). MT=16,103 : (n,2n), (n,p)Data of JENDL 1. evaluated on the basis of plenty experimen tal data, extended up to 20 MeV. MT-22: n.n.a/ Evaluated on the analogy of Cu-65(n,na) cross section. MT=28 : .n.n'p) Calculated with evaporation model.

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MT 51 : Inelastic scattering to the 1st level Evaluated on the basis of experimental data up to 7 MeV. Direct reaction calculated with DWBA and added above 7 MeV.

MT 52 72 Caler Co	2.91.102 : Ins) ilated with the opeting processed function	lastic scatter); statistical m mess : n.2n . ; n_considered	ng and capture odel code CASTD n.p , n.a , n.n	17 8 . 17 - n∓na).
The 1	ovel celebrate	dron from Rof 1	1	
3.917. 7	Ma	Environ MAV	Son Divity	
	10	Chergy nev		
	∦5,251. 1	1 452 452	• •	
		1,4040	2 '	
	2	C.4020	4 .	
	:5	2 1151	ć.	
	.1	2.130214	•	
	'>	8.96c2	0	
	i ,	3 0329	21	
		11 11 11		
		15 - 15 - 15 A	ts -	
	. 1	16 (C) 51 (F)	6	
	10	8,5318	0	
	11	8.5942	1 -	
	12	3.6200	4	
	1/3	3.7757	.1 .	
	14	13 BONDA	24.1	
	115	4 1080	2	
	145	4 2000	12	
	10	4 P.194	6	
	1 (2	4 9400	4	
	163	4.0400	4 7	
	10	4.0000	0	
	20	4.4010	4	
	13	4.45.83	12	
		4,476	1.5 • • • • • • • • • •	
	Contamma de	wels essured ab	ove 4.5]7 £eV.	
1. ju - ju	oved, decase (V as	ar obber, en Stor Arol, u	eted by Yoshida	a 10 .
	St isotope	157	1.2 59	
	a ti NeV	5,00 6	-45 6.97	
	Delta MeV	1.20 S	.47 1.20	
	Ex MeV	6.33 7	.30 8.60	
	To MeV	1.44 1	.49 1.35	
The ga the ca	anma ray streng apture cross se	th function of ection of natur	4.62E 5 obtair al Ni.	ied from
MT-107 Evalua	n.a ated on the ana	alogy of Co 59	n.a cross sect	ion.
MT 251 Calcul	Mu bar lated with opti	cal model.		
MF=4 Angular MT:2 MT-16.22.28 MT=51 72 MT=91	Distributions Calculate Isotropic 90 degree system. c 90 degree calculate	of Secondary N ed with optical in the laboral symmetric in f alculated with is symmetric in f ed with CASTHY.	entrons model. tory system. the center of m CASTHY. the laboratory	ess system.
MF 5 Energy Distributions of Secondary Neutrons MT (6.22.28.91 : Evaporation spectrum)

References

- Percy F.G. et al.: Neutron bata of Structural Materials for FBR.1977 Geel Meet. p.503.Pergamon Press 1979
- 2 Syme D.B. et al.: ibid.p.703.
- 3 Freehner F. 1 1bid.p.138.
- 4 Farrell J.A. et al.: Ann. Phys. 37.367 1966 .
- 5 Hockenbury R.W. et al.: Phys.Rev. 178.1746 1969 .
- 6 Kikuchi Y. and Sekine N. : to be published as JAERI M report.
- 7 Kawai M. : unpublished.
- 8 Igarasi S.; J.Nucl.Sci.Technol., 12,67 1975
- 9 Lederer C.M. and Shirley V.S. Table of Isotopes.7th Edition. Wiley Interscience 1978

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10 Yoshida T - anyablished

1 of Nickel 60

28 Ni 60 MAT number 2282

28 Ni 60 JAERI Eval Aug82 Y Kikuchi and N Sekine Dist Mar83 Rev1 Nov83

History

- 82 08 New evaluation for JENDL 2 was made by Y Kikuchi JAERI and - N Sekine - HEC .
- 83-11 Comment was added.

MF 1.MT 451 Comments and dictionary

MF 2.MT 151 Resolved resonance parameters 1.0E 5 eV 600 keV Evaluation based on the following data. Transmission Syme 1 "Stieghtz: 3 "Farrel: 4.

Cantures the boots of Streetitz 3 (Hockenbury) 5. A negative resonance $1/3 {\rm eV}$ for p wave. Assumed Gam $\mu < 2 {\rm eV}$ for s wave res. and 1 eV for p wave.

Calculated 2200 m s values and resonance integrals (barn): 2200 m s value res. int.

total	3.813	
elastic	1.013	
capture	2 801	1.50

MF 3 Neutron Cross Sections

Background cross sections BCCS applied to resonance region.

- MT 1.2 Many point BCCS to compensate errors due to constant ${\rm R}$ and resonance truncation $|6\rangle$.
- MT 102 1 Smooth POCS near 12 5 keV and above 150 keV.

Cross sections above 600 keV evaluated as follows :

MT 1 . Total cross section

Calculated with optical model.

Potential parameters obtained by fitting nat Ni data 7 : V 51.33 0.331 En .Ws 8.068 + 0.112 En .Vso 7.0 (MeV) r0 rso 1.24 .rs 1.40 (fm a0 aso 0.541 .b 0.4 (fm -

MT 2 : Elastic scattering Total All the other partial cross sections .

- MT 16.28 : n.2n and n.np Calculated with evaporation model.
- MT 51 : Inelastic scattering to the 1st level Evaluated on the basis of experimental data up to 7 MeV. Direct reaction calculated with dwba and added above 7 MeV

MT 52 72.91.102 Inelastic scattering and capture Calculated with the statistical model code CASTHY 78 . Competing processes : (n,2n), (n,p), (n,a), (n,np). Level fluctuation considered. .

1	The level	scheme ta	iken from Ref	9.	
		No	Energy Me	W Spin	Parity
		Р. 5 .	0	0	i -
		1	1 3325	2	
		2	5 1289	2	1
		3	2.2848	0	1
		4	2,5058	4	
		5	2.0202	3	•
		0	(3.14Q) (2.16A)	4	4
		μ Ω	3.1241	C A	1
		0	0 1000 9 1004	4	•
		10	3 1004	1	
		11	3 260/	2	•
		1:2	3 3183	í)	
		13	6.0460	4	
		1	52.00	2	
		1.5	12 12200	0	
		16.	18 Ex 200	3	
		• 7	3 14:14	3	,
		18	3.6710	4	,
		19	3 77290	3	1
		20	3 7355	1	t
		21	3 7410	0	t
		22	3 8714	2	1
	Con	tinuum le	vels assumed	above 3,89	5 MeV.
1	he level (lensity p	arameters ev	aluated by	Yoshidu 104.
	N1 12	solope	09	- 60 17 EE	
	a Dolta	<1 JPE2V S_MUSV	1 20	1.00	0.14
	Ev Der u	MaV	1.c0 8.00	10.00	7.00
	To	MaV	1 90	1.26	1.17
T	ha ramma r	av strong	th turetion	of 2 31F 5	obtained from
t }	he capturi	n moss s	etion data	of Einst 1	1
100 10					
PH IU	n = n = n			the burners	Culoute
Li	ita di Jir	4174	allated on	the pasts of $1 \sim 20$ MeV	1 prency
ez	oper imen ca	i udid .	excended up	to zo nev.	
MT 16	7 :				
Fa	valuated c	n the ana	Joev of Co 5	59(n,a) cros	ss section
2.	uruutee e	in the and			55 500010.1.
MT-25	51 : Mu-h	ar			
Ca	louiated	with opti	cal model		
MF=4 Angu	ılar Distr	ibutions	of Secondary	Neutrons	
MT=2	: Cal	culated w	ith optical	model.	
MT=16,28	3 : Iso	tropic in	the laborat	ory system.	
MT-51 72	2 : 90	degree sy	mmetric in t	he center-o	of-mass system,
_	cal	culated w	1th CASTHY.		
MT- 91	: 90	degree s y	mmetric in t	he laborato	ory system,
	cal	culated w	ith CASTHY.		
	D: -+:		6	Mautuona	

MF-5 Energy Distributions of Secondary Neutrons MT=16,28,91 : Evaporation spectrum.

- 1) Syme D.B. et al.: Neutron Data of Structural Materials for FBR, 1977 Geel Meet., p. 703.Pergamon Press 1979...
- 2) Froehner F. : ibid.p.138.
- 3) Stieglitz R.G. et al.: Nucl. Phys. A163,592 (1971).
- 4) Farrell J.A. et al.: Ann.Phys. 37,367 (1966).
- 5) Hockenbury R.W. et al.: Phys.Rev., 178, 1746 (1969).
- 6) Kikuchi Y, and Sekine N. : to be published as JAERI-M report.
- 7) Kawai M. : unpublished.
- 8) Igarasi S.: J.Nucl.Sci.Technol..12.67 (1975).
- 9) Lederer C.M. and Shirley V.S.: Table of Isotopes,7th Edition, Wiley-Interscience (1978).
- 10 Yoshida T. : unpublished.
- 11) Ernst A. et al.: Nuclear Data for Reactors, Proc. 1970 Helsinki Conf. p.633.IAEA 1970

25 Ni 61 MAT number 2283

28 Ni 61 JAERI Eval Aug82 Y.Kikuchi and N.Sekine Dist Mar83 Rev1 Nov83

History

- 82-08 New evaluation for JRNDL 2 was made by Y.Kikuchi (JAERI) and N.Sekine (HEC).
- 83 11 Comment was added.
- MF 1.MT 451 Comments and dictionary

MF 2.MT 151 Resolved resonance parameters : 1.0E 5 eV 68.5keV Evaluation based on the following data. Transmission : Cho 1 Capture : Freehner 2 Hockenbury: 3 A negative resonance added.

Assumed Gam $g \, \stackrel{<}{\scriptstyle{\sim}}\, 2 \,\, eV$ for s wave res. and 1 eV for p wave.

Calculated	2200 m s values and resonance	integrals	(barn):
	2200 m s value	res.int	
total	15.15		
elastic	9,611	1.1	
capture	2,500	2.44	

MF 3 Neutron Cross Sections

Background cross sections (BCCS) applied to resonance region. MT 1.2 : Many point BCCS to compensate errors due to constant

- R and resonance truncation 4.
- MT 102 : Smooth BGCS above 40 keV.

Cross sections above 68.5 keV evaluated as follows :

ΜT	1 : Total cross section	n		
	Calculated with optical	l model.		
	Potential parameters of	stained by	fitting nat-Ni data	/5/:
	V ⊴51.33 - 0.331⊹En	Ws 8.068	+ 0,112+En ,Vso=7.0	(MeV)
	r0-rso 1,24	rs 1.40		(fm)
	a0_aso-0.541	.b ⇔0.4		(fm)

MT=2 : Elastic scattering (Total) - (All the other partial cross sections).

MT=16,28: (n,2n) and (n,np) Calculated with evaporation model.

 $\begin{array}{rrrr} \text{MT=51-70.91,102}: \text{ Inelastic scattering and capture} \\ \text{Ce`culated with the statistical model code CASTHY /6/.} \\ \text{Competing processes : `n.2n...(n.p)...(n.a)...(n.np).} \\ \text{Level fluctuation considered.} \\ \text{The level scheme taken from Ref./7/.} \\ \text{No} & \text{Energy (MeV)} & \text{Spin-Parity} \\ \text{g.s.} & 0 & 3/2 - \\ 1 & 0.0374 & 5/2 - \end{array}$

	2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18	0.2830 0.6560 0.9088 1.0150 1.1000 1.1323 1.1857 1.4580 1.6100 1.7298 1.8080 1.9780 1.9970 2.0030 2.0190 2.1140 2.1230	1,22,22,22,22,22,22,22,22,22,22,22,22,22	
The The the	19 20 Continuum le level density p Ni~isotope a (1/MeV) Delta(MeV) Ex (MeV) Tc (MeV) gamma~ray stren capture cross s	2.4100 2.4680 vels assumed a arameters eval 60 7.55 2.47 10.00 1.26 gth function o ection data of	5.2 7.2 bove 2.528 M uated by Yos 61 8.14 8 1.20 2 7.00 9 1.17 1 of 4.65E-4 ob Ernst / /10/	1eV. 62 62 63,777 2.60 0.00 .08 vtained from
MT 103 Eval MT 107 Eval	: (n,p) uated on the and : (n,a) uated on the and	alogy of Ni 60 alogy of Co 59	(n _i p) cross	section. section.
MT 251 Calc	: Mu bar ulated with opti	ical model.		
MF=4 Angula: MT=2 MT=16,28 MT=51-70 MT=91	 Distributions Calculated w Isotropic ir 90 degree sy calculated w 90 degree sy calculated w 	of Secondary N with optical mo the laborator wmmetric in the with CASTHY. wmmetric in the with CASTHY.	Neutrons odel. ry system. ≥ center-of-r ≥ laboratory	nass system, system,
MF=5 Energy MT=16,28,91	Distributions o : Evaporation	of Secondary Ne spectrum.	eutrons	
References 1) Cho et al Conf., p. 2) Froehner 1977 Geel 3) Hockenbur 4) Kikuchi Y	: Nuclear Data 619. IAEA (1970) F. : Neutron Da Meet.,p.138,Pe y R.W. et al.: 1 '. and Sekine N.	for Reactors. ta of Structur rgamon Press (1 Phys.Rev.,178, to be publi	Proc. 1970 al Materials 979). 1746 (1969). shed as JAEF	Helsinki for FBR. MI-M report.

- 5 Kawai M. : unpublished.

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- 8) Yoshida T. : unpublished.
- 9) Ernst A. et al.: Nuclear Data for Reactors. Proc. 1970 Helsinki Conf., p.633, IAEA (1970),

28 Ni - 62 MAT number - 2284

28 Ni 62 JAERI Eval Aug82 Y.Kikuchi and N.Sekine Dist Mar83 Rev1 Nov83 History 82 08 New evaluation for JENDL 2 was made by Y.Kikuchi (JAERI) and N.Sekine (hec). 83 11 Comment was added. MF 1.MT-451 Comments and dictionary MF 2.MT 151 Resolved resonance parameters : 1.0E 5 eV 600 keV Evaluation based on the following data. Transmission : Beer 1 .Farrel 2 . Beer 1 Capture Assumed Gam g : 2 eV for s wave res. and 1 eV for p wave. Calculated 2200 m s values and resonance integrals (barn): 2200 m s value Res. Int 23.70 total elastic 9.505 capture 14.20 6.91 MF 3 Neutron Cross Sections Background cross sections (BOCS) applied to resonance region. MT 1.2 : Many point BGCS to compensate errors due to constant R and resonance truncation 3. MT 102 : Smooth BGCS above 150 keV. Cross sections above 600 keV evaluated as follows : MT 1 : Total cross section Calculated with optical model Potential parameters obtained by fitting nat Ni data /4/: V 51.33 0.331 En Ws 8.068 0.112(En Vsor7.0 (MeV) r0-rso-1.24 ,rs-1.40 (fm) a0-aso 0.541 .b 0.4 (fm) MT-2 : Elastic scattering Total : - (All the other partial cross sections). MT = 16,28 : (n,2n) and (n,np)Calculated with evaporation model. MT-51 : Inelastic scattering to the 1st level Evaluated on the basis of experimental data up to 7 MeV. Direct reaction calculated with DWBA and added above 7 MeV. MT=52 71.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /5/. Competing processes : (n.2n), (n.p), (n.a), (n.np). Level fluctuation considered. The level scheme taken from Ref. /6/.

Ňo	Energy MeV	Spin Parity
g.s.	0	0
2	2 0486	
5	2.3018	2.
4	2.3364	4 .
5	2.8915	() +
6	3.0582	S ·
7	3.1580	2 ·
8 0	3,1703	4 ·
10	3 2620	
11	3.2699	2
12	3.2774	4 .
13	3.3703	1 ·
14	3.4620	4
5	3.4850	0 ·
. 7	3,5125 0,5000	2
- / 18	3.0643	2000 19
19	3.8493	1 +
20	3.8530	2 (
21	3,8600	2 +
Continuum 1e	vels assumed ab	ove 3,967 MeV.
The level density p	arameters evalu	ated by Yoshida /7/
NI ISOLOPE	- 101 - 10 - 101 - 10"	≫2 003 77 0.97
Delta (MeV)	120 2 F	so 1.20
Ex (MeV)	7.00 9.0	00 3.00
Te (MeV)	1.17 1.0	08 1.36
The gamma ray stren the capture cross s	gth function of lection data of [1.38E 5 obtained from Secr. 1
MI 103 : h.p Evaluated on the an	alogy of Ni 60 r	n.p cross section.
MT-107 : n.a.		
Evaluated on the an	alogy of Co 59 m	a.a) cross section.
MT-251 : Mumbar Calculated with opt	ical model.	
ME-4 Angulan Distributions	of Secondary No	utrong
MT=2 : Calculated	with optical mod	el.
MT=16,28 : Isotropic in	n the laboratory	system.
MT=51-71 : 90 degree s	ymmetric in the	center-of-mass system.
calculated 1	with CASTHY.	
MT=91 : 90 degree sy calculated v	ymmetric in the with CASTHY.	laboratory system.

MF=5 Energy Distributions of Secondary Neutrons MT=16.28.91 : Evaporation spectrum.

- 1) Beer H. and Spencer R.R.: Nucl. Phys., A240, 29 (1975).
- 2) Farrell J.A. et al.: Ann. Phys., 37, 367 (1986).

- 3) Kikuchi Y. and Šekine N. : to be published as JAERI M report.

- 4) Kawai M. : unpublished.
 5) Igarasi S.: J.Nucl.Sci.Technol., 12.67 (1975).
 6) Lederer C.M. and Shirley V.S.: Table of Isotopes.7th Edition, Wiley Interscience (1978).
- 7) Yoshida T. : unpublished.

1 of Nickel 64

(fm)

(fm)

28 Ni 64 MAT number 2285 28 Ni 64 JAERI Eval Aug82 Y.Kikuchi and N.Sekine Dist. Mar83 Rev1 Nov83 History 82-08 New evaluation for JENDL 2 was made by Y.Kikuchi (JAERI) and N. Sekine (HEC). 83 11 Comment was added. MF 1.MT 451 Comments and dictionary MF 2.MT 151 Resolved resonance parameters : 1.0E 5 eV 600 keV Evaluation based on the following data. Transmission Beer 1 .Farrel 2. Canture . Peer Assumed Cam g 2 eV for s wave res. and 1 eV for p wave. Calculated 2200 m s values and resonance integrals (barn): 2200 m s value Res.Int 1.515 total 0.003464 elastic 1.480 capture. 0.819 MF 3 Neutron Cross Sections Background cross sections (BCCS) applied to resonance region. MT 1.2 : Many point bgcs to compensate errors due to constant R and resonance truncation 3 . MT 102 : Smooth bgcs above 370 keV. Cross sections above 600 keV evaluated as follows : MT-1 : Total cross section Calculated with optical model. Potential parameters obtained by fitting nat Ni data /4/: V 51.33 0.331 En Ws 8.068 0.112 En Vso 7.0 (MeV) r0-rso=1.24 .rs=1.40 a0 aso 0.541 ,b 0.4 MT-2 : Elastic scattering (Total) - (All the other partial cross sections). MT=16.17 : (n.2n) and (n.3n) Calculated with evaporation model. MT-51 : Inelastic scattering to the 1st level Direct reaction calculated with DWBA, added above 7 MeV MT-52-70.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /5/. Competing processes : (n,2n), (n,3n), (n,p), (n,a). Level fluctuation considered. The level scheme taken from Ref. /6/.

Energy (MeV) Spin-Parity

No

g.s. 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 Continuum leve	0 1.3459 2.2750 2.6080 2.7500 2.8650 2.9710 3.0280 3.1650 3.2730 3.393 3.459 3.483 3.560 3.647 3.748 3.705 3.808 3.848 3.965 els assumed	0 2 0 4 2 0 2 2 0 4 2 3 1 4 3 2 4 3 5 4 above 4.08	+ + + + + + + + + + + + + + + + + + +	
The level density par	ameters eva	aluated by	Yoshida /	7/
Ni~isotope	62	63	64	65
Delta (MeV)	2.60	1.20	2.70	1.20
Ex (MeV)	9,00	3.00	4.32	4.00
Tc (MeV)	1.08	1.36	1,15	0.947
the conture cross see	n function	of 7.5715-5 C Boord 17	optained	rom
	CION GUEG C		•	
MT-103 : (n,p) Evaluated on the anal	ogy of Ni 6	80×n.p∘cro	ss sectio	on.
MT 107 : n.a				
Evaluated on the anal	ogy of Co 5	9 n.a : cros	ss sectio	n.
Mf≈251 : Mu~bar Calculated with optic	al model.			
MF=4 Angular Distributions of	f Secondarv	Neutrons		
MT=2 : Calculated with	th optical	model.		
MT=16.17 : Isotropic in 1	the laborat	ory system.		
MI=DI-70 : 90 degree symmetry calculated with	netric in t th CASTHY	ne center-c	n-mass s	ystem,
MT=91 : 90 degree sym	metric in t	he laborato	ory syste	m,
calculated with	th CASTHY.			
MF=5 Energy Distributions of MT=16,17,91 : Evaporation sp	Secondary 1 Sectrum.	Neutrons		
D. Courses				
1 Beer H and Spencer R R	Nucl Dhus	A240.29 (1	975	
2) Farrell J.A. et al.: Ann.F	hys.,37,36	7 (1966).		
3) Kikuchi Y. and Sekine N. :	to be pub	lished as J	AERI-M re	eport.
4) Kawai M. : unpublished.	•			

- 5) Igarasi S.: J.Nucl.Sci.Technol., 12,67 (1975).
 6) Lederer C.M. and Shirley V.S.: Table of Isotopes,7th Edition, Wiley-Interscience (1978).
- 7) Yoshida T. : unpublished.

1 of Natural Copper

29-Cu- 0 MAT number - 2290

29 Cu – O JAERI,MAPI Eval Mar82 S.Igarasi,M.Sasaki Dist Mar83 Rev1 Jan84

History

75-03 Evaluation was made for JENDL 1 by M. Sakaki (MAPI).

- 83 03 Data in the energy region above 35 keV were re evaluated for JENDL 2 by S.Igarasi (JAERI), and the background data in the resonance region were modified.
- 84 01 Comment was added.
- MF-1 General Information MT-451 Descriptive data and dictionary
- MF 2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW formula Resonance region = 1.0E 5 eV to 35 keV. Parameters of each isotope were mainly taken from BNL 325 3rd edition 1%. A bound level was added to reproduce the 2200 m sec capture cross section of each isotope. The effective scattering radius of 6.70 fm was taken from Ref. - 27 for Cu 63 and 6.56 fm -1 for Cu 65.

Calculated 2200 m s cross sections and res. integrals.

	2200 m i	s	res, integ.
elastic	7.659	b	
capture	3.775	b	3.40 b
total	11.63	Ь	

MF 3 Neutron Cross Sections

Below 35 keV, all background cross sections are zero. Above 35 keV, data were constructed from isotope data which were evaluated as follows.

MT 1 Total

Optical and statistical model calculation was made with code CASTHY 3. The optical potential parameters used for both isotopes are as follows (in the units of MeV and fermi), $V = 46.0 - 0.250 E_{1}$, R0 = 1.16(A)+1/3(0.6, a0 = 0.62) Wi =0.125+E-0.0004+E++2. Ri = 1.16+A++1.3+0.6, ai = 0.62 ₩s ≈ 7.0 Rs = 1.16(A+1)(3+1.1), as = 0.35, Rso- 1.16+A++1/3+0.6, aso= 0.60 Wso= 7.0 Elastic scattering MT=2 (Total)-(All other partial cross sections) MT=4.51-67.91 Inelastic scattering Optical and statistical model calculation was made by taking into account of competing processes of n.2n). (n.n'a) and 'n.a' reactions for Cu-63, and m.2n . n.n.a and m.p. for

Cu-65. The level schemes were taken from Ref. 4.,

Cu	nat	Cu -83		Cu~65	
MT	Energy MeV	Energy MeV	/ Jp	Energy Me	V J-p
51	0.6698	0.66962	12		-
52	0.7704			0.7706	1/2 -
53	0.9623	0,96206	5/2 -	-	

2 of Natural Copper

.:

54	1.1153	1.00700	7 0	1.11554	5-2
55 56	1.3274	1.32703	52		
57	1.4815	1.11200	0 1.	1.48183	7.2
58	1.5475	1.54702	32		
59	1.6231			1.62344	5/2
60	1.7246			1.72494	3.2
61	1.8618	1.8613	52		
62	2.0117	2.0111	32		
63	2.0926			2.093	52
64	2.2118			2.2123	3/2
65	2.2795			2.280	5,2
66	2.3277			2.3282	3.2 .
67	2.5305			2.531	92+
91	2.0506	above 2.05		above 2.54	

MT 16.22.103.107 n.2n., n.n.a., n.p. and n.a. Recommended by Kobavashi 5.

MT 102 Capture Calculated with CASTHY 3 and normalized to the value of 23.7 milli barns for Cu-63 and 11.9 milli barns for Cu-65 at 230 keV which were measured by Zaikin 6. MT 251 Mu bar

Calculated with CASIHY 3

MF 4 Angular Distributions of Secondary Neutrons

MT 2 Calculated with CASTHY 3.

MT 16.22.91 : Assumed to be isotropic in the lab system.

- MT-51.67 : Assumed to be isotropic in the center-of-mass system.
- MF-5 Energy Distributions of Secondary Neutrons MT 16.22.91 Evaporation spectra.

- 1 Mughabghab S.F. and Cerber D.1.: DNL 325, 3rd Ed., vol. 1 1973.
- 2 Mughabghab S.F. et al.: Neutron Cross Sections, Vol. 1, Part A", Academic Press (1981).
- 3) Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975).
- 4) Auble R.L.: Nuclear Data Sheets, 14, 119 (1975).
- 5) Kobayashi K.: private communication.
- 6 Zaikin G.G. et al.: Atomnaya Energiya, 25, 526 (1968),

1 of Copper 63

29 Cu 63 MAT number 2291

29 Cu 63 JAERI MAPI Eval Mar82 S.1garasi M.Sasaki Dist Mar83 Revi Jan84

History

- 75 03 Evaluation was made for JENDL 1 by M.Sakaki MAPL.
- 83 03 Data in the energy region above 35 keV were re-evaluated for JENDL 2 by S.Igarasi JAERI, and the background data in the resonance region were modified.
- 84 01 Comment was added.
- MF 1 General Information

MT 451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT 151 Plass'ved recommendation became ters for MLBW formula Resonance region - 1 OF 5 eV to 35 keV.

Parameters were mainly taken from BNL 325 3rd edition -1. A bound level was added to reproduce the 2200-m sec capture cross section of 4.5 ± -0.1 barns -1. The effective scattering radius of 0.67 fm was taken from Ref. 21.

Calculated 2200 m s cross sections and res, integrals.

	2200 m s	res. integ.
elastic	4,979 b	
capture	4,492 b	5.41 b
total	9.471 b	

MF 3 Neutron Cross Sections

Below 35 keV. all background cross sections are zero. Above 35 keV, data vere evaluated as follows. MT Total Optical and statistical model calculation was made with code CASTHY 3. The optical potential parameters used are as follows in the units of MeV and fermi . 46.0 0.250 E . R0 1.16 A H 3 O.6, a0 0.62 v Wi 0.125 E 0.0004 E+2. Ri 1.16+A++1 3+0.6. ai 0.62 . Rs 1.16+A++1/3(1.1, as Ws 7.0 0.35 Wso 7.0 . Rso 1.16+A++1.3+0.6. as 0.60 MT 2 Elastic scattering Total - All other partial cross sections -MT 4.51-57.91 Inelastic scattering Optical and statistical model calculation was made by taking into account of (n.2n), (n.n'a) and (n.a) reactions as competing processes. The level scheme was taken from Ref. 47. Energy MeV Spin Parity No. 0.0 32 5.S. 0.66962 12 1 2 0.96206 5.2 .32708 70 3 1.41203 52 4 5 1.54702 32-6 1.8613 5/2 --

7 2.0111 32 Levels above 2.05 MeV were assumed to be overlapping. MT 16,22,107 n.2n ., n.n'a and n.a Recommended by Kobayashi 5. MT 102 Capture Calculated with CASTHY 3 and normalized to the value of 20.7 milli barns at 230 keV which was measured by Zaikin 6. MT 251 Mu bar Calculated with CASTHY 3 MF 4 Angular Distributions of Secondary Neutrons MT 2.51 57 Calculated with CASTHY 3. MT 16.22 : Assumed to be isotropic in the lab system. MT 91 : Calculated with CASTHY in the center of mass system, and assumed to be the same in the laboratory system ME 5. Energy Distributions of Merondary Neutrons

M. 16.22 Mr. Bar Litten station

- 1 Mughabghab S.F. and Garber D 1. PNL 325, 3rd Ed., vol. 1 1973.
- 2 Mughabghab S.F. et al.: Neutron Cross Sections, Vol. 1, Plat A , Academic Press 1981 .
- 3 Igarasi G. J. Nucl. Sci. Technol . 12, 67 (1975).
- 4 Auble R.L., Nuclear Data Sheets, 14, 119 1975).
- 5 Kobayashi K : private communication
- 6 Zaikin G.G. et al. Atomnaya Energiya, 25, 526 (1968).

29-Cu- 65 MAT number 2292

29 Cu 65 JAERI MAPI Eval Mar82 S.Igarasi M.Sasaki Dist Mar83 Revi Jan84

History

- 75 03 Evaluation was made for JENDL 1 by M.Sakaki (MAPI).
- 83 03 Data in the energy region above 35 keV were re-evaluated for JENDL 2 by S.Igarasi (JAERI), and the background data in the resonance region were modified.
- 84 01 Comment was added.
- MF 1 General Information

MT 451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW formula Resonance region = 1.0E 5 eV to 35 keV. Parameters were mainly taken from BNL 325 3rd edition ± 1 . A bound level was added to reproduce the 2200 m/sec capture cross section of 2.17 + 0.03 barns $\pm 1/2$. The effective scattering radius of 6.56 fm was taken from Ref. $\pm 1/2$.

Calculated 2200 m s cross sections and res. integrals.

	2200 m s	res. integ.
elastic	14.30 Б	
capture	2.170 Б	2.13 b
total	16.47 b	

MF 3 Neutron Cross Sections

Below 35 keV, all background cross sections are zero. Above 35 keV, data were evaluated as follows. MT 1 Total Optical and statistical model calculation was made with code CASTHY 2 The optical potential parameters used are as follows in the units of MeV and fermi /. ν 46.0 0.250 E , R0 1.16 A+11/3+0.6, a0 0.62 Wi 0.125+E 0.0004+E++2, Ri 1.16+A++1/3+0.6, ai 0.62 Ws 7.0 Rs = 1.16 + A + + 1/3 + 1.1, as = 0.35Wso 7.0 -, Rso≈ 1.16+A++1/3+0.6, aso≈ 0.60 MT 2 Elastic scattering Total All other partial cross sections) MT-4.51-60.91 Inelastic scattering Optical and statistical model calculation was made by taking into account of n.2n). n.n'a and (n.p) reactions as competing processes. The level scheme was taken from Ref. /3/. Energy MeV Spin Parity No. 0.0 g.s. 32. 0.7706 1.2 1 1.11**55**4 1.48183 5.2 2 12 З 1.62344 52 4 5 1.72494 3/2 -6 2.093 5/2 -

7 2.2123 32 52 8 2.280 2.3282 3.2 -9 10 2.531 921 Levels above 2.54 MeV were assumed to be overlapping. MT 16,22,103 (n,2n), (n,n'a) and (n,p)Recommended by Kobayashi 4 MT 102 Capture Calculated with CASTHY 2 and normalized to the value of 11.9 milli barns at 230 keV which was measured by Zaikin 5. MT 251 Mu bar Calculated with CASTHY 2. MF 4 Angular Distributions of Secondary Neutrons MT 2.51 60 Calculated with CASTHY 2 ... : Assumed to be isotropic in the lab system. MT 16.22 MT 91 Calculated with CASTHY in the center of mass system, and assumed to be the same in the

MF 5 Energy Distributions of Secondary Neutrons MT 16.22,91 : Evaporation spectra.

laboratory system.

- 1 Mughabghab S.F. and Garber D.1.: PNL 325, 3rd ed., vol. 1 1973 \.
- 2 Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975).
- 3 Auble R.L.: Nuclear Data Sheets, 14, 119 (1975).
- 4 Kobayashi K.: private communication.
- 5 Zaikin G.G. et al.: Atomnaya Energiya, 25, 526 (1968).

41-Nb- 93 MAT number = 2411 41 Nb 93 JNDC Eval Oct82 JNDC FPND W.G. Dist Mar83 Rev1 Nov83 History 82 10 New evaluation for JENDL 2 was made by W.G. on FP nuclear data of JNDC. 83 11 Small modification was made and comment was added. MF 1 General Information MT-451 Comments and dictionary MF 2.MT 151 Resonance Parameters Resolved resonances : 1.0E 5 eV 7 keV Evaluated by M.Kawai on the basis of the following data. Transmission : Garge 1 , Poittevin 2 , Iliescue 6/Scattering - Iliescu- 6 Macklin 3, Lopez 4, Iliescu 6/ Capture Gamma spectrum: Haster 5. Assumed Gam g : 172 milli eV 212 milli eV for doublet Unresolved resonances : 7 keV 50 keV Energy independent parameters are given: S0 0.37E 4 .S1 5.48E 4 .S2 3.65E 4. Dobs 80.5 eV .Gam g 0.160 eV.R 6.70 fm. Calculated 2200 m s values and resonance integrals (barns) : 2200 m s value res.int. 7.477 tota] 6.326 elastic 9.59 capture 1.152 MF 3 Neutron Cross Sections Slight background correction for Sig t and Sig c between 30 and 50 keV. Cross sections above 50 keV evaluated as follows; MT 1 : Total Calculated with optical model. Potential parameters obtained by systematic fitting to Sig-t by Iijima+ 7 : V 46.0 - 0.25+En ,₩s=7.0 .Vso::7.0 (MeV) R0=Rso-5.89 .Rs=6.39 (**fm**) ,b ∽0.35 a0-aso-0.62 (fm) MT=2 : Elastic scattering (Total - All other partial cross sections). MT=16.103.107 : (n,2n), n.p., n.a Calculated by Y.Kanda with GNASH code 8 considering preequilibrium process. MT=51-57.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /9/. Competing processes : (n,2n), (n,p), (n,a)

Level flucti The level schen No 6.s. 1 2 3 4 5 6 7 8 9 10 11 12 13 Continuu	ation considere e taken from Re Energy F 0.0 0.0304 0.6860 0.7440 0.8067 0.8067 0.8101 1.0826 1.2800 1.2974 1.3156 1.3640 n levels assume	d. fr. 10 fcY Spir () () 7 7 8 9 13 9 13 9 13 9 13 9 13 9 13 9 13 9 13	$\begin{array}{c} 1 \text{ Part } \\ 1 Part$	l y
The level densi	tv parameters e	valuated by	/ Tiiim	1a+ 11
Nb isotop	e 91	92 0	3	94
,, 1.№	A 0.01	10.39 12	1.5	12.81
Diel Lei Ple-V Lie Meitz	0,02	0,0 0 4 115 4	0.12	0.0
Te MeV	0.655	0.641 0	719	0.723
The gamma ray s reproduce the OL Mt 251 Mu bar Calculated with	.rength function WLA capture da opticalmodel.	n of 197E ta 3.	4 obta	ined so as to
MF 4 Angular Distrik MT 2 : Calcula MT 16 : Isotro MT 51 57 : 90 deg MT 91 : 90 deg	nutions of Secon ited with optica bic in the labor ree symmetric in ree symmetric in	ndary Neutro al model, ratory systen i the center i the labora	ons em. r of ma atory :	ass system. system.
MF 5 Energy Distribu MT 16.91 Evapore	tions of Second tion spectrum,	lary Neutron	ns	
References 1 Garg J.B. et al.: 2: Poittevin G.le. e 3: Macklin R.L.: Nuc 4: Lopez W.M. et al. 5: Haste T.J. and Th 7: Iijima S. and Kaw 8) Young P.G. and Ar 9: Igarasi S.: J. Nu 10: Lederer C.M. and Wiley-Interscience 11: Iijima S.et al.:	Phys. Rev., BJ t al.: Nucl. Ph 1. Sci. Eng., 5 : Nucl. Phys., omas B.W.: J. F ai M.: J. Nucl. thur E.D.: LA-6 cl. Sci. Techno Shirley V.S.: T e (1978). to be published	37. 547 19 lys 70. 49 l9. 12 (1976 A93. 340 (1 Physics. G1. Sci. Techr 1947 (1977). l 12. 67 able of Isc in J. Nucl	965). 97 (196 3). 1967). 967 (101., 2 (1975) 0topes. Sci.	65). (1975). 20, 77 (1983).). .7th Edition, Technol.

42-Mo- 0 MAT number = 2420

42 Mo O JNDC Eval Aug82 Y.Kikuchi et al. Dist Mar83 Rev1 Nov83 History

82 08 New evaluation was made by W.G. on FP nuclear data of JNDC. Natural Mo data constructed with data of Mo isotopes. 83 11 Comment was added. MF 1.MT 451 Comments and dictionary MF 2.MT 151 Resonance parameters : 1.0E 5 eV 100 keV Resolved resonances for MLBW formula. Evaluation based on the following data. Mo 92 Transmiss. Wasson 1 Capture Wasson 1, Weigmann 2, Musgrove 3 Capture Weigmann 2, Musgrove 3 Mo 94 Cat/ture Mo 95 Transmiss. : Shwe+ 4 Capture : Weigmann: 2 Mo 96 Capture : Weigmann: 2 .Musgrove: 3 Mo 97 Transmiss. : Shwee 4 Capture : Weigmann+ 2 Mo 98 Transmiss. : Chrien, 5 Cupture : Weigmann 2 .Musgrove 3 Mo 100 Transmiss. : Weigmann+ 6 Capture : Weigmann 2 .Musgrove: 3/ A negative resonance added for Mo 95, 97, 98 and 100. Connecting energy between resolved and unresolved resonances Mo 92 : 50 keV , Mo 94 : 20 keV , Mo 95 : 2 keV , Mo 96 : 19 keV . Mo 97 : 1.8 keV . Mo 98 : 32 keV . Mo 100: 26 keV Unresolved resonances up to 100 keV Energy independent parameters calculated with optical and statistical models are given. S0 0.37E 4 ... 5.48E 4 ... 5.65E-4.Calculated 2200 m s values and resonance integrals (barn) : 2200 m/s value Res.Int. 8.037 total 5.486 elastic 2.551 25.4 capture MF-3 Neutron Cross Sections

Slight background correction are applied for Sig-t and Sig-c in the unresolved resonance region.

Cross sections above '00 keV evaluated as follows

MT=1 : Total Calculated with optical model. Potential parameters obtained by systematic fitting of Sig-t

by	Iijima+7			
	V 46.0 0.25+En	Ws 7.0	.Vso 7.0	(MeV)
	R0-Rso-5.89	.Rs 6.39		ſm
	a0 aso 0.62	.ь 0.35		- £m

- MT 2 : Elastic scattering (Total) (All other partial cross sections).
- MF 51-91 : Inelastic scattering Calculated with the statistical model code CASTHY 9 . Competing processes : n.2n . n.3n . n.p . n.a Level fluctuation considered.

The level scheme taken from Ref. 10 for Mo 92 and 94 and from evaluation by Macumoto- 11 for the other isotopes.

Mo	92	No	Energy	MeV	Spin	Parity	No	Energy MeV	/ Spin	Parity
		g.s.	0		0	4	4	2.5270	5	er.
		1	1.509	95	2	1	5	2.6130	6	1
		ຂ	2.28	26	4	4	6	2.7600	8	ł
		3	2.519	97	0	+	7	2.8497	3	
			Continu	uum 1e	evels	assumed	abov	e 3 MeV,		

Mo	94	No	Energy (Me)	🗥 Spin	Parity	No	Energy (MeV)	Spin Parity
		g.s.	. 0	0	1	6	2.2940	4 +
		1	0.8710	2	1	- 7	2.3930	2 +
		- 2	1.5737	- 4	1	- 8	2.4230	6 -
		- 3	1.7420	0		9	2.5337	3
		4	1.8542	2		10	2.5670	4 -
		5	2.0874	2		11	2.6100	5
			Continuum	levels	assumed	abov	e 2.74 MeV.	

Mo 95	No	Energy Me	V Spin Parity	y No D	Energy MeV	🗇 Spin I	Parity
	g.s.	0	52+	10	1.3100	1/2 -	ł
	1	0.2039	32+	11	1.3760	3/2 -	ł
	2	0.7658	72	12	1.4350	5/2 -	L.
	3	0.7862	1/2 +	13	1.5410	11/2 +	
	4	0.8206	3/2 ≁	14	1.5528	9/2 +	
	5	0.9478	9,2 +	15	1.6202	3/2 +	-
	6	1.0391	1/2 +	16	1.6700	5/2 +	
	7	1.0590	5,2 +	17	1.6830	9/2 +	
	8	1.0741	7/2 +	18	1.7070	1/2 +	
	9	1.2225	5/2 +	19	1.9380	11/2 -	-
		Continuum	levels assume	ed above	e 2 MeV.		

Mo-96	No	Energy MeV	Spin-Parity	No	Energy (MeV)	Spin-Parity
	g.s.	0	0 ·	8	2,0956	2 +
	1	0.7783	2 ·	9	2.2193	4 +
	2	1479	0 -	10	2.2345	3
	3	1.4978	2 +	11	2,4262	3 +
	4	1.6260	2 +	12	2.4384	5 +
	5	1.6280	4 +	13	2,4406	6 +

6	1.8695	4 +	14	2.4807	4 +
,	Continuum	levels assum	ed above	2.5 MeV.	
Mo 97 No g.s 1 3 4 5 6 7 8 9 10	Energy (MeV 0 0.4809 0.6579 0.6796 0.7105 0.7211 0.8882 1.0245 1.0245 1.0226 1.1167 1.1486 Contanuum 1	<pre>Spin Parit; 52; 72; 12; 52; 32; 12; 52; 32; 12; 32; 72; 92; 72; 12; 72; 82; 72; 12; 72; 83; 83; 84; 72; 84; 72; 84; 72; 84; 72; 72; 84; 72; 72; 72; 72; 72; 72; 72; 72; 72; 72</pre>	y No E 11 12 13 14 15 16 17 18 19 20 20	nergy (MeV 1.2886 1.2730 1.2840 1.2846 1.4095 1.4373 1.4470 1.5156 1.5452 1.5651 1.58 MeV.	Spin Parity 72: 32: 32: 32: 11:2: 11:2: 32: 92: 5:2: 3:2:
Mo 98 No g s 1 2 3 4 5 6 7 8 9	Energy MeV 0 0,7349 0,7874 1,4323 1,5101 1,7585 1,8809 1,9650 1,9855 2,0176 Continuum 1	Spin Parity 0 + 2 + 2 + 4 + 3 + 0 + 1 + 3 evels assume	 No Ei 10 11 12 13 14 15 16 17 18 d above 	eergy MeV 2,1049 2,2069 2,2240 2,3334 2,3437 2,4198 2,4500 2,4854 2,5063 2,53 MeV.	Spin Parity 2 + 2 + 2 + 6 + 3 - 4 + 3 + 3 + 3 -
Mo 100 No g.s. 1 2 3 4 5 6 7 8 6	Energy MeV 0 0,5355 0,6944 1,037 1,1351 1,4633 1,7657 1,7704 1,9081 Continuum 1	Spin Parity 0 - 2 - 4 - 2 - 4 - 2 - 1 - 3 - evels assume	No En 9 10 11 12 13 14 15 16 16	ergy MeV 2.0330 2.0400 2.1014 2.3400 2.4156 2.4700 2.5632 2.5900 2.62 MeV.	Spin Parity 0 + 2 + 3 + 3 + 3 + 4 + 3 + 4 +
The ine molybder MT -Qt 51 0.2 52 0.4 53 0.5 54 0.6 55 0.6 55 0.6 55 0.7 58 0.7 58 0.7 59 0.7 60 0.6	lastic leve hum file as MeV) Mo-94 2039 1808 - 3354 - 5578 5941 - 7194 7194 7347 7659 - 7863 - 1207 -	ls of each is follows: 2 Mo-94 Mo- - 5 - 5 - 5 - 5 - 5	sotope a 95 Mo 51 52 51 33 44	re grouped 96 Mo-97 51 52.53 54.55	in natural Mo-98 Mo-100

61 62 63	0.8712 0.9479 1.0244		51	55 56		56 57		
64	1.0591			57,58				53
65	1.0925					58.59		
66	1.1356				52	60		- 54
67	1.2226			59				
68	1.2685					61.62		
						63.64		
69	1.3101			60				
70	1.3761			61		65		
71	1.4320			62		66	53	
72	1.4468					67		55
73	1.4978	51			53	68	54	
74	1.5412		52	63.64		69.70		
75	1.6204			65	54.55			
76	1 6702			66.67				
				68				
77	1.7424		53				55	56,57
78	1.8646		54		56		56	
79	1.9073			69				58
80	1.9646				57		57.58	
81	2.0172		55				59	59,60
82	2.0956				58		60	61
83	2 2064				59.60		61.62	
84	2 2836	52	56		00100		017665	
85	2 3329	(Als	0.0				63.64	62
86	2 3935		57.59		61		65	63
87	2 4384		01100		62.63		66	64
ÅÅ.	2 4807				RA		67 68	0.1
RG -	2 5208	59 51	50		04		01100	65
00	2,0200 9 5676	55	60 61					88
00	1 5709	56 57	00.01	01	01	01	01	01 01
91	1.01003	01.01	01	51	51	υŢ	51	51
		<7 I						

The level density parameters evaluated by lijima+/12'.

Mo isotope	91	92	93	94	95	96
a 1 MeV	10.87	10.20	11.25	11.80	13.60	14.03
Delta MeV	1.28	2.21	1.28	2.00	1.28	2.40
Ex (MeV)	5.428	6.665	3.14	6.228	5.835	7.645
Tc (MeV)	0.627	0.85	0.605	0.760	0.715	0.741
			~~			
Mo-isotope	97	98	99	100	101	
Mo-isotope a (1/MeV)	97 15.17	98 15.97	99 17.74	100 19.35	101 20.85	
Mo-isotope a (1/MeV) Delta(MeV)	97 15.17 1.28	98 15.97 2.57	99 17.74 1.28	100 19.35 2.22	101 20.85 1.28	
Mo-isotope a (1/MeV) Delta(MeV) Ex (MeV)	97 15.17 1.28 4.988	98 15.97 2.57 7.53	99 17.74 1.28 5.775	100 19.35 2.22 6.795	101 20.85 1.28 5.766	
Mo-isotope a (1/MeV) Delta(MeV) Ex (MeV) Tc (MeV)	97 15.17 1.28 4.988 0.618	98 15.97 2.57 7.53 0.671	99 17.74 1.28 5.775 0.605	100 19.35 2.22 6.795 0.600	101 20.85 1.28 5.766 0.549	

MT=102 : Capture

Calculated with the statistical model code CASTHY /9/. Competing processes : (n.2n), (n.3n), (n.p), (n.a) Level fluctuation considered. The gamma-ray strength function of each isotope was obtained so as to reproduce the ORELA capture data /3/: Mo-92 : 9.4E-5 , Mo-94 : 2.0E-4 , Mo-95 : 2.9E-3 , Mo-96 : 1.7E-4 , Mo-97 : 2.9E-3 , Mo-98 : 1.4E-4 ,

Mo-100: 1.4E-4

- MF-4 Angular Distributions of Secondary Neutrons MT-2 : Calculated with optical model. MT 51.52.53.55.57.60.62.67.69 : 90 degree symmetric in the center of mass system. MT 54 90 other than above : Isotropic in the center of mass system. MT 16.17.91 : Isotropic in the laboratory system.
- MF 5 Energy Distributions of Secondary Neutrons MT 16.17.91 Evaporation spectrum.

- 1 Wasson O A. et al.: Phys.Rev. C7.1532 1973 .
- 2 Weigmann H. et al.: 1971 Knoxville Conf. CONF 710801.p.749.
- 3 Musgrove A.R.de.L. et al.: Nucl. Phys. A270, 108 1976 .
- 4 Shwe H. and Cote R E.: Phys. Rev. 179,1148 (1969).
- 5 Chrien R.E. et al.: Phys. Rev. C13,578 1976
- 6 Weigmann H. et al.: Phys.Rev. C20,115 (909)
- 7 Iijima S. and Kawai M.: J.Nucl.Sci.Technol. .20,77 (1983).
- 8 Young P.G. and Arthur E.D.: LA 6947 (1977).
- 9 Igarasi S.: J.Nucl.Sci.Technol., 12.67 (1975).
- 10 Lederer C.M. and Shirley V.S.: Table of Isotopes.7th Edition. Wiley Interscience (1978).
- 11 Matumoto Z. et al.: JAERI M 7734 (1978 -
- 12 IIJIMA S. et al.: to be published in J.Nucl.Sci.Technol.

1 of Molybdenum 92

42 Mo 92 MAT number 2421 42 Mo 92 JNDC Eval Aug82 Y.Kikuchi et al. Dist Mar83 Rev1 Nov83 History 82.08 New evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC. 83-11 Comment was added. MF 1.MT 451 Comments and dictionary MF 2.MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV 50 keV Evaluation based on the following data. Thensmission Masson - Wasser : Weigmann 2 Musgrove 3 . Capture Assumed Gam g = 1 200 milli eV for s wave and 425 milli eV for p wave Unresolved resonances : 50 keV 100 keV Energy independent parameters are given: SO 0.37E 4 .S1 5.48E 4 .S2 3.65E 4, Dobs 2400 eV , Gam g 0.226 eV, R 6.72 fm. Calculated 2200 m s values and resonance integrals (barn) : 2200 m s value Res.Int. 5.566 total 5.545 elastic 0.020750.981 capture. MF 3 Neutron Cross Sections Slight background correction for Sig t and Sig c between 50 and 100 keV. Cross sections above '00 keV evaluated as follows. MT 1 : Total Calculated with optical model. Potential parameters obtained by systematic fitting to Sig-t by Iijima+ 4/ : V 46.0 0.25 En Ws 7.0 .Vso=7.0 (MeV) R0=Rso=5.89 ,Rs=6.39 (fm) a0=aso 0.62 .b ≃0.35 (fm) MT-2 : Elastic scattering (Total All other partial cross sections). MT=16, 103, 107 : (n, 2n), (n, p), (n, a)Calculated with GNASH code 5 considering pre-equilibrium process. MT-51-57.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /6/. Competing processes : (n,2n), (n,p), (n,a) Level fluctuation considered.

The level sche No g.s.	me taken from Ref. Energy:MeV O	7. D Spin 0	Parity
1	1.5095	2	•
2	2.2326	4	1
3	2.0197	0	+
4]	2.9270	5	
5	2,0130	5	1
0 7	2. 7000 2. BAOZ	0	+
Continu	m lavale secunad	ahowa 9 Ma	v
The level dong	un revers assumed ity paramotore ava	lusted by	Y. Trinner G
Mo isoto	rcypentemeterseve ne Ω1	aced by	11 j1 ma. 0 . Q2
	4eV 10.87	10.2	11.25
Delta Me	1 1 28	2 21	1 28
Ex Me	7 5 428	6 165	3 14
Te MeA	0.627	9.85	0.605
The gamma ray s	trength function	of 0.94E 4	obtained so as to
reproduce the (NELA captor - data	3	
MT 251 : Mu bar Calculated with	optical model.		
MF 4 Angular Distri MT 2 : Calcul MT 16 : Isotro MT 51 57 : 90 deg MT 91 : 90 deg	butions of Seconda ated with optical pic in the laboral ree symmetric in (ree symmetric in (ary Neutron model. tory system the center the laborat	ns 1. of mass system. cory system.
MF 5 Energy Distrib MT 16.91 : Evapor	utions of Secondar ation spectrum.	y Neutrons	3
References 1 Wasson O.A et a 2 Weigmann H. et a 3 Musgrove A R.de. 4 Hijima S. and Ka 5 Young P.G. and A 6 Igarasi S.: J.Nu 7 Lederer C.M. and Wiley Interscience 8 Hijima S. et al.	 Phys.RevC7.1 1971 Knoxville et al.: Nucl.Pr wai M.: J.Nucl.Sci rthur E.D.: LA 694 cl.Sci.Technol12 Shirley V.S.: Tab ce (1978). to be published 	532 1973 • ConfCON • vsA270.1 Technol 7 1977 • 67 1975 • 197	F 710301.p.749. 08 1976 20.77 1983 opes.7th Edition, Sci.Technol.

42 Mo 94 MAT number 2422

42 Mo 94 JNDC Eval Aug82 Y.Kikuch) et al. Dist Mar83 Rev1 Nov83 History

82 08 Nev evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC.

83 11 Comment was added.

MF 1.MT 451 Comments and dictionary

MF 2.MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV 20 keV Evaluation based on the following data. Canture : Weipmann 1 Musprove 2. Assumed Came : 135 milli eV for 5 wave and 175 milli eV for 5 wave. Unresolved resonances : 20 keV 100 keV Energy independent parameters are given: SO 0.37E 4 .S1 5.48E 4 .S2 3.65E 4.

Dobs 1150 eV .Gam g. 0.230 eV.R - 6.68 fm.

Calculated 2200 m s values and resonance integrals (barn): 2200 m s value Res.Int. total 6.011 elastic 5.998

capture 0.01311 1.43 MF 3 Neutron Cross Sections

Slight background correction for Sin t and Sin c between 30 keV and 100 $keV_{\rm c}$

Cross sections above 100 keV evaluated as follows.

MT-1 : Total Calculated with optical model. Potential parameters obtained by systematic fitting to Sig-t by Iijima+3 : V = 46.0 - 0.25+En .Ws=7.0 .Vso=7.0 (MeV) R0=Rso=5.89 .Rs=6.39 (fm) a0-aso=0.62 .b=0.35 (fm)

- MT+2 : Elastic scattering (Total) - (All other partial cross sections).
- MT=16.17.103.107 : (n.2n), (n.3n), (n.p), (n.a) Calculated with GNASH code [4] considering pre-equilibrium process.
- MT=51-61.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /5/. Competing processes : (n,2n), (n,3n), (n,p), (n,a) Level fluctuation considered. The level scheme taken from Ref./6/.

No	Energy MeV	Spin-Par.	ity
g.s.	0	0 1	
- 1	0.8710	2 -	
2	1.5737	4 1	
3	1.7420	0 +	
4	1,8642	5 +	
5	2.0674	2 .	
6	2,2940	4 ·	
7	2.3930	2 -	
8	2.4230	6 .	
9	2.5337	3	
10	2.5670	4 +	
11	2.6100	5	
Continuum le	vels assumed abo	ve 2.74 MeV	<i>'</i> .
The level density p	arameters evalua	ted by liji	ma 7.
Mo isotope	93	94	95
a ! Me-V	11.25	11.80	13.60
Delta MeV	2.22	2.0	1.28
Ex M-V	5.14	6.228	5.835
Te MeV	0.605	0.76	0.715
The gamma ray stren	gth function of	2.0E 4 obta	ined so as

The s to reproduce the ORELA capture data 2 .

MT 251 : Mu bar

Calculated with optical model.

MF 4 Angular Distributions of Secondary Neutrons

MI. 5	: Calculated with optical model.
MT 16.17	: Isotropic in the laboratory system.
MT 51 61	: 90 degree symmetric in the center of mass system.
MT 91	: 90 degree symmetric in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons MT 16.17.91 ... Evaporation spectrum.

- i Weigmann H. et al.: 1971 Knozville Conf. CONF 710301.p.749.
- 2 Musgrove A.R.de.L. et.al.: Nucl.Phys., A270, 108 (1976).
- 3 Iijima S. and Kawai M.: J.Nucl.Sci.Technol., 20,77 (1983).
- 4 Young P.G. and Arthur E.D.: LA-6947 (1977).
- 5) Igarasi S.: J.Nucl.Sci.Technol., 12,67 (1975).
 6 Lederer C.M. and Shirley V.S.: Table of Isotopes,7th Edition. Wiley-Interscience (1978).
- 7 Iijima S. et al.: to be published in J.Nucl.Sci.Technol.

42 Mo 95 MAT number 2423

42 Mo 95 JNDC Fval Aug82 Y.Kikuchi et al. Dist Mar83 Rev1 Nov83 History 82 08 New evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC. 83 1 Comment was added. MF 1 MT 451 Comments and dictionary MF 7 MT 151 Resonance parameters Re dived resonances for MLBW formula 1.0E 5 eV 2 keV Evaluation based on the following data. Transmission & Shve-Capture Wetemann - 2 Assumed Gam g = 150 milli eV for s wave and 180 milli eV for p wave. A negative resonance added at 20 eV. Un esolved resonances : 2 keV = 100 keV Energy independent parameters are given: SO 0.37E 4 .SI 5.48E 4 .S2 3.65E 4. Dobs 80 eV .Gam g 0.232 eV.R 6,70 fm. Conculated 2200 m s values and resonance integrals (barn) : 2200 m s value Res.Int. 19.58 total 5.586 elastic capture 13.99119 MF 3 Neutron Cross Sections Slight background correction for Sig t and Sig c between 40 and 100 keV Cross sections above 100 keV were evaluated as follows. MT 1 Total Calculated with optical model. Potential parameters obtained by systematic fitting of Sig-t by Iijima+ 3 💠 .Vso 7.0 (MeV) V 46.0 0.25 En ,Ws 7.0 R0-Rso-5,89 .Rs 6.39 (fm) a0-aso-0.62 . 0.35 (fm) MT=2 : Elastic scattering (Total - All other partial cross sections). MT-16.17.103.107 : n.2n . n.3n . n.p. n.a. Calculated with GNASH code 4 considering pre-equilibrium process. MT-51-69.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /5/. Competing processes : (n,2n), (n,3n), (n,p), (n,a)

Level The level	fluctuation scheme take	considered n from eva	luation by	Matumoto	6.
	No	Energy Me	V Spin	Parity	
	g.s.	0	5.	2	
	- 1	0.2039	3.4	2 +	
	2	0.7658	7.3	2	
	3	0.7862	1 1	2,	
	4	0.8206	3-3	2,	
	5	0.9478	9 7	2 ,	
	6	1.0391	1.2	2 +	
	.7	1.0590	5 2	2 ₁	
	8	1.0741	7 8	2 +	
	9	1.2225	5 2	2.	
	10	1.3100	1 2	2	
	11	1.3760	3 2	\$	
	12	1.4350	5 2	2 -	
	13	1.5410	11 2	2 .	
	17	1 5529	9 2	2	
	1	1.0202	. 32	2.	
	16	6700	5 2	, .	
	17	1,6830	9 2	- 	
	18	1.7070	1 2	<u>+</u>	
	19	1.9380	11/2	2	
Cor	ntinuum level	is assumed	above 2 Me	Υ.	
The level	density para	meters eva	luated by	lijima+/7	·
Mo i	sotope	93	94	95	96
a	1 MeV	11.25	11.80	13,60	14.03
Delt	a MeV	1.28	2.0	1.28	2.40
Ex	MeV	3.14	6.228	5.835	7.645
Те	MeV	0.605	C 76	0.715	0.741
The gamma	ray strength	function	of 2.9E-3	obtained s	so as to
reproduce	the ORELA ca	pture data	8.		• •• ••
MT 251 : Mu	bar				
Calculated	with optica	1 model			
MF 4 Angular D	istributions	of Second	arv Neutro	ns	
MT-2	: Calculated	with opti	cal model.		
MT-16,17	: Isotropic	in the lab	oratory sv	smt.em.	
MT 51 69	: 90 degree	symmetric	in the cen	ter-of-mas	s system
MT-91	: 90 degree	symmetric	in the lab	oratorv sv	stem.
		0,7111100110	in one rub	5140019 59	b ben .
MF-5 Energy Di	stributions	of Seconda	rv Neutron	5	
MT=16,17.91	: Evaporation	n spectrum			
	•	-			
References					
1 Shwe H. and	Cote R.E.: 1	Phys.Rev.1	79,1148 (19	9 69)	
2) Weigmann H.	et al.: 197	1 Knoxville	e ConfCOM	F-710301.	p.749.
3 Iijima S. a	nd Kawai M.:	J.Nucl.Sc:	i.Technol.	20,77 (19	B 3).
4 Young P.G. a	and Arthur E	.D.: LA 69	1977	-	
5 Igarasi S.:	J.Nucl.Sci.	Fechnol12	2.67 1975		
6 Matumoto Z.	et al.: JAER	I M 7734 19	278 .		
7) Iijima S. et	t al.: to be	published	in J.Nucl.	Sci.Techno	ol.
8 Musgrove A.	R.de.L. et al	L: Nucl.Pl	nys.,A270.1	08 (1976)	

42-Mo- 96 MAT number 2424 42 Mo 96 JNDC Eval-Aug82 Y.Kikuchi et al. Dist Mar83 Rev1 Nov83 History 82 08 New evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC. 83 11 Comment was added. MF 1.MF 451 Comments and dictionary MF 2.MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV 19 keV Evaluation based on the following data. Capture : Weismann 1 Musgrove 2 Assumed Gam g 1114 milli eV for s wave and 136 milli eV for p wave. Unresolved resonances : 19 keV 100 keV Energy independent parameters are given: 0.37E 4 .SI 5.48E-4 .S2 3.65E-4. S0 Dobs 950 eV .Gam g 162 MeV .R 6.68 fm. Calculated 2200 m s values and resonance integrals (barn) : 2200 m s value Res.Int. total 5.3224.721 elastic 0.595417.6 capture MF 3 Neutron Cross Sections Slight background correction for Sig t and Sig c between 30 and 100 keV. Cross sections above 100 keV were evaluated as follows. MT 1 · Total Calculated with optical model. Potential parameters obtained by systematic fitting of Sig-t by Iijima+ 3 : V =46.0 - 0.25+En ,Ws≈7.0 .Vso-7.0 (MeV) R0=Rso=5.89 ,Rs=6.39 (fm) a0=aso=0.62 .b ≈0.35 (fm) MT=2 : Elastic scattering (Total) - (All other partial cross sections). MT=16, 17, 103, 107 :: (n, 2n), (n, 3n), (n, p), (n, a)Calculated with GNASH code 4 considering pre-equilibrium process. MT-51-64.91.102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /5/. Competing processes : (n,2n), (n,3n), (n,p), (n,a)Level fluctuation considered. The level scheme taken from evaluation by Matumoto+/6/.

No	Energy (Me	V) Spin	Parity			
6.S. 1	0 7799	2	\$* •			
2	1 1/70	2				
2 3	1 1978	2				
Ă	1 6260	5				
5	1 6280	Δ.				
ě	8695	4				
7	1 9783	ä				
Ŕ	2.0956	ž				
9	2,2193	4				
10	2.2345	3				
11	2,4262	3	,			
12	2,4384	5	•			
13	2,4406	6				
14	2,4807	4				
Continuum Leve	els assumed	above 2.5	MeV.			
The level density par	ameters eva	aluated by	Tijima+	7.		
Me isotope	94 94	95	96	97		
a 1 MeV	11,80	13.60	14.03	15.17		
Delta MeV	2.0	1.28	2.40	1.28		
Ex (MeV)	6.228	5.835	7.645	4.988		
TC (MeV)	0.76	0.715	0.741	0.618		
The gamma ray strength function of 1.7E 4 obtained so as to reproduce the ORELA capture data 2.						
MT 251 : Mu bar Calculated with optic	al model.					
MF 4 Angular Distribution	s of Second	ary Neutro	ns			
MI 2 Calculate	d with opti	cal_model.				
MI 16.17 Isotropic	in the lab	oratory sy	stem.			
MI 51 64 90 degree	symmetric	in the cen	ter of m	ass system.		
ni 91 : 90 degree	symme Ditc	in the lab	oratory	system.		
MF 5 Energy Distributions MT 16.17.91 : Evaporatio	of Seconda on spectrum	ry Neutron	s			
References						
1; weigmann H. et al.: 1971 Knoxville Conf., CONE-710301, p. 749.						
2) Husgrove A.R. de.L. et al.: Nucl. Mys. A2/0, 106 (1976).						
5 11 Jina 5. anu Kawai P. J. Nuci. Sci. reconot 20, 77 (1983).						

- 4) Young P.G. and Arthur E.D.: LA-5947 (1977).
- 5) Igarasi S.: J.Nucl.Sci.Technol., 12,67 (1975). 6) Matumoto Z. et al.: JAERI-M 7734(1978).
- 7 Iijima S. et al.: to be published in J.Nucl.Sci.Technol.

1 of Molybdenum-97

42 Mo 97 MAT number 2425 42 Mo 97 JNDC Eval Aug82 Y, Kikuchi et al. Dist Mar83 Rev1 Nov83 History 82-08 New evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC. 83 11 Comment was added. MF-1.MT 451 Comments and dictionary MF 2.MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV 1.8 keV Evaluation based on the following data. Transmission : Shwe 1 Capture Weigmann 2 Assumed Gam g : 130 milli eV for s-wave and 150 milli eV for p wave. A negative resonance added at $\sim 20 \text{ eV}$. Unresolved resonances : 1.8 keV 100 keV Energy independent parameters are given: 0.37E 4 .SI 5.48E 4 .S2 3.65E-4, S0 Dobs 60 eV .Gam g 180 MeV .R - 6.67 fm. Calculated 2200 m is values and resonance integrals (barn) : 2200 m/s value Res.Int. Lota1 7.953 elastic 5,853 17.3 capture. 2.100 MF 3 Neutron Cross Sections Slight background correction for Sig t and Sig c between 20 and 100 keV. Cross sections above 100 keV were evaluated as follows. MT-1 : Total Calculated with optical model. Potential parameters obtaine by systematic fitting of Sig-t by lijima+/3/ : V =46.0 - 0.25 En ,Ws=7.0 ,Vso:7.0 (MeV) R0=Rso=5.89 ,Rs=6.39 (**fm**) a0=aso=0.62 ,b =0.35 (fm) MT=2 : Elastic scattering (Total) - (All other partial cross sections). MT=16, 17, 103, 107 : (n, 2n), (n, 3n), (n, p), (n, a)Calculated with GNASH code 4 considering pre-equilibrium process. MT=51-70,91,102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /5/. Competing processes : (n,2n), (n,3n), (n,p), (n,a)

Level fluc The level sch	tuation conside eme taken from	red. evaluation b	y Matumolo)	6.
No	Energy	MeV Spi	n Parity 👘	
g.s	. 0	- 5	2	
l	0.48	09 3	2 ·	
2	0.65	79 7	- 2 - 4	
3	0.67	96 1	2 H	
4	0.71	95 5	/ 2 +	
5	0.72	11 3	2 +	
6	0.88	92 1	·2 +	
1	1.02	45 7	2 +	
8	1.09	20 <u>3</u>	2+	
9	1.11	চ। সূ স	/Z + /0	
10	1.14	50 <i>1.</i> 26 7	2 ···	
11 11	1.20		2 1	
13	1.21	-0 -0 -0 -03	2	
13	1.20	10 IS 16 3	2 ·	
15		25 11	2	
16	1.43	73 11	2	
17	1.44	70 3	2.	
18	1.51	36 9.	2	
19	1.545	2 5	2	
20	1.569	51 3/	2 -	
Continu	um levels assum	ed above 1.5	8 MeV.	
The level dens	ity parameters	evaluated by	Iijima+/7/	· .
Mo-isoto	pe 95	96	97	98
a (1)	MeV) 13.60	14.03	15.17	15,94
Delta (Me	V) 1.28	2.40	1.28	2.57
Ex (Me	V) 5.835	7.645	4.988	7,53
Tc (Me	V) 0.715	0.741	0.618	0.671
The gamma ray	strength functi	on of 2.9E-3	obtained s	o as to
reproduce the	ORELA capture c	ata : 84.		
M1 201 MU Dar	المصيدة فيسام			
calculated wit.	n optical model	•		
MEnd Angulan Digth	ibutions of fea	andawe Nauta		
MT 2	loutions of Sec	ondary Neutr	ons	
MT-16 17 Is	otropic in the	Jeporatory e	Vetom	
MT=51-70 : 90	degree symmetr	ic in the ce	yscan. ntor_of…mas	e sustem
MT=91 : 90	degree symmetr	ic in the la	horatory sy	stem
111-51 : 55	degree Synaneer	ie in che la	bolacory sy	o cem.
MF=5 Energy Distril	outions of Seco	ndary Neutro	ns	
MT=16,17.91 : Eva	aporation spect	rum.		
References				
1) Shwe H. and Cote	≥ R.E.: Phys.Re	v.179,1148 (1969).	
2) Weigmann H. et a	al.: 1971 Knoxv	ille Conf.,C	NF-710301,1	o.749.
3) Iijima S. and Ka	wai M.: J.Nucl	.Sci.Technol	.,20,77 (196	33).
4) Young P.G. and I	Arthur E.D.: LA	- 6947 (1977).		
5) Igarasi S.: J.Nu	cl.Sci.Technol	. 12.67 (1973	5).	
6) Matumoto Z. et a	1.: JAERI-M 773	4(19/8).	~	
7) Iijima S. et al.	: to be publis!	ied in J.Nucl	.Sci.Techno	ы.
8) Musgrove A.R.de.	L. et al : i.ec.		108 (1976).	
			•	
		-		
42 Mo 98 MAT number 2426 42 Mo 98 JNDC Eval Aug82 Y.Kikuchi et al. Dist Mar83 Rev1 Nov83 History 82.08 New evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC . 83 11 Comment was added. MF 1.MF 451 Comments and dictionary MF 2.MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV - 32 keV Evaluation based on the following data. Transmission : Chriens 1 Capture : Weigmann 2 Musgrover 3 Assumed Gam g : 85 milli eV for s wave and 120 milli eV for p-wave. A negative resonance added at - 980eV. Unresolved resonances : 32 keV - 100 keV Energy independent parameters are given: 0.37E 4 .S1 5.48E 4 .S2 3.65E-4, SO Dobs 950 eV .Cam g 0.133 eV.R = 6.66 fm. Calculated 2200 m s values and resonance integrals (barn) : 2200 m/s value Res.Int. total 5.772 5.642 elastic 0.1300 6.56 capture MF 3 Neutron Cross Sections Slight background correction for Sig t and Sig-c between 32 and 100 keV. Cross sections above 100 keV were evaluated as follows. MI-1 Total Calculated with optical model. Potential parameters obtained by systematic fitting of Sig-t by Iijima 4/ : V ≈46.0 - 0.25+En .Ws≈7.0 ,Vso 7.0 (MeV) R0=Rso=5.89 ,Rs=6.39 (fm) .b =0.35 a0=aso=0.62 (fm) MT=2 : Elastic scattering (Total) - (All other partial cross sections). MT=16, 17, 103, 107 : (n, 2n), (n, 3n), (n, p), (n, a)Calculated with GNASH code /5/ considering pre-equilibrium process. MT=51-68,91,102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /6/. Competing processes : (n,2n), (n,3n), (n,p), (n,a)

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Level	fluctuati	on considered	•		
The leve	el scheme t	aken from eva	luation by	Matumoto	o+∞7°.
	NO	Energy (Me	v Spin	Parity	
	6.S. 1	0 7949	0	1	
	2	0.7874	, v		
	3	1.4323	2	1	
	4	1.5101	4	1	
	5	1.7585	2	ł	
	6	1.8809	3	+	
	7	1.9650	0	+	
	8	1.9855	1	4	
	.9	2.0176	3		
	10	2.1049	2	•	
	10	2.2009	2	•	
	12	2 3334	2	•	
	10	2 3437	6		
	15	2.4198			
	16	2.4500	4	•	
	17	2.4854	3	1	
	18	2.5063	3		
С	ontinuum le	evels assumed	above 2.53	MeV.	
The leve	l_density p	arameters eva	luated by	lijima⊬∕	8/
Mo	isotope	96	97	.98	99
8	(1./MeV)⊢	14.03	15.17	15.94	17.74
De	Ita (MeV)	2.40	1.28	2.5/	1.28
EX. To	(MeV)	7.645	4.908	0 671	5.775
The comm	(ner) a nor stror	U.141 with Cupation	0.010	obtained	
reproduce	a ray scren a tha ORM A	igen runceron Loanture data	01 1.46.4	obrained	so as to
reproduce		a captore data			
MT 251 : M	a bar				
Calculate	ed with opt	ical model.			
MF 4 Angular	Distributi	ons of Second	ary Neutro	ns	
MT 2	🔆 Calcula	ted with opti	cal model.		
MT-16,17	Isotrop	ic in the lab	oratory sy	stem.	
MI 51-68	: 90 degr	ee symmetric	in the cen	ter-oi-m	ass system.
MI-91	90 degr	ee symmetric	in the lab	oratory a	system.
MR-5 Frenzy F	Vistributio	ne of Seconda	ry Neutron	-	
MT. 16 17 91	· Fyanora	tion spectrum	iy neutron:	5	
11-10,11,01	· Liupoid	cron spectrum	•		
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b) igarasi S.	J.Nucl.Sc	ci.iechnoi.i	2.67 1975	· •	
i) Matumoto Z	. et ai. J <i>i</i>	anni n 1134 i	115 · .		

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1 of Molybdenum-100

42-Mo-100 MAT number - 2427

42 Mo 100 JNDC Eval Aug82 Y.Kikuchi et al. Dist Mar83 Rev1 Nov83

History

- 82.08 New evaluation for JENDL 2 by W.G. on FP nuclear data of JNDC.
- 83 11 Comment was added.
- MF-1.MT 451 Comments and arctionary
- MF 2.MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV 26 keV Evaluation based on the following data. Transmission : Weigmann 1 Capture : Weigmann 2 Musgrove 3 Assumed Gam g : 65 milli eV for s wave and 80 milli eV for p-wave. A negative resonance added at = 172 eV. Unresolved resonances : 26 keV = 100 keV Energy independent parameters are given;
 - SO 0.37E 4 .SI 5.48E 4 .S2 3.65E 4. Dobs 620 eV .Gam g 0.85 eV .R = 6.64 fm.
- Calculated 2200 m s values and resonance integrals (barn) : 2200 m·s value Res.Int. total 5.499

elastic	5.300	
capture	0.1990	3.92

- MF 3 Neutron Cross Sections
- Slight background correction for Sig t and Sig c between 26 and 100 keV.
 - Cross sections above 100 keV were evaluated as follows.
 - MT-1 : Total Calculated with optical model. Potential parameters obtained by systematic fitting of Sig-t by Iijima+4 : V -46.0 - 0.25+En .Ws-7.0 .Vso-7.0 (MeV) R0=Rso=5.89 .Rs=6.39 (fm) a0-aso=0.62 .b = 0.35 (fm)
 - MT-2 : Elastic scattering (Total - (All other partial cross sections))
- MT=16.17.103.107 : (n.2n).(n.3n).(n.p).(n.a) Calculated with GNASH code 5/ considering pre-equilibrium process.
- MT=51-66.91,102 : Inelastic scattering and capture Calculated with the statistical model code CASTHY /6/. Competing processes : (n,2n), (n,3n), (n,p), (n,a)

Level fluctuat	ion considered.		
The level scheme	taken from evalua	tion by Matumote	or 7.
No	Energy (MeV)	Spin Parity	
g.s.	0	0 +	
1	0.5356	2 +	
2	0.6944	0 +	
3	1.0637	2 +	
4	1.1361	4 +	
5	1.4633	2 .	
6	1.7657	1 +	
7	1.7704	3 (
8	1.9081	3 -	
9	2.0330	0 /	
10	2.0400	2 +	
11	2.1014	4 :	
12	2.3400	2 ·	
13	2.4156	3	
14	2,4700	4	
15	2.1682	· 3 ·	
16	2,5900	4 +	
Continuum	levels assumed ab	ove 2.62 MeV.	
The level density	parameters evalu	ated by Iijima+/	8∕.
Mo isotope	98 9	9 100	101
a (1 MeV) 15,97 17	.74 19.35	20.85
Delta (MeV)	2.57	.23 2,22	1.28
Ex (MeV)	7.53 5	.775 6,795	5.766
To (MeV)	0.671 0	.605 0,600	0.549
The gamma ray stro	ength function of	1.4E 4 obtained	so as to
reproduce the ORE	🗚 capture data 🔅	3,	
MT 251 : Mu bar Calculated with o	ptical model.		
MF 4 Angular Distribut MT 2 : Calcul MT 16.17 : Isotro MT 51 66 : 90 deg MT-91 : 90 deg	lions of Secondary lated with optical opic in the labora pree symmetric in gree symmetric in	y Sentrons model. tory system. the center of m the laboratory =	ass system. system.
MF-5 Energy Distributi MT=16,17.91 : Evapor	ons of Secondary ation spectrum.	Neutrons	

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72 Hf 174 MAT number 2721 72 Hf 174 NAIG: Eval Dec82 Hida, Yoshida, Lijima, Takano (JAERI) Dist Mar83 Rev1 Dec83 History 82.12 New evaluation for JENDL 2 was made by K.Hida, T.Yoshida. S. Iijima (NAIG) and H. Takano (JAERI). 83 12 Angular distributions were modified and comment was added. MF-1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters Resolved resonances for MLBW formula 0.5 eV to 220 eV. Energy + ange Res. energies and Gam n : BNL S25 4 and Drake et al. /2/. : 0.060 eV assumed if unknown. Gam namma Background cross sections introduced. Unresolved resonances : 220 eV to 50 keV. Energy range : BNL 325 /1 . S0 : Adjusted so that the total cross SLR and Gam gamma section of 9.49 b and capture cross section of 0.752 b at 50 keV were reproduced well. 16.0 eV, S0 2.80E 4, S1 - 1.06E-4. Parameters are D obs 6.01 fm and Gam gamma 0.0844 eV. R 2200 m see cross sections and calculated res. integrals. 2200 m see res. integ. 393.0 h 3 total 0.0 h eles de 390.0 b -4 492 b canture MF 3 Neutron Cross Sections Below 0.5 eV: MT-1 Total Sum of the elastic scattering and capture cross sections. MT-2 Elastic scattering The constant cross section of 8.0 barns was assumed. MT=102 Capture The curve in the form of 1/v was adjusted to 390 barns /4/ at 0.0253 eV. From 0.5 eV to 50 keV: Background cross sections are given. Above 50 keV: MT=1.2.4.51-68.91.102 Total.elastic.inelastic and capture Calculated with optical and statistical models. Optical potential parameters were fitted to the total cross section of natural hafnium. V0 = 38.0, Ws = 8.0+0.5+SQRT(En), Vso = 7.0 (MeV), a0 = 0.47, as = 0.52, aso = 0.47 (fm),

r0 = 1.32, rs = 1.32, rso 1.32 fm -Statistical model calculation was made with CASTHY code 5. Competing processes ; (n.2n), (n.3n), n.p., (n.alpha), and level fluctuation were considered. Level schem taken from Table of Isotopes 6 Energy (MeV) No. Spin Parity g.s. 0.0 0 F 2 1 1 0.0910 2 0.2975 4 . 3 0.6084 6 . 4 0.8282 0 -5 2 . S006.0 4 . 6 1.0622 7 1.2268 2 . 8 1.3034 3 -9 1.3087 2 10 2 1.31942235 11 13 $1 - (20) \leq 1$ 12 4 13 1.4253 \mathbf{d} 5 14 1.442915 1.4489 4 1 16 1.4964 2 + 3 + 17 1.5034 18 1.6261 4 . Continuum levels assumed above 1.649 MeV. Level density paramters were newly evaluated for Gilbert and Cameron's formula 7 a 1 MeV C+1 MeV T MeV HeV) Hf 174 23.09 2.31 0.477 5.01 Hf 175 22.9310.0 0.484 4.42 MT 16.17.103.107 n.2n. n.3n. n.p and n.alpha Calculated with multi step Hauser Feshbach model using GNASH code 8 . MT 251 Mulber Calculated with CASTHY code 5 MF 4 Angular Distributions of Secondary Neutrons MT 2.51 68 Calculated with CASTHY code 5. MT-16.17 : Isotropic in the laboratry system. MT-91 : Calculated with CASTHY code /5/. MF-5 Energy Distributions of Secondary Neutrons MT=16.17.91 : Evaporation spectra. References 1) Mughabghab S.F. and Garber D.I.: BNL 325 3rd Edition (1973). 2) Drake M.K., Sargis D.A. and Maung T.: EPRI NP-250 (1976). 3 Conrad C.A. et al.: Bull. Am. Phys. Soc., 14, 496 (1969). 4: Esch L.J. and Moore W.E.: Bull. Am. Phys. Soc., 6, 70 (1961). 5 | Igarası S.; J. Nucl. Sci. Technol., 12, 67 (1975). 6 Lederer C.M. and Shirley V.S.: Table of Isotopes 7th Edition 1979 7) Gilbert A. and Cameron A.G.W.: Can. J. Phys., 43, 1446 (1965). 8) Young P.G. and Arthur E.D.: LA-6947 (1977).

1 of Hafnium 176

72 Hr 176 MAT number 2722 72 Hr 176 NAIG+ Eval Dec82 Hida.Yoshida.Jijima.Takano JAENI Dist Mart8 Rev1 Dec83 History 82 12 New evaluation for JENDL 2 was made by K.Hida, T.Yoshida, S.I.J.ma NAIG and H.Takano JAERI 83-12 Angular distributions were modified and comment was added. MF-1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters Resolved resonances for MLRW termula Ener dy Honge-2.5. eV to 200 eV. Bes energies and Cam n PN. 325 1 and Drake et al. 2. Gam Pantia. 0.000 eV assumed, if unknown. Background cross sections introduced Unresolved resonances Energy Lange 700 eV to 50 keV. SO Calculated with optical model. SLR and Gam gamma Fitted to the capture cross section of Kapchigashev 3 at 30 keV. Parameters are D obs. 32.0 eV. SO 1.02E 4. SI 1.11E 4, R 7.15 fm and Gam gamma 0,120 eV. 2200 m sec cross sections and calculated res. integrals. 2200 m sec 148 1114C 40 00 b 4 total 5 CC 5 elastic 20 10 4 5 280 S 1000 m 10 MF 3 Neutron Cross Sections Below 0.5 eV: Total MT 1 Sum of the elastic scattering and capture cross sections. S TM Elastic scattering The constant cross section of 8.0 barns was assumed. MT-102 capture The curve in the form of 1 v was adjusted to 38 barns /4/ at 0.0253 eV. From 0.5 eV to 50 keV: Background cross sections are given. Above 50 keV: MT=1.2.4.51 73.91.102 Total.elastic.inelastic and capture Calculated with optical and statistical models. Optical potential parameters were fitted to the total cross section of natural hafnium. V0 = 33.0 Ws 8.0-0.5 SQPT En . Vso = 7.0 (MeV), aC = 0.47. as = 0.52 aso = 0.47 / fm). r0 = 1.32, rs = 1.32, rso = 1.32 (fm).

Statistical model calculation was made with CASTHY code /6/.

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2 of Hafnium-176

Competin	g processes ;	(n.2	2n (n. 3n	, (n.p., (n.alpha	ο,
and leve	fluctuation	were	considere	d	
Level schem	taken from Ta	able	of Isotope	s 7	
NO. LI	iergy (Mev - S	spin	Parity		
E.S. (7.0	0	3		
2 1	1,0000	ي ۸	1		
3 () 6970	å			
4 (19980	Ř			
5	1499	õ			
6 1	.2266	ž	•		
7 1	2477	2			
8 1	. 2932	0	•		
9 1	3133	3			
10 1	.3413	2	•		
11 1	3794	2			
12 1	. 4046	4			
13 1	-44085	· •			
16 1	. D (+ + - 6 A G A				
10 1	6729	1			
17 1	7046	5			
18 1	7102	- 3			
19 1	7221	ĭ			
20 1	.7675	2			
21 1	.7861	3	3		
22 1	. 7937	3			
23 1	.8190	0			
Continuum	levels assumed	d abo	ove 1.840 h	leV.	
Level densit;	y paramters w	ere r	newly evalu	ated for Gilbert	
and Cameron's	s formula 8		-	17 IV	
6 071 041	I MeV CI	MeV Z	I MeV	tix MeV	
11 170 110 177	22.10 I. 22.61 0	00	0.404	4,30	
MT 16 17 103 10	$\frac{c_{\rm L}}{1}$ or $\frac{1}{2}$ or $\frac{1}{2}$ or $\frac{1}{2}$ or $\frac{1}{2}$ or $\frac{1}{2}$	- 00 - 35	0.000	4.00 n oloho	
Calculated w	r uardes ua ith nulli star	n Pan	isor Eashba	ch model using	
GNASH code 9	3		ioci - comen	on moner opting	
MT 251 Mu bar	•				
Calculated wi	th CASTHY cod	le 6	5.4 J		
MF-4 Angular Dist	ributions of	Seco	ndary Neut	rons	
MT 2.51 68 C	alculated wit	h CA	STHY code	/6/ .	
MT-16.17 : 1	sotropic in t	he 1	aboratry s	ystem.	
MT-91 : C	alculated wit	h CA	STHY code	6.	
MT 16 17 01	ibutions of S	econ	oary Neutr	ons	
m-10.11.91 · E	vaporation sp	ecun	a.		
References					
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3 Kapchigashev A	.P.: Atomizda	t. Me	oscov 1970)).	
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() Legerer U.M. a	nu Shirley V.:	5. i I	able of 15	colopes i th Ed.	

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1 of Hafnium-177

72-Hf-177 MAT number = 2723 72 Hr 177 NAIG Eval Dec82 Hida, Yeshida, Iijima, Takano (JAERI) Dist Mar83 Rev1 Dec83 History 82-12 New evaluation for JENDL 2 was made by K.Hida, T.Yoshida, S. Litima (NAIG) and H. Takano (JAERI). 83 12 Angular distributions were modified and comment was added. MF-1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters Resolved resonances for MLPW formula Emergy range C.5 eV to 250 eV. Res. energies and Cam n : BNL 325 1 and Drake et al. 2. : 0.066 eV assumed, if unknown. Gam gamma Unresolved resonances : 250 eV to 50 keV. Energy range S0 calculated with optical model. : fitted to the 15 percent greater SLR and Gam gamma value than capture cross section of Kapchigashev /3/ at 30 keV. Parameters are D obs 2.40 eV, SO 1.92E-4, SI = 1.12E-4, R 6.86 fm and Gam gamma 0.125 eV. 2200 m s cross sections and calculated res. integrals. 2200 m s Res. Integ. 359.0 b 4 total 7.0 5 elastic capture 352.0 b 5 6950 b MF 3 Neutron Cross Sections Below 0.5 eV: MT-- 1 Total Sum of the elastic scattering and capture cross sections. MT=2 Elastic scattering The constant value of 7.0 barns was assumed at low energies. MT=102 Capture The curve in the form of 1/v was adjusted to 352 barns /5/ at 0.0253 eV. From 0.5 eV to 50 keV: Background cross sections are zero. Above 50 keV. MT-1.2.4.51-66.91.102 Total.elastic.inelastic and capture Calculated with optical and statistical models. Optical potential parameters were fitted to the total cross section of natural hafnium. V0 = 38.0, $W_S = 8.0(0.5(SQRT En))$, $V_{SO} = 7.0$ (MeV), Statistical model calculation was made with CASTHY code /6/.

Competing processes ; (n,2n), (n,3n), (n,p), (n,alpha), and level fluctuation were considered. Level scheme taken from Table of Isotopes '7 No. Energy (MeV / Spin Parity C.S. 0.0 72 9.5 0.1130 1 2 0.249711 2 З 0.3213 1.50 4 0.409513.2 5 0.4267 11 2 -6 5.2 0.5081 7 0.5552 1321 8 15/2 0.5913 9 72 0.6044 10 0.7085 15 2 . 0.7459 72. 11 !2 0.7945 17 2 13 0.8057 32 14 0.8474 92. 52 15 0.8730 16 0.8828 172 . Continuum levels assumed above 0.948 MeV. Level density paramters were newly evaluated for Gilbert and Cameron's formula 8-. a I MeV C(1 MeV) T (MeV) Ex (MeV) Hf 177 22.619.03 0.4864.38 22.36 Hf 178 5 55 0.451 4.08 MT 16.17.103.107 (n, 2n), (n, 3n), (n, p) and (n, alpha)Calculated with multi-step Hauser Feshbach model using GNASH code 9 MT-251 Mu bar Calculated with CASTHY code 6 . MF 4 Angular Distributions of Secondary Neutrons MT 2.51 66 : Calculated with CASTHY code 6. MT 16.17 : Isotropic in the laboratory system. MT-91 : Assumed the same distributions in the laboratory system as those calculated with CASTHY code in the center-of mass system.

MF=5 Energy Distributions of Secondary Neutrons MT=16,17,91 : Evaporation spectra.

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72 Hf-178 MAT number 2724

72 Hf 178 NAIG+ Eval Dec82 Hida, Yoshida, Iijima, Takano (JAERI) Dist Mar83 Rev1 Jan84 History 82 12 New evaluation for JENDL 2 was made by K.Hida, T.Yoshida, S. Lijima (NAIG) and H. Takano (JAERI). 83-12 Angular distributions were modified and comment was added. 84 01 Point wise cross sections below 0.5 eV and background cross sections were modified. MF 1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters Resolved resonances for MLBW formula : 0.5 eV to 1.5 keV. Energy range Res. energies and Gam n : BNL 325 /1/ and Drake et al. /2/. Gam mamma : 0.060 eV assumed. Background cross sections introduced, Unresolved resonances : 1.5 keV to 50 keV. Energy region : calculated with optical model. 50 fitted to the capture cross sec-S1.R and Gam gamma tion of Beer and Macklin /3/. Parameters are D obs 52.5 eV. S0 1.92E 4. S1 0.435E 4, R 7.17 fm and Gam gamma - 0.060 eV. 2200 m s cross sections and calculated res. integrals. 2200 m s Res. Integ. 91 00 5 4 total 5.00 5 elastic 86.00 b 5 1920 b capture MF-3 Neutron Cross Sections Below 0.5 eV: MT= 1 Total Sum of the elastic scattering and capture cross sections. MT - 2 Elastic scattering The cross section of 5.0 barns was assumed below 0.1 eV. Above C.1 eV, the cross section was connected smoothly to the cross sections calculated from resonance parameters. MT=102 Capture The curve in the form of 1 v was normalized to 86 barns /5/ at 0.0253 eV. From 0.5 eV to 50 keV: Background cross section was given for the elastic scattering cross section below 3.0 eV. Above 50 keV: MT=1.2.4.51-71.91.102 Total.elastic.inelastic and capture Calculated with optical and statistical models. Optical potential parameters were fitted to the total cross section

of natural hafnium VO. 38.0, Ws 8.0:0.5:SQRT En , Vso 7.0 (MeV . 0.47, as 0.47 fm , a0 -. aso 0.521.32, rs 1.32r0, rso 1.32 (fm). Statistical model calculation was made with CASTHY code 6. Competing processes (n.2n., n.3n), n.p., (n.alpha), and level fluctuation were considered. Level scheme taken from Table of Isotopes 7 Energy MeV Spin Parity No. 0.0 0 . E.S. 0.0932 $S \rightarrow$ 1 2 0.3066 4 + 3 0.6322 6 . 4 1.0585 BI 51 1474 8 2 -6 1.1746 5 1.1993 1 2002 r. \$. (\cdot, \cdot) 1.1543 10 1 3099 1 11 1.3224 3 12 2 1.3624 13 1.3641 9 0 + 14 1.4340 15 1.4438 0 + 16 1.4790 B 17 2. 1.4961 18 1.5136 1.5613 19 2 . 20 1.5665 1 21 1.6015 10 Continuum levels assumed above 1.640 MeV. Level density paramters were newly evaluated for Gilbert and Cameron's formula 8 a 1 MeV – C i MeV T. Me-V. Ex MeV -21.00 4.08 Hf 178 2.20 -0.451Hr 179 22.57 6.88 0.465 3.98 MT 16,17,103,107 (n.2n., (n.3n., (n.p. and (n.alpha)) Calculated with multi-step Hauser Feshbach model using GNASH code 9 . MT 251 Mu bar Calculated with CASIHY code 6. MF-4 Angular Distributions of Secondary Neutrons MT=2.51-71 Calculated with CASTHY code /6/. MT=16,17 : Isotropic in the laboratory system. MT=91 : Assumed the same distributions in the laboratory system as those calculated with CASTHY code in the center-of-mass system. MF-5 Energy Distributions of Secondary Neutrons MT-16.17.91 : Evaporation spectra.

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- 8) Gilbert A. and Cameron A.G.W.: Can. J. Phys., 43, 1446 (1965).
- 9 Young P.G. and Arthur E.D.: LA 6947 (1977).

72 Hf 179 MAT number 2725 72 Hf 179 NAIG+ Eval Dec82 Hida, Yoshida, Limma, Takano (JAERI) Dist Mar83 Rev1 Dee83 History 82 12 New evaluation for JENDL 2 was made by K.Hida, T.Yoshida, S. Lijima NAIG and H. Takano JAERI . 83 12 Angular distributions were modified and comment was added. MF 1 General Information MT 451 Descriptive data and dictionary MF 2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters Paso Ver meren and the second 0.5 ef to 250 eV. Fnerry comp Best encluses and Gamin - BNL S25 1 and Drake et al. 20. . 0.000 eV assumed. Cam gamma Background cross sections introduced. Unresolved resonances Energy range 250 eV to 50 keV. 80 - calculated with optical model. SI,R and Gam gamma - fitted to the capture cross sec tion of Beer and Macklin 37. Parameters are D obs 4,73 eV. SO 1.92E 4, SI - 1.14E-4, R 7 12 fm and Gam gamma 0 063 eV. 2200 m s cross sections and calculated res. integrals. Res Integ. 2200 m s -10 D B 4 total. 6 2 5 elastic. e and mark 4 - 6 - 5 - 1 - 517 b MF 3 Neutron Cross Sections Below 0.5 eV: MT 1 Total Sum of the elastic scattering and capture cross sections. S TM Elastic scattering The constant cross section of 6.0 barns was assumed. MT 102 Capture The curve in the form of 1/v was normalized to 45 barns /1/at 0.0253 eV. From 0.5 eV to 50 keV: Background cross section was given. Above 50 keV: MT=1.2.4.51-62.91.102 Total.elastic.inelastic and capture Calculated with optical and statistical models. Optical potential parameters were fitted to the total cross section of natural hafnium. VO = 38.0, Ws = 8.040.5 (SQRT (En), Vso = 7.0 (MeV), a0 = 0.47, as = 0.52r0 = 1.32, rs = 1.32. aso = 0.47 (fm)., rso = 1.32 (fm). Statistical model calculation was made with CASTHY code /5/.

Competing processes ; (n,2n), (n,3n), (n,p), (n,alpha), and level fluctuation were considered. Level scheme taken from Table of Isotopes 6 No. Energy (MeV) Spin Parity 0.0 g.s. 9/2 + 0.1227 11.2 + 1 7/2 2 0.21433 0.2688 13.2 + 0.3377 9.2 -4 5 0.3750 12 6 0.4386 15/2 + 7 0.5184 5.2 8 7.2 . 0.6169 9 0.6312 17.2 + 19.2 10 0.8483 72 11 0.8702 1.0034 52. 12 Continuum levels assumed above 1.070 MeV. Level density paramters were newly evaluated for Gilbert and Cameron's formula 7. a 1 MeV C(1.MeV) T (MeV) ex (MeV) Hf 179 0.465 3,98 22.57 6.88Hf 180 2.35 21.37 0.519 5.42 MF 16.17.103.107 (n.2n), (n.3n), (n.p) and (n.alpha) Calculated with multi step Hauser Feshbach model using GNASH code 8 MF 4 Angular Distributions of Secondary Neutrons MT 2.51 62 \sim : Calculated with CASTHY code /6/. : Isotropic in the laboratory system. MT 16.17 : Assumed the same distributions in the laboratory MT 91 system as those calculated with CASTHY code in

MF 5 Energy Distributions of Secondary Neutrons MT 16.17.91 - Evaporation spectra.

References

- 1) Mughabghab S.F. and Garber D.I.: BNL 325 3rd Edition (1973).
- 2) Drake M.K., Sargis D.A. and Maung T.: EPRI NP-250 (1976).
- 3) Beer H. and Macklin R.L.: Phys. Rev., C26, 1404 (1982).

the center of mass system.

- 4 Conrad C.A. et al.: Bull. Am. Phys. Soc., 14, 496 (1969).
- 5: Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975).
- 6) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed. (1979).
- 7) Gilbert A. and Cameron A.G.W.: Can. J. Phys., 43, 1446 (1965).
- 8) Young P.G. and Arthur E.D.: LA-6947 (1977).

1 of Hafnium 180

MAT number - 2726 72 Hf 180 -----72 SE 180 NAIGE Eval Dec82 Hida, Yoshida, Itjima, Takano (JAERI) Dist Mar83 Rev1 Dec83 History 82 12 New evaluation for JENDL 2 was made by K.Hida, T.Yoshida, S. Lijima (NAIG) and H. Takano (JAERI). 83 12 Angular distributions were modified and comment was added. MF 1 General Information MT 451 Descriptive data and dictionary MF-2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters Resolved resonances for MLBW formula Energy range : 0.5 eV to 2.5 keV. Res. energies and Gam n (ENL 325 1 and Drake et al. (2/. : 0.060 eV assumed, if unknown. Gam gamma Unresolved resonances í Energy region 2.5 keV to 50 keV. : calculated with optical model. SO SLR and Gam gamma : fitted to the capture cross section of Beer and Macklin /3/. 140 eV, S0 1.92E 4, S1 0.496E 4, Parameters are D obs R 7.14 fm and Gam gamma 0.060 eV. 2200 m s cross sections and calculated res. integrals. 2200 m s Res. Integ. 32.00 b 4 total elastic 19.40 b 12.60 b 5 34.6 b capture MF 3 Neutron Cross Sections Below 0.5 eV: Point wise cross sections were given. From 0.5 eV to 50 keV: Background cross section of zero was given. Above 50 keV: MT=1.2.4.51 58.91,102 Total, elastic inelastic and capture Calculated with optical and statistical models. Optical potential parameters were fitted to the total cross section of natural hafnium. V0 = 38.0, Ws = 8.0+0.5; SQRT(En), Vso = 7.0 (MeV), a0 = 0.47, as = 0.52 , aso = 0.47 (fm), r0 - 1.32, rs = 1.32rso = 1.32 (fm). Statistical model calculation was made with CASTHY code /6/. Competing processes : n.2n . (n.3n . (n.p., (n.alpha), and level fluctuation were considered. Level scheme taken from Table of Isotopes 77 No. Energy MeV Spin-Parity 0.0 0 g.s. 0.0458 1 -1 2 0.0680 4 +

Ca	3 4 5 6 7 8 sontinuum	0.0986 0.1700 0.2070 0.2570 0.2980 0.3320 1 levels as:	2 5 3 1 6 + 2 sumed above	≥ 0.445 M	e¥.	
Leve and	el densi Cameron If 180	ty paramter is formula a(1/MeV)	rs were new 80, C(1/MeV)	T(MeV)	Ex (MeV)	ilbert
H MT 16.1 Cale GNAS MT 251 Cale	lf 181 7,103,1 wlated H code Mu bar wlated	21.91 07 (n.2n), with multi 9 . with CASTHY	6.47 (n.3n), (step Hause (code 6 .	0.479 0.p and r Feshbad	4.08 (n.alpha) sh model us	sing
MF 4 Ang MT 2.51 MT 16.1 MT 91	ular Di 58 : 7 :	stributions Calculated Isotropic Assumed th system as the center	s of Second I with CAST in the lab ie same dis those calc of mass s	lary Neutr HY code Horatory s stribution sulated wi system.	ons 6. system. is in the l th CASTHY	aboratory code in
MF 5 Ene MT 16.1	rgy Dis 7.91 :	tributions Evaporatio	of Seconda n spectra.	ry Neutro	ms	
Reference: 1 Mughal 2 Drake 3 Beer 1 4 Conrac 5 Schart 1937	s oghab S M.K., S H. and f f C.A. « If Goldl	.F. and Gar Sargis D.A. Macklin R.L. >t al.: Bul maber G. and	ber D.I.: and Maung .: Phys. R 1. Am. Phys d Meleown 1	BNL 325 3 T.: LPRI ev., C26, s. Soc., M.: Phys.	9rd Edition NP-250 (1 1404 (198 14.496 (1) Rev., 158	(1973). 976). 2 969). 1105

- 6 Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975).
 7 Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Ed. 1979 .
- 8) Gilbert A. and Cameron A.G.W.: Can. J. Phys., 43, 1446 (1965), 9) Young P.G. and Arthur E.D.: LA 6947 (1977).

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73-Ta-181 MAT number 2731

73-Ta 181 SRI + Eval-Mar76 H.Yamakoshi, JENDL CG.Y.Kikuchi Dist Mar83 Rev1 Nov83

History

- 76 03 The evaluation for JENDL 1 (1) was made by H.Yamakoshi Ship Research Institute) and JENDL 1 Compilation Group.
 83 03 JENDL 1 data were adopted for JENDL 2 and extended to 20
 - MeV. MF-5 was revised, and unresolved resonance parameters were added by Y. Kikuchi (JAER).
- 83 11 Comment data were added.
- MF 1 General Information

MT 451 Descriptive data and dictionary

MF 2 Resonance Parameters

MT 151 Resolved and unresolved resonance parameters Resolved parameters for MLBW formula

The energy region is from 1.0E-5 eV to 1.0 keV. Parameters were taken from Ref. 22 for positive resonances, and from ENDF.B I' for a negative resonance. The radiative width of 0.0559 eV was assumed for the resonances whose radiative width was unknown.

Unresolved parameters

In the energy range from 1 to 50 keV, parameters were determined to reproduce the capture cross section evaluated for JENDL 1. The calculated total and elastic scattering cross sections were corrected by background cross sections. The parameters are as follows.

 $R=4.435~{\rm fm}, D~{\rm obs}=4.39~{\rm eV},$ radiative width $\sim 0.0598~{\rm eV},$ S0= S.466E 4, S1= 0.45E 4, S2= 3.2E 4.

Calculated 2200 m sec cross sections and resonance integrals. 2200 m sec res. integ.

. 1	C 110 k	_		
erastic	0,110 0)		
capture	21.21 b)	744.	b
total	27.32 b	3		

MF-3 Neutron Cross Sections

Below 50 keV.

Background cross sections were given for the unresolved resonance parameters.

Above 50 keV.

MT=1 Total

Evaluated from experimental data.

MT=2 Elastic scattering

Obtained by subtracting partial cross sections from the total cross section.

Mi=4.51-61.91 Inelastic scattering

Calculated with statistica and optical model code CASTHY 3. Optical potential parameters were determined so as to reproduce the average total cross section in the high energy region.

 $V = 46.0 \cdot 0.25 E$ MeV), r 1.268, a0 0.62 (fm) Wi =0.125+E-0.0004+E++2 (MeV , ri 1.268. ai 0.62 fm Ws 14.0 0.2 E MeV, rs 1,316, as 0.7 f'm 🗉 Vso 6.0 (MeV . rso 1.099, aso 0.62 fm) The level scheme was adopted from Ref. 4. No. Energy (MeV -Spin Parity 721 g.s. 0.0 1 0.00621 92 2 0.13625 92. 3 112 0.1587 4 0.3014 112. 5 0.3390 13.2 6 0.482152. 7 13 2 . 0.4951 ß 0.5480 15.2 9 12. 0.6151 0.6190 10 32. 11 0.7166 15.2 Levels above 780 keV were assumed to overlapping. MT 16 n.2n Calculated with Pearlstein's method 5. MT 102 Capture Calculated withn CASTHY 3 by using D obs 4.4 eV and the average radiative width of 0.0598 eV. MT 103 (n.p. Calculated with Pearlstein's method 5. MT 251 Mu bar Calculated with CASTHY 3 . MF 4 Angular Distributions of Secondary Neutrons MT 2 Calculated with CASTHY code 3 MT 51 61 Isotropic in the center of mass system. MT 16.91 Isotropic in the laboratory system MF 5 Energy Distributions of Secondary Neutrons MT 16.91 "Vaperation spectrum References Igarasi S. et al.: JAERI 1261 1979 1 2) Mughabghab S.F. and Garber D.I.: BNL 325, 3rd Ed. (1975). 3) Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975). 4 Ellis Y.A.: Nucl. Data Sheets, 9, 319 (1973). 5 Pearlstein S.: Nucl. Sci. Eng., 23, 238 (1965).

1 of Natural Lead

82 Pb 0 MAT number 2820

82 Pb O JAERI Eval Mar81 T.Asami Dist Mar83 Revi Jan84

History

81 03 Newly evaluated for JENDL 2 by T.Asami JAERI .

84.01 Interpolation laws of cross sections and angular distributions were corrected. The total cross section was replaced with the data evaluated on the basis of experimental data. Comment was added.

MF 1 General Information MT 451 Descriptive data and dictionary

MF 2 Resonance Perlate term

MT 151 Resolved reconcise parameters for MLBW formula Resonance ranges

Pb 204. 1 OE 5 eV 50 keV. Pb 206: 1.0E 5 eV 200 keV Pb 207: 1.0e 5 eV 500 keV. pb 208: 1.0E 5 eV 500 keV Parameters were evaluated from the following exp. data. Pb 204: Block-63 1. G.bbons-67 2. Allen-73 3'. Pb 206: Allen-73 3. Horen-79 4. Mizumoto-79 5. Pb 207: Allen-73 3. Horen-77 6. Horen-79 4. Pb 208: Allen-73 3. Wilenbick-61 7. Macklin-77 8. Fowler66 9.

For unknown radiative widths, average values of known radia tive widths were used.

Calculated $\boxtimes 00$ m/s errors sections and rest integrals. $\boxtimes 2200$ m/s (rest integ.)

elastic	11.42 b	
Cartine	0.1716b	0-148-Б
total	11.50 h	

MF 3 Neutron Cross Sections

Below 500 keV

Background cross sections are given to take account of contribution from bound levels and to reproduce the thermal capture cross section of 0.171 barns which was evaluated from the experimental data -10-12. Contributions from Pb-204 and Pb-206 were added in the background cross section below 500 keV.

Above 500 keV.

MT-1 Total

Cross sections in the energies from 500 keV to 15 MeV were obtained based on the experimental data of Schwartz-77 (13). Above 15 MeV, cross sections were calculated with optical and statistical model code CASTHY (14) by using optical potentail parameters given below.

MT-2 Elastic scattering

(Total - (All other partial cross sections) MT=4.51-90.91 Inelastic scattering Calculated with CASTHY /14/ for each isotope and constructed taking account of their isotope abundances. Contributions from some levels of Pb 204 and 206 were lumped and contributions from levels above 2.9454 MeV were put together into continuum (MT-91). The optical potential parameters used in the calculation were obtained by fitting average total cross section of natural lead as follows.

 $V=47.0=0.250\,(E,~Ws=2.30\pm0.41\,(E,~Vso=6.0~(MeV),$ r0=1.25 , rs=1.30 , rso=1.30 (fm), a0=0.65 , b=0.48 , aso=0.689 (fm). Level density parameters for back shifted fermi gas model were determined using low lying level data and observed neutron resonance spacing. Some data for level density parameters were taken from Ref. 15 . Level scheme was taken from Ref. 16 . Level energies and corresponding isotopes are as follows:

MT	Energy MeV	Isotope	MĽ	Energy MeV	Isotope
51	0,5697	:207	71	2,7260	207
52	0,8031	206	12	3.0165	206
53	0.8977	207	73	3.1800	207
54	0.8991	204	74	3, 1977	208
55	1.1650	206	75	3,2000	207
56	1.2738	204	76	3 2230	207
57	1.3406	206	77	3.2793	206
-58	1.4670	206	78	3,3000	207
59	1.5629	204	79	3.3840	207
60	1.6333	207	80	3.4130	207
61	1.6841	206	- 81	3.4750	208
62	1.9977	206	82	3,5090	207
63	2.0650	204	83	3.5830	207
64	2.2641	204	84	3.7085	208
65	2.3399	207	85	3.7440	206
-06	2.3843	206	123	3,9199	208
67	2.6146	208	87	3.9464	208
62	2.6244	207	563	3,9609	208
GÐ	2.6470	200	- 3(3	3,9957	208
'70	2.0624	207	90	4.1252	203

MT 16.17.22.28.103.107 = n.2n , n.3n , n.n'p , n.n'a , (n.p) and n.a (

Calculated with evaporation model code GROGI /17/ for each isotope and constructed considering isotope abundances.

MT-102 Capture

Calculated with CASTHY /14/ for Pb-204, 206 and 207. For Pb-208, estimated from the experimental data of Pb-208 and natural lead. The capture cross section of natural lead were constructed from these isotopic data.

MT=251 Mu bar

Calculated with CASTHY 14

 MF-4 Angular Distributions of Secondary Neutrons

 MT-2
 : Calculated with CASTHY 14/.

 MT-16.17.22.28
 : Assumed to be isotropic in the lab system.

 MT 51
 90
 : Assumed to be isotropic in the center of mass system.

 MT=91
 : Assumed to be isotropic in the lab system.

MF 5 Energy Distributions of Secondary Neutrons MT 16,17,22,28,91 Evaporation spectra.

References

- 1 Block R.C. and Moxon M.C.: BAPS 8, 513 (1963).
- 2 Gibbons J.H. and Macklin R.L.: Phys. Rev., 153, 1356 (1967).
- 3 Allen B.J. et al.: Phys. Rev., C8, 1504 (1973).
- 4 Horen D.J. et al.: Phys. Rev., C20, 478 (1979).
- 5 Mizumoto M. et al.: Phys. Rev., C19, 335 (1979).
- 6 Raman S. et al.: Phys. Rev. Lett., 39, 598 (1977).
- 7 Wilenzick R.M. et al.: Phys. Rev., 121, 1150 (1961).
- 8 Macklin R.L. et al.: Astrophys. J., 217, 222 (1977).
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- 10 Cranton et al.: Nucl. Phys., A169, 95 1971 -.
- 11 Tattersall R.B. et al.: J. Nucl. Ener., A12, 32 1960 .
- 12 Stefanescu Al. et al : International Conf. on Phys. and Technol of Research Reacters. Bucurest. p.553 1961;
- 13 Schwartz R.B. et al., NRS mono 139, 1974 data in EXFOR 1971.
- 14 Igarasi S. J. Nucl. Sci. Technol., 12, 67 1975).
- 15 Dilg W. et al.: Nucl. Phys., A217, 269 (1973).
- 16 Lederer C.M. and Shirley V.F.: Table of Isotopes, 7th Edition 1978 .
- 17 Gilat J. BNL 50246 T 580 (1970).

1 of Lead-204

82-Pb-204 MAT number = 2821

82 Pb 204 JAERI Eval Mar81 T Asami Dist Mar83 Rev1 Nov83

History

- 81 03 Newly evaluated for JENDL 2 by T.Asami (JAERI).
- 83 11 Interpolation laws of cross sections and angular distributions were corrected. Comment data were added.
- MF | General Information

MT-451 Descriptive Data and Dictionary

MF 2 Resonance Parameters

MT 151 Resolved Resonance Parameters for MLBW Formula Resonance range : 10^{-5} eV to 50 keV. Parameters were evaluated from the data of Block+63 /1 . Gibbons 67 2 and Allen 73 31. Effective scattering radius of 8.5 fm was taken from Ref. 47. For unknown radiative widths, assumed average values of 1.2 eV for swave and 0.6 eV for p wave resonances.

Calculated 2200 m/s cross sections and res. integrals. 2200 m/s res. integ. elastic 11.34 b d 69.5 d 169.0 capture total 12.00 b

- MF 3 Neutron Cross Sections
 - Below 50 keV.

Background cross sections are given to take account of contribution from bound levels and to reproduce the thermal capture cross section of 0.661 barns 5.

Above 50 keV.

Cross sections were obtained from optical and statistical model calculations. The optical potential parameters were obtained by fitting average total cross section of natural lead as follows.

V = 47.0 = 0.250 + E, Ws = 2.30 + 0.41 + E, Vso = 6.0 (MeV), , aso = 0.689 (fm).Level density narameters were determined using low-lying level data and observed neutron resonance spacing. Some data for leve) density parameters were taken from Ref. 767. MT 1 Total Calculated with optical and statistical model code CASTHY/7/.

MT-2 Elastic Scattering

Total All other partial cross sections -MT-4.51 68.91 Inelastic Scattering Calculated with CASTRY 7 . Level scheme taken from Ref. 8. No. Energy (MeV) Spin-Parity g.s. 0.0 0 +

0.8991 2 +1 2 1.2738 4 1 з 2 1 1.3538 4 1.5629 A . 5 3 . 1.6047 6 1.8173 4 1 7 5 + 2.0650 8 2.1855 9 9 2.2578 5 7 10 2.2641 11 2.3381 5 12 2.3858 5 7 13 2.4050 14 2,4339 6 15 6 2.480116 2,5069 5 З 17 2.6271 7 18 2.6603 Levels above 2.945 MeV were assumed to be continuum. MT 16.17.22.28.103.107 n.2n, n.3n, n.n'p, (n.n'a), (n.p) and n.a. Calculated with evaporation model code GROGI /9/. MT 102 Capture Calculated with CASTHY /7 and normalized to 30 milliobarns at 100 keV. MT 251 Mu bar Calculated with CASTHY 7 . MF 4 Angular Distributions of Secondary Neutrons Calculated with CASTHY 7 . MT 2.51 68 MT 16.17.22.28 : Assumed to be isotropic in the lab system. MT 91 : Assumed the same distributions in the lab system as those calculated with CASTHY in the center of mass system. MF 5 Energy Distributions of Secondary Neutrons MT 16.17.22.28.91 : Evaporation spectra. References 1) Block R.C. and Moxon M.C.: BAPS 8, 513 (1963). 2) Gibbons J.H. and Macklin R.L.: Phys. Rev., 153, 1356 (1967). 3) Allen B.J. et al.: Phys. Rev., C8, 1504 (1973). 4) Macklin R.L. et al.: Phys. Rev., 136, B695 (1964). 5) Jurney E. et al.: ANL-6797, 236 (1963). 6) Dilg W. et al.: Nucl. Phys., A217, 269 (1973). 7) Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975). 8) Lederer C.M. and Shirley V.S.: Table of Isotopes, 7th Edition (1978).

9 Gilat J.: BNL-50246 (T-580) (1970).

82 Pb 206 MAT number = 2822

ter an energy of the second second

82 Pb 206 JAERI Eval Mar81 T.Asami Dist Mar83 Rev1 Nov83

History

- 81 03 Newly evaluated for JENDL 2 by T.Asami (JAERI).
- 83 11 Angular distributions were corrected. Comment data were added.
- MF-1 General Information

MT 451 Descriptive Data and Dictionary

MF 2 Resonance Parameters

MT 151 Resolved Resonance Parameters for MLBW Formula Resonance range = 10⁻¹ eV to 200 keV. Parameters were evaluated from the data of Allen(73, 1, Horen(79, 2) and Mizumoto(79, 3). Effective scattering radius of 8.5 fm was taken from Ref. 4. For unknown radiative widths, assumed average values of 0.8 eV for swave, 0.25 eV for p wave and 0.08 eV for d-wave resonances.

Calculated 2200 m s cross sections and res. integrals.

	2200 m s	res. integ.
elastic	11.34 b	
capture	0.028 Б	0.0962 Б
total	11. 37 b	

MF 3 Neutron Cross Sections

Below 200 keV.

Background cross sections are given to take account of contribution from bound levels and to reproduce the thermal capture cross section of 0.028 barns which was evaluated from experimental data to reproduce the thermal capture cross section of natural lead -0.171 barns .

Above 200 keV.

Cross sections were obtained from optical and statistical model calculations. The optical potential parameters were obtained by fitting average total cross section of natural lead as follows.

Calculated with optical and statistical model code CASTHY/6/. MT-2 - Elastic Scattering

Total - All other partial cross sections -

MT-4.51 67.91 Inelastic Scattering

Calculated with CASTHY /6/.

Level scheme taken from Ref. /7/.

.

No. g.s. 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 Levels MT 16. 17.22.2	Energy MeV 0.0 0.8031 1.1650 1.3403 1.4670 1.6841 1.7030 1.7840 1.9977 2.1490 2.3843 2.4240 2.6479 3.0165 3.1220 3.2793 3.744 above 4.027 Me 8.102.107 m.	Spin Pa 0 2 2 2 4 4 1 2 4 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5 5	sumed to be continuum.
Calculated MT 102 Captu Calculated at 50 keV. MT 251 Mu ba	with evaporat re with CASTHY	fon model 6 and no	code GROGI /8/. rmalized to 4 milli-barns
Calculated MF 4 Angular D. MT-2.51 67 MT 16.17.22.28 MT 91	with CASTHY istributions o : Calculated 3 : Assumed to : Assumed the system as center of a	6 . f Secondal with CASI be isotro e same dis those calc mass system	ry Neutrons FHY 6 opic in the lab system. stributions in the lab culated with CASTHY in the En.
MF-5 Energy Dis MT 16.17.22.28	stributions of 3.91 Evaporat	Secondary	/ Neutrons Lro.
References 1 > Allen J.B. e 2 > Horen D.J. e 3 > Mizumoto M. 4 > Macklin R.L. 5 > Dilg W. et a 6 > Igarasi S.: 7 > Lederer C.M. (1978 - 8 > Gilat J.: BN	et al.: Phys. I et al.: Phys. F et al.: Phys. et al.: Phys. l.: Nucl. Phys. J. Nucl. Sci. and Shirley V L-50246 (T-580	Rev., C8, Rev., C20, Rev., C19 Rev., 13 , A217, Technol., V.S.: Tabl	1504 (1973). 478 (1979). 3, 335 (1979). 6, B695 (1964). 269 (1973). 12, 67 (1975). e of Isotopes, 7th Edition

82-Pb-207 MAT number = 2823

82-Pb-207 JAERI Eval Mar81 T.Asami Dist Mar83 Revi Nov83

History

81 03 Newly evaluated for JENDL 2 by T.Asami (JAERI).

- 83 11 Angular distributions were corrected. Comment data were added.
- MF 1 General Information

MT 451 Descriptive Data and Dictionary

MF 2 Resonance Parameters

MT 151 Resolved Resonance Parameters for MLBW Formula Resonance range 1.0E 5 eV to 500 keV. Parameters were evaluated from the data of Allen 73 1 , Raman 77 2 and Horen 78 3. Effective scattering radius of 8.04 fm was taken from Ref. 3. For unknown radiative widths, assumed average value of 9.1 eV for swave resonances.

Calculated 2200 m s cross sections and res, integrals. 2200 m s res, integ. elastic 11.34 b capture 0.703 b 0.375 b total 12.04 b

MF 3 Neutron Cross Sections

Below 500 keV.

Background cross sections are given to take account of contribution from bound levels and to reproduce the thermal capture cross section of 0.703 barns which was evaluated from experimental data to reproduce the thermal capture cross section of natural lead 0.171 barns.

Above 500 keV.

Cross sections were obtained from optical and statistical model calculations. The optical potential parameters were obtained by fitting average total cross section of natural lead as follows.

Total All other partial cross sections

MT=4.51-67.91 Inelastic Scattering

Calculated with CASTHY /5/. Level scheme taken from Ref. /6/.

No. g.s. 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 10	Energy (MeV) 0.0 0.5697 0.8977 1.6333 2.3399 2.6244 2.6624 2.7260 3.18 3.2 3.223 3.3 3.384 3.413 3.509 3.593 3.593 3.624	Spin Par: 122 522 1322 522 1322 522 1322 122 122 122 122 122 122 122 122	.ty
Levels MT 16.17.22.2	above 4.100 Mc 28.103.107 n.;	V were assu 2nn.3n ·.	med to be continuum. n.n'p).(n.n'a).(n.p) and
Calculated The calcul milli barn cross sect other cros as for the MT 102 Captu Calculated at 50 keV. MT 251 Mn ba Calculated	l with evaporat ated (n.p. cros is at 14.5 MeV fon was normal: is sections were (n.p.) cross so re with CASTHY - S r the CASTHY - S	ion model c ss section (Beloviteki ized to 1.8 e also norm ection 5 and norm	ode GROGI /7/. was normalized to 1.6 j:76 8/). The (n.2n) 9 borns at 15.5 MeV. alized by the same factor alized to 2 milli barns
MF 4 Angular D MT 2.51 67 MT 16.17.22.2 MT 91	Istributions of Calculated 8 : Assumed to : Assumed the system as t center of m	Secondary with CASTIF be isotrop. same dist chose calcul mass system	Neutrons (5) ic in the lab system, ributions in the lab lated with CASTHY in the
MF-5 Energy Di. MT 16.17.22.2	stributions of 8.91 : Evaporat	Secondary 1 ion spectra	leutrons A.
References 1) Allen J.B. (2) Raman S. et 3) Horen D.J. (4) Dilg W. et (5) Igarasi S.: 6) Lederer C.M. 1973 . 7) Gilat i BN	et al.: Phys. R al.: Phys. Rev et al.: Phys. R al.: Nucl. Phys J. Nucl. Sci. . and Shirley V	Rev., C8, 18 . Lett., 39 Rev., C20, 4 ., A217, 26 Technol., 1 .S.: Table	04 (1973).), 598 (1977). (78 (1979). 99 (1973). 2, 67 (1975). of Isotopes, 7th Edition

7) Gilat j.: BNL 50246 T-580) (1970). 8) Belovitckij G.E.: 1975 Kiev Conf., Vol.4. 209 (1976).

1 of Lead-208

82-Pb-208 MAT number 2324

82-Pb 208 JAERE Eval Mar81 T.Asami Dist Mar83 Rev1 Nov83

History

- BI 03 Newly evaluated for JENDL 2 by T.Asami (JAERI).
- 83-11 Angular distributions were corrected. Comment data were added.
- MF-1 General Information

MT 451 Descriptive Data and Dictionary

MF 2 Resonance Parameters

MI 151 Resolved Resonance Parameters for MLBW Formula Resonance range 1.0E 5 eV to 800 keV. Parameters were evaluated from the data of Allen(73 /1/, Wilenzick 61 2. Macklin(77 3 and Fowler66 4/. Effective scattering radius of 6.5 fm was selected. Neutron width of 515 keV s wave resonance was estimated to reproduce the 2200m sec capture cross section of 0.48 milli-barns /5/.

Calculated 2200 m s cross sections end res. integrals. 2200 m s res. integ.

elastic	11.49 Б	-
capture	0.4799 milli b	7,83 milli-b
total	11.49 Б	

MF 3 Neutron Cross Sections

Below 800 keV.

Background cross sections are given for the elastic scattering cross section.

Above 800 keV.

Cross sections were obtained from optical and statistical model calculations. The optical potential parameters were obtained by fitting average total cross section of natural lead as follows.

MT-1 Total

Calculated with optical and statistical model code CASTHY/7/. MT 2 Elastic Scattering

Total All other partial cross sections;

- MT-4.51-64.91 Inelastic Scattering Calculated with CASTHY 7. Level scheme taken from Ref. 8. No. Energy MeV Spin Parity g.s. 0.0 0+
 - 1 2.6146 3 -

$\begin{array}{cccccccccccccccccccccccccccccccccccc$
Levels above 4,577 MeV were assumed to be continuum. MT 16.17 = n.2n + and (n.3n) Calculated with evaporation model code GROGI [9]. The (n.2n) cross section was normalized to 2 OL barns at 15.5 MeV
MT 22 n.n'alpha calculated with code GROGI 9 and normalized by the same factor as for no gross section
MT 28 n.n.p Calculated with code GROGI /9/ and normalized by the same
 factor as for (n,p) cross section. MT 102 Capture Estimaed from the following experimental data: Leipunskii+58 /10/. Csikai+67 /11/. Drake+71 /12/. Bergqvist+72 /13. and Diven+60 /14/. MT 103 (n,p) Calculated with code GROGI 91 and normalized to an average value of experimental data by Hankla+72 /15/ and Belovitckij+76 /16/ (0.48 milli-barns at 14.5 MeV). MT 107 (n.a) Calculated with code GROGI 91 and normalized to 1.5 millibarns at 14.5 MeV by Coleman 59 17. MT-251 Mu bar Calculated with CASTHY 7. MF-4 Angular Distributions of Secondary Neutrons MT-25.1 64 : Calculated with CASTHY/7/. MT-16,17.22.28 : Assumed to be isotropic in the lab system. MT=91 : Assumed the same distributions in the lab system.
center-of-mass system. MF-5 Energy Distributions of Secondary Neutrons
MT=16,17.22,28,91 : Evaporation spectra.
 Kererences 1) Allen J.B. et al.: Phys. Rev., C8, 1504 (1973). 2) Wilenzick R.M. et al.: Phys. Rev., 121, 1150 (1961). 3) Macklin R.L. et al.: Astophys. J., 217, 222 (1977). 4) Fowler J.L.: Phys. Rev., 147, 870 (1966). 5) Emery J.F.: ORNL 4341, 71 (1968). 6) Dilg W. et al.: Nucl. Phys., A217, 269 (1973). 7) Igarasi S.: J. Nucl. Sci. Technol., 12, 67 (1975). 8) Lederer C.M. and Shirley Y.S.: Table of Isotopes, 7th Edition

3 of Lead-208

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- 9) Gilat J.: BNL-50246(T-580) (1970).
- 10) Leipunskii et al.: 2nd Geneva Conf., 15, 50 (1958).
- 11) Csikai J. et al.: Nucl. Phys., A95, 279 (1967).
- 12) Drake D. et al.: Phys., Letters, B36, 557 (1971).
- 13) Bergqvist I. et al.: Nucl. Phys., A191, 641 (1972).
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 15) Hankla A.K. et al.: Nucl. Phys., A180, 157 (1972).

- 16) Belovitckij G.E.: 1975 Kiev Conf., Vol.4, 209 (1978).
- 17) Coleman R.F. et al.: Proc. Roy. Soc. (London), 73,215 (1959).

90 Th 228 MAT number 2901

90 Th 228 Kyushu U. Eval Apr31 T.Ohsawa and M.Ohta Dist-Mar83 Revi-Nev83

History

- 81 04 New evaluation was made by T. Osawa and M. Ohta (Kyushu University). Details of the evaluation are described in Ref. (17).
- 83-11 Fission spectrum was added. Resonance formula was changed to MLBW formula. The total, (n.2n) and (n.3n) cross sections were modified. Comment was added.
- MF 1 General Information
 - MT 451 Comments and dictionary
 - MT 452 Tota' number of neutrons cmitted per fission Calculated with the semi empirical formula of Howerton 2'.
- MF 2 Resonance Parameters
 - MT 151 Resolved resonances

Resonance region is below 7.798 eV. Parameters were given for the MLBW formula. Only two resonances were observed by Simpson et al. 37. An additional term with 17v dependence was assumed to reproduce the thermal capture cross section. Fission cross section was also assumed to have 17v behavior.

Calculated 2200 m s cross sections and res. integ.(barns) 2200 m s Res. Integ.

elastic	12.81	
capture	119.9	1170
fission	0.200	1.02
total	133.0	

MF 3 Neutron Cross Sections

Below 7.798 eV is the resonance region. Background data were given. The cross sections were evaluated above 7.798 eV as follows.

MT-1 Total cross section

Optical model calculation with the following parameters: V = 41.0 = 0.05 + E (MeV). Ws = 6.4 + 0.15 + SQRT(E) (MeV). - der, Woods-Saxon ----

Vso= 7.0(MeV).r0 = rso = 1.31(fm).rs = 1.38(fm).a = b = aso = 0.47(fm).These parameters were taken from those for Th-232 /4/.

These parameters were taken from those for Th-232 /4/. MT-2 Elastic scattering cross section

Statistical and opical model calculations using the code CASTHY 5

MT-4.51-62.91 Inelastic scattering cross section Statistical and optical model calculations.

> Level scheme of Th-228 /6/. No. Energy (MeV) Spin-Parity

g.s. 1 2 3 4 5 6 7 8 9 10 11 12 Levels abo	0.0 0.0576 0.1869 0.328 0.3961 0.5193 0.8317 0.8746 0.9441 0.952 0.9688 1.016 1.0224 we 1.025 MeV	0 + 2 + 1 3 5 0 2 + 1 2 + 3 4 Were assume	ed to be overlapping.
MT 16.17 n.2n a Calculated by Caner 7	nd n.3n or means of the	ross section: e evaporation	s n model of Segev and
MT 18 Fission er The data of Vo The fission er isotope Th-230 Vorotnikov et	ess section rotnikov et oss section normalized al. was adop	al. 8 were of the neigh to join smoo ted above 5	a adopted up to 5 MeV. hboring even even othly to the data of MeV.
MT 102 Capture er Statistical an strength funct	oss section d optical me ion of 0.007	del calculat 91.	tions with gemma ray
MT 251 Mu bar Calculated wit	h optical mo	del.	
MF 4 Angular Distri MT 2.51 62.91	butions of S	econdary Neu	itrons
Statistical and MT 16.17.18 Assumed to be	d optical mo isofropic in	del calculat	.16NS. .orv.svstem
MF 5 Energy Distrib	itions of Se	condary Neut	rons
Evaporation spe MT=18	ectra		
Fission spectru et al. 97.	um estimated	from Z++2 A	systematics by Smith
References i) Ohsawa T. and Oht Kyushu Univ. 40, 2) Howerton R.J.: Nu 3) Simpson O.D. et a 4) Ohsawa T. and Oht 5) Igarasi S.: ibid. 6) Lederer C.M. and 7th Edition 1978 7) Segev M. and Cane	ta M.: Memoii 149 (1980). 161. Sci. En 11.: ibid. 20 12. 67 (197 Shirly V.S. 13. 14. Ann. N	rs Faculty o 5. 62, 438 (9. 423 (1967 cl. Sci Tecl 75). Ed. : Table Wucl. Energy	f Engineering, 1977).), hmol. 18, 408 (1981). e of Isotopes. 5. 239 (1978).

9) Smith A.B. et al.: ANL NDM 50 (1979).

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1 of Thorium 230

90 in 230 MAT number 2902

90 Th 230 Kyushu U. Eval Apr81 T.Ohsawa and M.Ohta Dist Mar83 Revi Nov83

flistory

81 04 New evaluation was made by T. Ohsawa and M. Ohta (Kyushu University). Details of evaluation are described in Ref. 1.

- 83 11 Fission spectrum was added. Pesonance parameters, and total. n.2n and m.3n cross sections were modified. Comment data were added.
- MF 1 General Information
 - MT 451 Comments and Dictionary
 - \mathbb{M} 452. Total number of numbers emitted per fission $\mathcal{D}_{\rm ele}$, the sum empirical formula of Howerton 2 .

MF 2 Resonance Parameters

MT 151 Resolved resonances

Resonance region is below 564.20 eV. The MLBW formula was selected to reproduce resonance cross sections. A total number of 28 resonances up to 563 eV measured by Kalebin et al. 3 were adopted in the present evaluation. A background term with 1 v dependence was added in order to reproduce the thermal capture cross section.

Calculated 2200 m s cross sections and res. integ. barns) 2200 m s — Res. Integ.

elastic	9.774	
capture	12.55	1040
fission	0.0	1.12
total		

MF 3 Neutron Cross Sections

Below $564.26~{\rm eV}$ is the resonance region where the background cross sections are given. Above $564.26~{\rm eV}$, the cross sections were evaluated as follows.

MT 1 Total cross section

Optical model calculation with the following parameters:

v	41.0 0.05 E	mev .		
Ws	6.4 + 0.15 SQRT E	MeV .	der.	Woods-Saxon
Vse	5 7.0	MeV I.		
r0	rso 1,31	(fm).		
rs	1.38	(fm).		
а	b aso 0.47	fm:		
These	parameters were taken	from those	∋ for	Th 232 /4/.
MT-2 E	lastic scattering cros	s section		
Statis	stical and opical mode	l calculat;	ions i	sing the code
CASTH	75.			
MT 4.51 63	3.91 Inelastic scatte	ring cross	secti	on
Statis	stical and optical mod	el calculat	ions.	
Le	evel scheme of Th-230	/6/.		

No.	Energy (MeV)	Spin Parity
g.s.	0,0	0 ·
1	0.0534	2 -
2	0.173	4
3	0.357	6 .
4	0.506	1
5	0.571	3
6	0.635	0 -
7	0.678	2 -
8	0.682	5
9	0.781	5 .
10	0,881	4 ·
11	0.951	1
12	1.009	2 ·
13	1.012	3

Levels above 1.02 MeV were assumed to be overlapping.

- MT 16.17 n.2n and n.3n cross sections Calculated by means of the evaporation model of Segev and Caner 7.
- MT 18 Fission cross section Evaluation was made on the basis of the data of Muir et al. 8 up to 2 MeV. Above 2 MeV, the fission probability data of Back et al. 9 were used to calculate the fission cross section.
- MT 102 Capture cross section Statistical and optical model calculations with gamma ray strength function of 0.00791.
- MT 251 Mu bar Calculated with CASTHY.
- MF 4 Angular Distributions of Secondary Neutrons MT 2.51 63.91

Statistical and optical model calculations.

MT 16.17.18

Assumed to be isotropic in the laboratory system.

- MF 5 Energy Distributions of Secondary Neutrons
 - MT 16.17.91

Evaporation spectra.

MT 18

Fission spectrum estimated from Z(+2)A sistematics by Smith et al. -10 .

References

- 1) Ohsawa T. and Ohta M.: Memoirs Faculty of Engineering. Kyushu Univ. 40, 149 (1980).
- 2 Howerton R.J.: Nucl. Sci. Eng. 62, 438 (1977).
- 3 Kalebin S.M. et al.: Sov. J. Atom. Energy 26, 588 (1969).
- 4 Chsava T. and Ohta M.: J. Nucl. Sci. Technol. 18, 408 (1981).
- 5 Igarasi S.: ibid. 12. 67 1975.
- 8-Lederer C.M. and Shirly V.S. Ed. : Table of Isotopes. 7th Edition 1978 .
- 7 Segev M. and Caner M. Ann. Nucl. Energy 5, 239 1978 ...
- 8: Muir D.W. et al.: Proc. 3rd Conf. on Neutron Cross Sections and Technology, Knoxville (1971), p.292.
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3 of Thorium 230

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9) Back B.B. et al.: Phys. Rev. C13, 2374 $^\circ$ 1974). 10) Smith A.B. et al.: ANL NDM-50 $^\circ$ 1979 .

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1 of Thorium-232

90 Th-232 MAT number = 2903

Territoria in a constant degrade en estate

90 Th-232 Kyushu U. Eval Mar80 T.Ohsawa and M.Ohta NST 18.408(1981) Dist Mar83 Rev1 Nov83 History 80 03 New evaluation for JeNDL 2 was made by T. Ohsawa and M. Ohta Kyushu University : Details given in Ref. 11. 83-11 Comment. was added. MF 1 General Information MT 451 Comments and dictionary MT 452 Number of neutrons per fission Sum of NT 455 and MT 456. MT 454 Fission product vield data Taken from ENDF B 1V. MT 455 Delayed rentron data MT 456 Number of prompt neutrons per fission A single linear fit by Conde and Holmberg /2 was adopted. MF 2 Resonance Parameters MT 151 Resolved resonances for MLBW formula : 1.0E 5 eV - 3.5 keV Resonance parameters taken from Rahn et al. /3/ and BNL/ 325. 3rd Edition. 14 negative resonances were added. Unresolved resonances: 3.5 keV 50 keV Parameters determined so as to reproduce the evaluated smooth cross sections. Strength functions have slight energy dependence. D J 1 2 18.64 eV. Gam g 21.2 mil Average SD 0.94E 4. Average S1 2.00E 4. - 21.2 milli eV. Calculated 2200 m s cross sections and res. integrals barns) 2200 m s Res Inter. 12,16 elastic 0.0 7,2**5**8 0,cs 79,9 0.636 fission capture total 19.42 MF 3 Neutron Cross Sections Below 50 keV, background cross sections for unresolved resonance

Below 50 keV, background cross sections for unresolved resonance parameters were given. The cross sections represented with the unresolved resonance parameters and those above 50 keV were evaluated as follows.

r0 = rso = 1.31

(fm),

rs 🗇 1.38	(ſm),
a - b - aso - 0.47	⊂rîm ∿.
Derivative Woods Saxon	form assumed for surface
imaginary part.	

Statistical model calculation with CASTHY code (8). Competing processes (fission, m.2m and m.3m) Level width fluctuation considered.

The	level	scheme taken	from Refs. 9 and	10 .
	No.	Energy (ke	V) Spin Parity	
	g.s.	0.0	Ö i	
	1	49.5	5 +	
	2	162.5	4 +	
	3	333.0	G ·	
	4	555.0	8 -	
	! /	714.3	1	
	C	720.4	0	
	i	76.1	2	
	8	774.3	3	
	9	786.3	2 .	
	10	829,7	3 +	
	11	873.1	4	
	12	883.3	5	
	13	890.4	4 +	
	14	960.3	5 .	
	15	1053.9	2	
	16	1073.3	2 +	
	17	1077.7	1	
	18	1078.8	0	
	19	1095.0	3 -	
	50	1106.0	3	
Cont	1 muum	levels assumer	Labove 1.11 MeV.	
The lev	el den	sity parameter	's of Gilbert and (Lameron /11/
was use	el.			

MT 16.17 n.2n and n.3n

Calculated by means of the evaporation model of Segev et al. -12° .

MT-18 Fission

Evaluated on the basis of the data of Behrens et al. /13/. using the U 235 fission cross section data of Matsunobu $^{-14}$.

MT=102 Capture

Evaluated on the basis of the measured data of Kobayashi et al. $<\!\!15\!\!<$ and Lindner et al. $<\!\!16\!\!<$

MF=4 Angular Distributions of Secondary Neutrons

MT=2.51-70.91 Statistical model calculation with CASTHY /8/. MT=16.17.18 Isocropic in the laboratory system.

MF=5Energy Distributions of Secondary NeutronsMT=16.17Evaporation spectrum.MT=18Fission spectrum Watth.MT=455Taken from ENDF.B-IV.

References

- 1) Ohsawa T. and Ohta M.: J. Nucl. Sci. Technol. 18, 408 (1981).
- 2) Conde H. and Holmberg M.: Proc. Symp. Phys. and Chem. of Fission, 2, 57 (1965).
- 3) Rahn F. et al.: Phys. Rev. C6, 1854 (1972).
- 4) Foster D.G. et al.: private communication; Phys. Rev. C3, 576 (1971).
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- 6) Kobayashi K. et al.: Nucl. Sci. Eng. 65, 347 (1978),
- 7) Uttley C.A. et al.: EANDC Conf. TOP Methods (1961) p.109; 1st Conf. Nuclear Data for Reactors, Paris (1966).
- 8) Igarasi S.: J. Nucl. Sci. Technol. 12, 67 (1975).
- 9 McMurray W.R. et al.: private communication (1978).
- 10 McGowan F.K.: proc. Heavy Ion Summer Study. (1972) p.32.
- 11 Gilbert A. and Cameron A.G.W.: Can. J. Phys. 43, 1446 (1965).
- 12 Segev M. and Caner M.: Ann. Nucl. Energy 5, 239 (1978)
- 13 Behrens J.W. et al.: UCID 17442 1977
- 14 Matsunobu H.: private communication 1979 . MAT-2923 of JENDL 2
- 15 Kobayashi K. et al.: J. Nucl. Sci. Technol. 18, 823 (1981).
- 16 Lindner M. et al.: Nucl. Sci. Eng. 59, 387 (1976).

90 Th 233 MAT number = 2904

90-Th 233 Kyushu U. Eval Apr81 T.Ohsawa and M.Ohta Dist Mar83 Rev1 Nov83

History

- 81-04 Evaluation was made by T. Ohsawa and M. Ohta (Kyushu University). Details of the evaluation are described in Ref. (1).
- $83\cdot11$ Fission spectrum was added. The total, (n,2n) and (n,3n) cross sections were modified. Comment was added.
- MF 1 General Information
 - MT 451 Comments and dictionary
 - MT 452 Total number of neutrons emitted per fission Calculated with the semi empirical formula of Howerton 21.
- MF 2 Resonance Parameters
 - MT 151 Resolved resonances

No resolved resonances were adopted, since there were no measurements made. Capture and fission cross sections at 0.0253 eV were extrapolated up to 200 eV by assuming the form of 1 \vee for the former, and up to 20 keV by assuming the form of 1 \vee plus the constant value of 0.3 barns for the latter.

Calculated 2200 m s cross sections and res. integ. (barns) 2200 m s Res. Integ. elastic 13.0 capture 1450.0 643 fission 15.0 11.1

MF 3 Neutron Cross Sections

total

MT-1 Total cross section

These parameters were taken from those for Th-232 /3/. MT=2 Elastic scattering cross section

1478.0

- Statistical and opical model calculations using the code CASTHY 4 .
- MT=4.51-65.91 Inelastic scattering cross section Statistical and optical model calculations.

Level scheme of Th-233 5 . No. Energy(MeV) Spin-Parity g.s. 0.0 1/2 + 1 0.01687 3/2 + 2 0.05456 5/2 +

3	0.09363	72 -
4	0.37121	52
5	0,53958	1 2
6	0.58393	12.
7	0.6115	32
8	0.62902	52+
9	0.6822	32
10	0.7135	12+
11	0.7218	32 .
12	0.7695	52,
13	0.8145	32 -
14	0.8914	32 -
15	0.9476	32

Levels above 0.95 MeV were assumed to be overlapping.

- MT 16.17 $^-$ (n.2n) and (n.3n) cross sections Calculated by means of the evaporation model of Segev and Caner -6 .
- MT 18 Fission cross section Fission probability deduced from direct reaction /7, 8/ was used to calculate the fission cross section.
- MT 102 Capture cross section Statistical and optical model calculations with gamma-ray strength function of 0.00352.
- MT 251 Mu bar Calculated with optical model.

MF-4 Augular Distributions of Secondary Neutrons MT 2.51-65.91

- Statistical and optical model calculations.
- MT 16.17.18

Assumed to be isotropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

MT 16,17.91

- Evaporation spectra.
- MT 18

Fission spectrum estimated from $Z(\cdot,2)A$ systematics of Smith et al. \otimes

References

- 1) Ohsawa T. and Ohta M.: Memoirs Faculty of Engineering. Kyushu Univ. 40, 149 (1980).
- 2) Howerton R.J.: Nucl. Sci. Eng. 62, 438 (1977).
- 3) Ohsawa T. and Ohta M.: J. Nucl. Sci. Technol. 18, 408 (1981).
- 4) Igarasi S.: ibid. 12, 67 (1975).
- 5) 'ederer C.M. and Shirly V.S. (Ed.): Table of Isotopes, (th Edition (1978).
- 6) Segev M. and Caner M.: Ann. Nucl. Energy 5, 239(1978).
- 7) Back B.B. et al.: Phys. Rev. C13, 2374 (1974).
- 8) Cramer J.D. and Britt H.C.: Nucl. Sci. Eng. 41, 177 (1970).
- 9) Smith A.B. et al.: ANL NDM 50 (1979).

90-Th-234 MAT number = 2905 an experimental data and a second second

90 Th 234 Kyushu U. Eval Apr81 T.Ohsava and M.Ohta Dist Mar83 Rev1 Nov83

History

- 81-04 Evaluation was made by T. Ohsawa and M. Ohta (Kyushu University). Details of the evaluation are described in Ref. 1 .
- 83 11 Fission spectrum was given. The total. (n.2n) and (n.3n) cross sections were modified. Comment was added.
- MF 1 General Information

 - MT 451 Comments and dictionary MT 452 Total number of neutrons emitted per fission Calculated with the semi empirical formula of Howerton 2 .
- MF-2 Resonance Parameters
 - MT 151 Resolved resonances

No resolved resonances were adopted, since there were no measurements made. Capture and fission cross sections at 0.0253 ev were extrapolated on an 1 v basis up to an energy of 15 eV.

Calculated 2200 m s cross sections and res. integ. (barns) 2200 m s Res. Integ.

		nes, integ.
elastic	13.0	
capture	1.75	93.7
fission	0.0	0.26
total	14,75	

MF 3 Neutron Cross Sections

MT 1 Total cross section

Obtical model calculation with the following parameters: V 41.0 0.05 E (MeV)

₩S -	6.4 +	0.15 SQRE(E)	(Mev),	1.1	der.	Woods Saxon	1000 × 1
Vso	7.0		(MeV).				
r0 -	rso	1.31	(fm).				
rs -	1.38		(f m).				

a b - aso- 0.47 (fm).

These parameters were taken from those for Th-232 /3/. Elastic scattering cross section MT:2

Statistical and opical model calculations using the code CASTHY 4

MT=4.51-67.91 Inelastic scattering cross section Statistical and optical model calculations.

> Level scheme of Th 234 (estimated from systematics) Energy MeV Spin Parity No. 0.0 _**0** + g.s. 2 + 0.048 1 2 0.160 4 + 0.336 3 6+ 4 0.576 8 +

5	0.730	0 +
6	0.767	2 ,
7	0.785	2 ·
8	0.853	4 .
9	0.882	1
10	0.889	4 -
11	0.942	3
12	0,997	6 (
13	1.050	5
14	1.053	6 +
15	1.073	8 -
16	1.206	7.
17	1.277	8 -

Levels above 1.06 MeV were assumed to be overlapping.

- MT 16.17 n.2n and n.3n cross sections Calculated by means of the evaporation model of Segev and Cenet 5
- MT 18 Fission cross section Fission probability deduced from d.rect reaction 6 and systematics of Behrens 7 were used to obtain fission cross section.
- MT 102 Capture cross section

Statistical and optical model calculations with gamma ray strength function of 0.00791.

MT 251 Mu bar Calculated with optical model.

MF 4 Angular Distributions of Secondary Neutrons MT 2.51 67.91

Statistical and optical model calculations.

MT 16.17.18 Assumed to be isotropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

MT 16.17.91

Evaporation spectra were given.

MT 18

References

- 1 Ohsava T. and Ohta M.: Memoirs Faculty of Engineering. Kyushu Univ. 40. 149 1980
- 2) Howerton R.J.; Nucl. Sci. Eng. 62, 438 (1977).
 3) Ohsava T. and Ohta M.; J. Nucl. Sci. Technol. 18, 408 (1981).
- 4) Igarasi S.: ibid. 12, 67 (1975).
- 5) Segev M. and Caner M.: Ann. Nucl. Energy 5, 239(1978).
- 6) Back B.B. et al.: Phys. Rev. C13, 2374 1974 .
- 7 MBehrens J.W.: UCID 17509 2 1977 : Phys. Rev. Lett. 39, 68 1977 .
- 8 Smith A B. et al.: ANL NDM 50 1979 .

Fission spectrum was estimated from Z-2 A sistematics of Smith et al. 8%

1 of Protactinium 233

91 Pa 233 MAT number 2911

91 Pa 233 Kyushu u. Eval Mar76 Y.Kanda, JENDL CG Dist Mar83 Revi Jan84

History

76.03 The evaluation of Drake and Nichols 1 was recommended by Kanda (Kyushu univ.) for JENDL 1. JENDL 1 compilation group made partly recaluculation with optical and statis model.

83-03 JENDL 1 data were adopted for JENDL 2 and extended to 20 MeV. MF 5 was revised.

84 01 Small modification. Comment data were added.

MF 1 General Information

MT 451 Descriptive data and dictionery

MT 452 Number of neutrons per fission

Taken from Drake and Nichols 1 . and extended to 20 MeV.

MF 2 Resonance Parameters

MT-151 Resolved and unresolved resonance parameters

Resolved resonances for SLBW formula : from 2.38 to 17 eV. Taken from the evaluation of Drake and Nichols based on the data by Simpson and Codding 2.

Unresolved resonances : from 17 eV to 1 keV.

Taken from the evaluation of Drake and Nichols.

2200 m sec cross sections and calculated resonance integrals.

2200 m sec res. integ,

elastic	11.02 b	
capture	42.80 b	779 b
fission	0.0 Б	4.68 b
totat	53.52 5	

MF 3 Neutron Cross Sections

Below 2.38 eV.

The evaluation of Drake and Nichols was adopted. The cross sections are calculated values from resonance parameters in ME-2.

Between 2.38 eV to 1 keV.

Background cross sections of zero were given.

Above 1 keV.

MT-1 Total

The sum of partial cross sections

MT-2 Elastic scattering

Taken from Drake and Nichols' evaluation.

MT=4.51-66.91 Inelastic scattering

Calculated with CASTHY 3. The fission. (n.2n), and (n.3n) were considered as competing processes. The optical potential parameters determined by Igarasi 4 were adopted. $V0 = 40.5 + 0.5 \cdot \text{En}$ MeV Ws = 8.2 + 0.5 \cdot SQRT(En = (MeV)

a0 = as = as The level sche	so ≕ 0.47 me used in the	(im) calculation is as follows,
No.	Energy (MeV)	Spin-Parity
g.s.	0.0	3/2 -
1	0.0067	1/2 ~
ے۔ م	0.0371	1/6 5 0
3	0.0965	5/2 ** 5/2 ·
5	0.094''	321
Ğ	0.1037	7 2 +
7	0.1090	97 2 4
8	0.1633	11/2
9	0.1692	1.2.1
10	0.1800	92
11	0.2017	32 · 59
12	0.2120	52
18	0.2795	72.
15	0.3004	72
16	0.3662	9 2
Continuum level	s vere assumed	d above 400 keV.
MT=16,17 (n.2n) a	ind (n.3n)	_
Calculated with	Pearlstein's	method 75%.
MT 18 Fission		m)
Taken from Drak	e and Nichols	The cross section was obtained
by graving a sm for the renge f	both curve the	Tough the experimental data /0,//
to have the same	n ginana ng gin	nitar nuclei.
MT 102 Capture	e anope da ari	HIGH MACECI:
Taken from Drak	e and Nichols.	They calculated the capture
cross section f	rom the unrest	lved resonance parameters.
MT-251 Mu bar		
Calculated with	CASTHY 3 .	
		u,
MF 4 Angular Distri	butions of Sec	condary Neutrons
NT 51 CC	Teotropic in	the conter of magg sustem
MT 16 17 18 91	Isotropic in	the laboratory system.
11.10.17,10.91	isociopie in	the fabriatory system.
MF-5 Energy Distrib	utions of Seco	ndary Neutrons
MT=16.17.91	Evaporation s	pectrum.
MT-18	Maxwellian fi	ssion spectrum estimated from
	Z++2/A system	atics 787.
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1 of Uranium-233

92-U -233 MAT number = 2921 92 U 233 SAEL: Eval Jun82 N. Asano, H. Matsunobu, Y. Kikuchi NST 19 1037 Dist-Mar83 Rev1-Nov83 82 08 New evaluation for JENDL 2 by N.Asano, H.Matsunobu (SAEI) and Y.Kikuchi (JAERI). 83-11 Comment was added. MF-1 General Information MT-451 Comments and dictionary MT-452 Nu total Sum of Nu d and Nu p MT 454 Fission vield data Taken from ENDF B IV. MT 455 Nu d Below 4 MeV Nu d 0.007549 4.627E 5 In E MeV Between 4 and 20 MeV Based on the data of Masters et al. 417 and Evans et al. 2. MT 456 Nu p Renormalization was made to 3.756 of Cf 242. Below 1 MeV Nu p 2.486 + 0.121+(E DE), where DE is difference of average fragment kinetic energy between incident and thermal neutron energies. It was taken from data of Boldeman et al. 3. Between 1 and 2.73 MeV Nu p 2.436 + 0.1279 E Between 2.73 and 7.47 MeV Nu p 2.327 + 0.1678-E Above 7.47 MeV Nu p 2.657 0.09689 E MF 2 Resonance Parameters MT 151 a Resolved resonance region (| eV to 100 eV) Resolved resonance parameters for the single-level Breit-Wigner formula based on the data of Nizamuddin and Blons 47. b) Unresolved resonance region (0.1 keV to 30 keV) Resonance parameters were deduced with ASPEP code /5/ so as to reproduce the evaluated cross sections in this energy region. MF-3 Neutron Cross Sections a) Thermal energy region (below 1.0 eV) MT-1 Total Sum of partial cross sections MT-2 Elastic scattering Calculated from resolved resonance parameters by using the effective scattering radius of 9.93 fm. MT=18 Fission

Based on data of Weston et al. 76. Cao et al. 7 Deruytter and Wagemans 87 and Federichny et al. 9. MT=102 Capture Based on the data of Weston et al. 6 . 2200 m/s cross sections and calculated res. integrals 2200 m s Res. Integ. 12,70 b elastic 45.30 b 139 b capture 529.9 b 772 b fission total 587.9 Ь b Resonance Region (from 1 eV to 30 keV) Represented with resolved and unresolved resonance parameters. Background cross section was given for the total and fission cross sections in the resolved resonance region. c Smooth part above 30 keV Total MT 1 Based on the data of Poenitz 10, Green and Mitchell 11/. Foster and Glasgov 12. Between 10 and 50 keV, and above 15 MeV, optical model calcuation was applied by using the following optical potential parameters. V 41.8 0.20(E(0,008(E))2, Ws 6.50 0.15(E, Vso 6.0 (MeV) r0 1.31 . rs:1.36 . rso=1.32 (fm) a0 0.57 . b 0.44 , aso:0.50 (fm) NT 2 Elastic Obtained by subtructing non-elastic scattering cross section from the total cross section. MT 4 and 51 64.91 Inelastic scattering Based on optical and statistical model calculation. Level scheme taken from Ref. 13 was as follows. No. Eterny MeV Soin Parity 0.0 52 E.S. 72 0.04035 1 0.0022 2 92. 3 0.1551 11.2 -4 0.23982 52 5 0.31191 3.2 + 7.2 . 6 0.3208 7 0.34047 52+ 8 9.2 . 0.3537 9 0.397 11.2 -1/2 + 10 0.39649 11 0.41576 3/2 + 7.2 -12 0.5039 13 0.5467 5/2 + 7/2 + 14 0.5971 Above 0.6 MeV, assumed to be overlapped. MT=16,17 (n.2n) and (n.3n) Calculated by Pearlstein's method 14 and normalized to fission spectrum averaged value by Kobayashi (15%) MT=18 Fission Besed on data of Blons (16), Gwin et al. (17), Poenitz (18) and Alkhazov et al. /19/. For cross section shape, based on ratio data of Carlson and Behrens /20/ and Shpak and

3 of Uranium-233

Smirenkin 21 MT 102 Capture Based on statistical and optical model calculations, and normalized to the data of Hopkins and Diven 22. MT 251 Mu bar Based on optical model calculation. MF-4 Angular Distributions of Secondary Neutrons MT 2 Based on optical model calculation. MT 51 64 Assumed to be isotropic in the center of mass system. MT 16.17.18 and 91 Assumed to be isotropic in the lab system. MF 5 Energy Distributions of Secondary Neutrons MT 16.17.91 Evaporation spectrum. MT 18 Maxwellian at temp, of 1.338 MeV estimated from Z 2 A systematics of Smith et al. 23. MT 455 Taken from ENDF B IV. References 1 Master C.F. et al.: Nucl. Sci. Eng., 36, 202 (1969). 2 Evans A.E. et al.: Nucl. Sci. Eng., 50, 80 (1973). 3 Boldeman J.W. et al.: Nucl. Phys., A265, 337 (1976). 4 Nizamuddin S. and Blons J.; Nucl. Sci. Eng., 54, 116 (1974). 5. Kikuchi Y.: to be published. 6 Weston L.W. et al.: Nucl. Sci. Eng., 42, 143 (1970). 7: Cao M.G. et al.: J. Nucl. Energy, 24, 111 (1970). 8 Deruytter A.J. and Wagmans .: Nucl. Sci. Eng., 54, 423 (1974). 9 Pshenichny V.A. et al.: INDC (CCP) 111 U. 23 (1978). 10 Poenitz W.P. et al.: Nucl. Sci. Eng., 68, 358 1978 .. 11 Green L. and Mitchell J.A.: WAPD TM 1073 1973 . 12 Foster D.G. and Glasgow D.W. Phys. Rev., C3, 576 (1971). 13 Lederer D.G. and Shirley V.S.: Table of Isotopes, 7th Ed. 1978 . 14 Pearlstein S.: Nucl. Sci. Eng., 23, 238 (1965). 15 Kobayashi K.: J. Nucl. Sci. Technol., 10, 668 (1973). 16) Blons J.: Nucl. Sci. Eng., 51, 130 (1973). 17 Gwin R. et al.: Nucl. Sci. Eng., 59, 79 (1976). 18: Poenitz W.P.: ANL NDM-36 (1978). 19) Alkhazov I.D. et al.: INDC(CCP)-118/G, 155 (1977). 20: Carlson G.W. and Behrens J.W.: Nucl. Sci. Eng., 66, 205 (1978) 21) Shpak D.L. and Smirenkin G.N.: Sov. J. Nucl. Phys., 21, 363 (1975). 22) Hopkins J.C. and Diven B.C.: Nucl. Sci. Eng., 12, 169 (1962).

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1 of Uranium 234

92-U 234 MAT number 2922

92 U 234 JAERI

- Eval Mar76 T.Asami Dist Mar83 Rev1 Jan84

History

- 76 03 The evaluation was made by T.Asami (JAERI) for JENDL-1 /1 .
- 83-03 JENDL 1 data were adopted for JENDL 2 and extended to 20 MeV.
- 84 01 Resonance formula was changed. MT 5 and comment data were added.
- MF 1 General Information

MT 451 Descriptive data and dictionary MT 452 Number of neutrons per fission Taken from ENDE B IV evaluation.

MF 2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW formula Energy range : 1.0E 5 eV to 215.0 eV. Parameters were adopted from James and Slaughter /2. A bound level at 1.6777 eV was taken from Drake and Nichols 3 to reproduce the thermal cross section.

Calculated 2200 m sec cross sections and resonance integrals. 2200 m sec res. integ.

elastic	14.72 b	
capture	95.44 b	609. b
fission	0.006367 b	6.44 Ъ
tota]	110.2 Б	

MF 3 Neutron Cross Sections

10

0.948

Below 215 eV: Background cross sections of zero. Above 215 eV:

MT 1.2.4.51 63.91.102 Total. Elastic. Inelastic and Capture Calculated with statistical and optical model code CASTHY 4. Potential parameters were taken from Agee and Rosen 5.

v 40.5(0.5)En, Ws 8.2, Vso 7.0 (MeV) 1.32 . rs 1.32, rso 1.32 (fm) r . as - 0.47. aso 0.47 fm a0 0.47 The level scheme used was as follows. Energy MeV Spin-Parity No. 0 + g.s. 0.0 2 . 1 0.044 2 0.144 4 . З 6 · 0.2974 8. 0,499 5 0.790 1 6 η. 0.811 ? S 0.8496 8 2 . 0.8516 9 0.9269 2 +

4 ÷

11	0.965	3 +
12	1.023	3
13	1.046	0.

Levels above 1.06 MeV were assumed to be overlapping. The fission, (n,2n) and (n,3n) cross sections were used as the cross sections of competing processes. The parameters D obs of 12.3 eV and average radiative width of 0.025 eV were adopted to normalize the gamma ray transmission coefficients.

- MT 16.17 (n.2n) and (n.3n) Calculated with Pearlstein's method (6).
 MT 18.19.20.21 Fission The evaluated data by Drake and Nichols (3) were adopted from 215 eV to 2 MeV. Above 2 MeV, the cross section calculated by Jary (7) was adopted.
 Aft 251 Mu bar
 - Calculated with CASTHY 4 .

ME 4 Angular Distributions of Secondary Neutrons

- MT 2 Calculated with CASTHY code 4 .
- MT 51 63 Isotropic in the center of mass system,
- MT 16.17.18.19.20.21.91 Isotropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

MT 16.17.91 Evaporation spectrum.

MT 18.19.20.21 Maxwellian fission spectrum taken from ENDF B IV.

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1 of Uranium 235

92-U 235 MAT number 2923

92 U ~235 SAEL+ 79Knoxyi))e.715 Eval Sep79 H.Matsunobu, A. Asami, Y. Kikuchi Dist Mar83 Rev1 Nov83

History

79 09 Newly evaluated for JENDL-2 by the following evaluaters. A.Asami (High Energy R.I.) Resolved resonance parameters Y.Kikuchi (JAERI) Unresolved resonance parameters H.Matsunobu (SAEI) other quantities

83 11 Comment was added.

MF 1 General Information

MT 451 Comments and dictionary

MT 452 Total number of neutrons per fission Sum of nu p MT 458 and nu d MT 455.

- MT 454 Fission product yield data taken from ENDE B IV.
- MT 455 Delayed neutron data Adopted the evaluated data by Schatz 2.

MT 456 Number of prompt neutrons Evaluated on the basis of the experimental data by Boldeman and Walsh /3/, Soleilhac et al. /4/, Frehaut et al. /5.6/, Meadows and Whalen /7/, Prokhorova et al. /8.9 and Savin et al. 10.

MF 2 Resonance Parameters

MT 151

Resolved resonances : 1.0 - 100 eV
 2g:Gamma n : Simple average of experimental data.
 Gamma g : Weighted average of experimental data.
 Gamma f : Calculated from the averaged fission area.
 Details of the evaluation given in Ref. 11 -.

2 Unresolved resonance parameters : 100 eV - 30 keV The evaluated total, capture and fission cross sections were fitted by adjusting SO, S1 and Gamma-f. Fixed parameters : R= 9.9 fm, Gamma-g- 45 milli-eV, Dobs= 0.567 eV.

2200 m s cross sections and calculated res. integrals. 2200 m/s res. integ.

elastic	17.0 Ь	
fission	584.0 b	279 Ь
capture	96.0 b	153 b
total	697.0 b	

MF=3 Neutron Cross Sections

Below 1.0 eV: Based on the experimental data.

- Between 1.0 and 100 eV: Background data for resonance parameters are given to well reproduce the experimental data.
- Above 100 eV: Data were evaluated as follows. Between 100 ev and 30 keV, the unresolved resonance parameters and the background data were given to reproduce

these cross sections.

MF 1 Total			
Evaluated on the	basis of th	e experimental data by Uttley –	
et al. 12. Boe	et al. 12. Boeckoff et al. 13. Schwartz et al. 14.		
Green et al. 15	and Foster	and Glasgov 16	
MIZ Elastic scat	tering		
Evaluated on the	basis of the	e experimental data by Smith	
17. Smith and	Whaten /18/	and Knitter et al. 19 in the	
energy range fro	11 U.3 LO 2.3	mey. In the remaining energy	
range it was der	ived by subt	ractin' sum of partial cross	
MT 4 51.70 01 251 Tr	alactic cost	toring cross costion and mu ban	
Fuelueted on the	hasis of evi	serimental data and calculation	
with optical and	statistical	models	
with opercut and	Statiotical	models.	
The optical pote	ntial parames	ters were obtained by fitting	
the experimental	data of the	total cross section.	
V 40,90 (0.04 En	MoV	
Ws 6,50 (0.25 Un	MeV	
vso 7.0		mev -	
r0 1.312, rs	s 1.375. ro) 1,320 (fm)	
a 0,490,b	0,454, ac	> 0,470 (fm)	
Statistical model	calculation	with CASTHY code $/20/$.	
Competing proc	esses : fiss	(n, 2n), (n, 3n), (n, 4n),	
Level fluctuat	ion was cons.	idered.	
711	alaan Onan Ba	¢ 01	
ine level sceme t	aken from Re	Conin Domites	
טאן .	Chergy (Kev	opin-rarity 79	
<u>в</u> .з.	0.075	12	
2	13 038	32	
3	46 347	9.2	
Ŭ A	51.697	52.	
5	61.732	72.	
6	103.1	11.2	
7	129.292	5.2 -	
8	150.6	924	
9	170.7	13/2 -	
10	171.378	7/2 +	
11	197.1	11 2 +	
12	225.40	92 +	
13	249.1	15/2 -	
14	291.1		
15	294.7	13/2 +	
10	202.010	0/2 1 17/9	
17	357.2	15.2 -	
10	367.05	7.2 -	
20	368.8	13.2 -	
21	393.184	32	
22	412.3	9.2	
23	426.71	5.2 -	
24	438.5	19/2 -	
25	445.72	7/2 +	
26	474.27	7/2 +	

27	510.0	9	2	1
28	532.0	9	2	•
29	550.4	21	2	
• •		13 A A		

Continuum levels assumed above 580 keV. The level density parameters : Gilbert and Cameron 22.

MI 16.17.37 (n, 2n), (n, 3n), (n, 4n)Evaluated on the basis of the following experimental data and calculation with evaporation model. : Mather et al. 23 n.2nn.3n and n.4n : Veeser and Arthur 24 MT 18 Fission Evaluated on the basis of the following experimental data: 100 eV 10 keV : Perez et al. 25,26 10 keV1 MeV : Szabo and Marguette 27. Poenitz 28.29 . White 30 1 MeV 20 MeV : Szabo and Marguette 27 . Barton et al. 31 . Poenitz 29 . Czirr and Sidnu 32.33 . Cance and Grenier 34 A special care was paid for evaluating the fission cross section of U 235 so that the consistency could be kept between the relative and absolute measurements for the other heavy nuclides. MT 102 Capture Derived from the evaluated alpha value and fission cross section below 1 MeV. Calculated with the statistical model above 1 MeV. Alpha value was evaluated on the basis of the experimental data by Kononov et al. 30 . Dvukhsherstnov et al. /36 . Gwin et al. 37, Bluhm and Yen 38 and Hopkins and Diven 39 MF 4 Angular Distributions of Secondary Neutrons MT 2 Calculated with optical model. MT 51 79 Isotropic in the center of mass system. MT 16.17.18.37.91 Isotropic in the lab system. MF-5 Energy Distributions of Secondary Neutrons MT 16.17.37.91 Evaporation spectrum MT 18,455 Taken from ENDF/B-IV. References 1 : Matsunobu H. et al.: 1979 Knoxville Conf. (NBS-SP-594), p.715 1979 : 2 Schatz B.: KfK-1629 (1973). 3) Boldeman J.W. and Walsh R.L.: J. Nucl. Energy, 24, 191 (1970). 4: Soleilhac M. et al.: J. Nucl. Energy, 23, 257 (1969). 5 Frehaut J. et al.: EANDC 154U, p.67 1973 6 Frehaut J. et al.: 1973 Rochester Symp., Vol.2, p.201 (1973). 7 Meadows J.W. and Whalen J.F.: J. Nucl. Energy, 21, 157 (1967). 8 Prokhorova L.I. and Smirenkin G.N. Sov. J. Nucl. Phys., 7. 579 :1968 .

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1 of Uranium 236

92-U - 236 MAT number = 2924

92 U 236 NAIG Eval Mar79 T.Yoshida Dist Mar83 Revi Dec83 History 79 03 New evaluation for JENDL 2 was made by T.Yoshida (NAIG). 83 03 MF 1 and MF-5 were added by Y.Kikuchi-JAERI . 83 12 MF 1.MT 451 was added. MF | General Information MT 451 Descriptive data and dictionary MT 452 Number of neutrons per fission Taken from ENDF B IV. MF 2 MT 151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 eV to 1.5 keV Res. energies and Gam n for Gam n grater than 0.14Gam g) : Carraro (17 Gam n for Gam n smaller than 0.1+Gam g / : Mewissen /2/ Gam g : Mewissen 2 , when not given average value was taken. $Gam f : Theobald \cdot 3$. - 23.0 milli eV Average Gam g Average Gam f = 0.354 milli eV A negative resonance was introduced to reproduce the 2200m s capture cross section of (5,2+0,3) barns recommend in BNL 325 3rd edition. Calculated 2200 m is cross sections and res. integrals 2200 m sec Res. Integ. 8.337 Б elastic 0.043 b 7.61 b 5.295 b 347. b fission capture total 13.67 b MF-3 Neutron Cross Sections Below 1.5 keV, all background cross sections are zero. Above 1.5 keV, data were evaluated as follows. MT-1.2.4.51-79.91,102.251 Sig-t.Sig-el.Sig-in.Sig-c.Mu-bar Calculated with optical and statistical models. The spherical optical potential parameters /4/ : V=40.8 - 0.05+En, Ws=0.5 + 0.15+En, Vso=7.0 (MeV), r=1.32, rs=1.38, rso=1.32 (fm). a=as=aso=0.47 (fm). Optical and statistical model calculation was made with CASTHY code 5 . Competing processes : fission. (n,2n) and (n,3n)Level fluctuation was considered. The gamma-ray strength function was determined so that the calculated capture cross section reproduced the measured value of 1.05 barns 6 around 10 keV. The level scheme taken from Ref. 77. No. Energy (MeV) J-Parity No. Energy (MeV) J-Parity

.

g3	0.0	0 +	1	0.04524	2 .
2	0.14948	4 ·	3	0.30979	6+
4	0.52225	8 +	5	0.68757	1
6	0.7442	3	7	0.7828	10 +
-8	0.8476	5 -	9	0.91916	0 +
10	0,9581	2 +	11	0.9604	2 +
15	0.9670	1	13	0.9980	2
14	1.0014	3 +	15	1.0020	7
16	1.0356	3 -	17	1.0512	4 +
18	1.0529	4	19	1.0587	4 +
20	1.0661	3 +	21	1.0700	4 -
22	1.0862	12 +	23	1.0938	2 4
24	1.1044	5	25	1.1110	2
26	1.1267	5 ×	27	1.1470	3 ·
28	1.1494	3	29	1.1640	6
Cont	innum level	s assumed	above 1	.17 MeV.	

MT 16.17 n.2n and n.3n

Calculated with the Pearlstein's method 8. MT 18 Fission

- Evaluated on the basis of measured data of U 236 U 235 9.10. To get absolute value Matsunobu's evaluation 2117 for U 235(n.f) was employed.
- MF-4 Angular Distributions of Secondary Neutrons
 - MT 2 Calculated with optical model.
 - MT 51 79 Isotropic in the center of mass system.
 - MT 16 18.91 Isotropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

- MT-16.17.91 Evaporation spectrum.
- MT-18 Maxwellian fission spectrum. Temperature was estimated from Z = 2 A values = 12 .

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92 U -- 238 MAT number -- 2925

92 U. 238 Kyushu U.: Eval Aug79 Y.Kanda, T.Nakagawa, Y.Kikuchi Dist. Mar83 Rev1 Jan84

History

79 08 Evaluation was made by

T.Nakagé a(JAERI) for resonance parameters and background cross sections,

Y.Kikuchi (JAFRI) for unresolved resonance parameters.

Y.Kanda (Kyushu Univ.) for cross sections above resonance region and other quantities.

83 03 MF 5 was revised.

84 01 Comment data were added

MF 1 General Information

MT 451 Descriptive data and dictionary

The following four MTs' data were entirely taken from ENDF B IV evaluation.

MT 452 Number of neutrons per fission

- MT 454 Fission product yields
- MT 455 Delayed neutron data

MT 456 Number of prompt neutrons per fission

MF 2 Resonance parameters

MI 151 Resolved and unresolved resonance parameters

Resolved resonances for MLBW formula : from 1.0E-5 eV to 4 keV. Reference 1 gives an outline of the present evaluation. Resonance energies : Evaluated on the basis of the experimental data by Poortmans et al. 2. Olsen et al. -3.4 and Nakajima 5. Four negative resonances were added to consider interference effects in the low energy region.

Neutron and capture widths : Obtained from resonance areas calculated from various measurements by giving relatively

high weight to the recent data 28.

Sub-threshold fission widths : Determined from the fission resonance areas measured by Diffilippo et al. /9/ for 28 s-wave resonances.

Effective scattering radius : 9.48 fm obtained by averaging the values of Olsen et al. below 2.2 keV.

Unresolved resonances : from 4 keV to 50 keV.

The energy dependent parameters were determined to reproduce cross sections evaluated by Kanda (see the description of MF=3).

2200 m sec cross sections and calculated resonance integrals.

		acc	100 1000	- 13 -
elastic	8.873	b		
capture	2,700	5	279.	b
fission	3.22E S	5	2.05	Ь
total	11.57	b		

MF=3 Neutron Cross Sections

Below 4.0 keV: Background data for resolved resonances. The background cross sections were obtained with the picket fence model. The contributions from missed p wave resonan ces were also taken into account for the capture cross sec tion in the energy region above 1.5 keV. Above 4 keV. Evaluated cross sections were replaced with unresolved reso nance parameters and background cross sections in the energy region between 4 keV and 50 keV. MT 1 Total Evaluated from the following experimental data. Below 500 kev: Uttley et al. 10., Whalen et al. 11. 4.5 MeV: Kopsch et al. 12 0.5 4 5 15 MeV: Foster and Glasgow 13 . 15 20 MeV: Bratenahl et al. 14. Peterson et al. 15. MT 2 Elastic scattering Calculated as total partial cross sections . MT 4 Total inelastic Evaluated by Kanda MT 51 75.9' Inclastic scattering Calculated as follows by using the presently evaluated total inelastic and partial inelastic scattering cross sections of JENDL, L. Sign th level) Signinel. ((branching ratio to i th level) where the branching ratio to the i th level was calculated from JENDL 1 data 16 . The level scheme. No. Energy MeV > Spin Parity g.s. 0.0 0 + 2. 0.0447 1 2 0.148 4 . 3 0.3016 -0.5208 -4 5 0.680 1 6 0.732 3 17 0.790 10 -5 8 0.8389 0.9392 . 21 10 0.968 0 + 1.006 11 i2 1.047 2 + 13 1.076 2 + 12 + 14 1.100 15 1.123 1 ... 2 _ 16 1.150 3 -17 1.190 2 + 18 1.210 4 19 1.246 ---20 1.272 5 ---2 + 21 1.313 2 + 22 1.361 2 + 23 1.401 14 + 24 1.437 25 1 -1.470 Continuum levels were assumed above 1.5 MeV. MT=16, 17 (n, 2n) and (n, 3n)

Calculated with the evaporation model of Segev and Caner /17/. and normalized to the data by Frehaut and Mosinski /18/. MT-18 Fission Evaluated on the basis of the data of Diffilippo et al. /19/, Behrens and Carlson /20/, Nordborg et al. /21/ and Meadows 22.23 . They were renormalized with the U-235 fission cross section evaluated by Matsunobu 24/. MT-102 Capture Determined mainly from the measurements by Poenitz /25/. Panitkin and Sherman (26/, Moxon (27/, Fricke et al. (28/ and Menlove and Poenitz /29/. MT-251 Mulbar Taken from JENDL-1 /16 MF 4 Angular Distributions of Secondary Neutrons S-TM Taken from JENDL 1 16 . MT 51.52 Taken from JENDL 1 16 . MT 53 75 Isotropic in the center of mass system. MT 16.17.18.91 Isotropic in the laboratory system. MF 5 Energy Distributions of Secondary Neutrons MT 16.17.91 Evaporation spectrum. MT 18 Maxwellian fission spectrum estimated from ZH2/A systematics 30% MT 455 Taken from ENDF/B-IV. References 1) Nakagawa T. et al.: JAERI-M 9823 (1981). 2) Poortmans F. et al.: 1976 Lovell conf., 1264. 3) Olsen D.K. et al.: Nucl. Sci. Eng., 62, 479 (1977). 4 Olsen D.K. et al.: Nucl. Sci. Eng., 69, 202 (1979). 5 Nakajiwa Y.: Ann. Nucl. Energy, 7, 25 (1980). 6 Liou H.I. and Chrien R.E.: Nucl. Sci. Eng., 62, 463 (1977). 7 Haste T.J. and Moxon M.C.: 1978 Harvell Conf., 337. 8 Block R.C. et al.: EPRI NP 996 1979 . 9 Diffilippo F.C. et al.: Phys. Rev., C21, 1400 (1980). 10 Uttley C.A. et al.: 1966 Paris Conf., 1, 165 (1967). 11) Whalen J.F. et al.: Nucl. Inst. Meth., 39, 185 (1966). 12) Kopsch D. et al.: 1970 Helsinki Conf., 2, 39 (1970). 13) Foster D.G.Jr. and Glasgow D.W.: Phys. Rev., C3, 576 (1971). 14) Bratenahl A. et al.: Phys. Rev., 110, 927 (1958). 15) Peterson J.M. et al.: Phys. Rev., 120, 520 (1960). 16) Igarasi S. et al.: JAERI 1261 (1979). 17) Segev M. and Caner M.: Ann. Nucl. Energy, 5, 239 (1978).
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93 Np 237 MAT number - 2931 93 Np 237 Kyushu U. + Eval Mar79 N. Wachi, Y. Kanda, Y. Kikuchi Dist Mar83 Rev1 Nov83 History '79-03 New evaluation was made by N.Wachi and Y.Kanda (Kyushu University), and Y.Kikuchi (JAERI). 83 11 Comment was added. MF 1 General Information MT 451 Comments and dictionary MT 452 Number of neutrons per fission Experimental data of Frehaut + 1. MT 455 Delayed neutron data Experimental data of Benedetti + 2 and systematics by Tuttle 3. MF 2, MT 151 Resonance parameters Resolved resonances for SLBW formula : 1.0E 5 130 eV Res. energy, Gam n. Gam g: Weston and Todd /4/. : Plattard + 5% Gam f Average Gam g 40 milli eV. A negative resonance added. Unresolved resonances : 130 eV 30 keV Parameters by Weston and Todd 4 with slight modification Adopted parameters : SO 1.02E 4 . S1 1.888E 4 . D obs 0.45 eV Gam g 40 milli eV. Gam f values determined so that Sig f = 0.009 b. Calculated 2200 m s cross sections and resonance integrals: 2200 m s value Res.Int. total : 208.5 b 27.52 b elastic : 6.26 b fission: 0.01921 b 6.26 b capture: 181.0 b 663 b MF=3 Neutron Cross Sections MT=1.2.4.51-64.91.102.251 Sig-t.Sig-el.Sig-in.Sig-c.Mu-bar Calculated with optical and statistical models. The spherical optical potential parameters : V = 43.55 , Ws = 11.0 , Vso = 7.0(MeV) (fm) (fm). Competing processes : fission. (n.2n) and (n.3n). Level fluctuation considered. The gamma ray strength function determined so that Sig-c = 0.742 b at 200 keV. The level scheme taken from compilation by Ellis /7/. No Energy (MeV) Spin-Parity 0.0 g.s. 5/2+ 0.03320 1 7/2+

0.05954	5/2
0.07580	9/21
0.10296	72
0.13000	11-20
0.15852	9.2
0.2260	11.2
0.26754	3,2
0.281	12
0.305	13 2
0.327	7 2-
0.332	1/2+
0.357	52
0.369	5/2(
	0.05954 0.07580 0.10296 0.13000 0.15852 0.2260 0.26754 0.281 0.305 0.327 0.332 0.357 0.369

Continuum levels assumed above 0.370 MeV. The level density parameters of Gilbert and Cameron 8.

- MT 16.17 n.2n . n.3n Calculated with the evaporation model by Segev + 9..
- MT 18 fission

Evaluated from measured data. Verified by calculating spectrum averaged cross section.

MF 4 Angular Distributions of Secondary Neutrons

MT 2.51 64.91 Calculated with the optical model. MT 16.17.18 Isotropic in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

- MT 16.17.91 Evaporation spectrum.
- MT 18 Estimated from 2++2 A systematics by Smith +/10/ by assuming E CF 252 2.13 MeV.

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93-Np-239 MAT number = 2932

93 Np 239 Kyushu U.+ Eval Mar76 Y.Kanda, JENDL CG Dist Mar83 Rev1 Jan84

History

- 76 03 The evaluation for JENDL 1 was performed by Kanda (Kyushu Univ. and JENDL 1 Compilation Group. Details are given in Ref. 1.
- 83 03 JENDL 1 data were adopted for JENDL 2 and extended to 20 MeV. MF 5 was revised.
- 84 D1 Comment data were added.
- MF 1 General Information
 - MT 451 Descriptive data and dictionary MT 452 Number of neutrons per fission Taken from the Nn 237 data of ENDF B IV.
- MF 2 Resonance Parameters
 - MT 151 No resonance parameters were given.

2200 m sec cross sections and calculated resonance integrals. 2200 m sec res. integ.

elastic	10.50 b		
capture	37.00 b	445.	b
fission	0.0 b	7.06	b
total	47.50 b		

MF 3 Neutron Cross Sections

Below 4.0 eV.

MT 1 Total

Sum of partial cross sections.

- MT 2 Elastic scattering The constant cross section of 10.5 barns was assumed from Sig 4.3.14+0.147(A)+1.3-+2.
- MT 18 Fission
- Assumed to be zero barns.
- MT-102 Capture

The form of 1/v was assumed. The 2200-m/sec cross section was adopted from the experimental data by Stoughton and Halperin 2%.

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Above 4.0 eV.
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MT=1 Total
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Calculated with optical and statistical model code CASTHY /3/. Optical potential parameters were obtained by Ohta and Miyamoto 4/ by using the total cross section of Pu-239. V = 45.87 0.2 en, Wi = 0.06, Ws= 14.1, Vso= 7.3 (MeV) r ~ 1.27 , ri = 1.27, rs=1.302, rso= 1.27 (fm = a0= 0.652), ai 0.315, as= 0.98, aso=0.652 (fm = a0= 0.652)

MT-2 Elastic scattering

- Calculated with CASTHY 3.
- MT:4.51-58.91 Inelastic scattering Calculated with CASTHY 37. The level scheme was adopted from Nucl. Data Sheets Vol.6.

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No.	Energy (MeV)	Spin Parity
g.s.	0.0	5.2 .
1	0.03114	72
2	0.07112	92 /
3	0.07467	5-2
4	0.11766	11.2 +
5	0.1230	72
6	0.17305	9.2
7	0.2414	11 2
8	0.320	13.2
Levels above 4	130 keV were i	assumed to overlapping in the
calculation th	ne capture, f	ission, (n.2n) and (n.3n) cross
sections were	considered a	s competing processes.
MT 16.17 n.2n	and n.3n	
Calculated wit	h Pearlstein	's method 5.
MT 18 Fission		
Estimated from	i the Np 237 i	fission cross section by normaliz-
ing with neutr	on separation	i energies.
MT 102 Capture	·	-1-
Below 100 keV,	the cross se	ection was calculated from
Sig 435	SQRT (En) ba	arns.
Above 100 keV.	tha shape of	the experimental data for Np-237
by Nagle et al	. 6 was add	opted and normalized to 1.4 barns
at 100 keV.	• • • • • • • • • • • •	
MT 251 Mu bar		
Calculated wit	h CASTHY 3	
MF-4 Angular Distr	ibutions of S	Secondary Neutrons
MT-2	Calculated	with CASTHY code /3/.
MT-51-58	Isotropic i	n the center of mass system.
MT 16.17.18.91	Isotropic i	n the laboratory system.
MF 5 Energy Distri	butions of Se	condary Neutrons
MT 16, 17, 91	Evaporation	spectrum
MT. 18	Maxwellian	fission spectrum estimated from
in io	7 2 A sust	amailine 7
	e e nayat	end JCS (
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O/ Mager N.J. et a.		$v_{111} = 0011., E03 (13/1).$

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1 of Plutonium 236

94-Pu-236 MAT number = 2941

94-Pu-236 FBEC Eval Apr79 T.Hojuyama Dist Mar83 Rev1 Nov83

History

79 04 New evaluation was made by T. Hojuyama (FBEC) 17 in the energy range from 1.0E 5 eV to 20 MeV.

83 11 Comment was added.

MF-1 General Information

MT-451 Comment and dictionary

- MT 452 Number of neutrons per fission Nu p and Nu d for thermal neutron based on Manero's semiempirical formula 2. Neutron energy dependence of Nu besed on Howerton's evaluation 3 .
- MF-2 Resonance Parameters
 - MT 151 Resolved resonance (one resonance)

Energy range from 1.0E-5 to 6 eV. Gam-f was so determined that Sig f calculated from the unresolved resonance formula with the Gam f may smoothly connect at 10 keV to Sig-f in the high energy region. Gam g, SO. SI, <D and R were estimated from systematics. EO was so determined that the fission rate calculated with the resonance parameters in a thermal reactor may agree with exp. data >6 .

Resonance Parameters

E0		:	0.445	eV	2 6 ,4
Gam	n	:	0.526	milli eV	from SO and (D*)
Gam	ſ	:	3.55	milli eV	see above
Gam	g	:	41.5	milli eV	4
R		:	9.46	ſm	see above
D		:	6.3	eV	4
S0		:	1.25E	L _i	4.5
S1		:	2.22E	4	5.

Calculated 2200 m/s cross sections and resonance integrals. 2200 m/sec Res. Integ.

	Z2UU m/	sec	Kes.	INU	
elastic	3,376	ib			
fission	65.41	b	101	Ь	
capture	764.6	b	1067	b	
total	833.4	b	-		

MF=3 Neutron Cross Sections

MT= 1 Total cross section

Obtained by optical model calculation. Optical potential parameters taken from Murata's evaluation /7/ except real potential.

----Optical Potential Parameters-----V = 39.5 0.05 En MeV Ws = 6.5-C.15 En MeV : Vso = 7.0(MeV) r0 = rso = 1.32, rs = 1.38 (fm) a = aso = 0.47, b = 0.47 (fm)

MT 2 Elastic scattering cross section Obtained by optical and statistical model calculations. MT-4.51 54.91 Inelastic scattering cross sections Obtained by optical and statistical model calculations, Level scheme taken from N.D.S. 8/ except 4th level of which energy based on Lynn 9. En(keV) Spin Parity No. g.s. 0.0 0 + 1 44.6 2 . 2 145 4 1 3 305 6 + 4 523 8 + Continuum levels assumed above 661 keV. MT-16,17 (n.2n) and (n.3n) cross sections Calculated with statistical model based on Pearlstein 10. MT 18,19.20.21 Fission cross sections Below 10 keV: Calculated from the unresolved resonance formula with the parameters given in the table of resonance parameters. Above 10 keV: Calculated from fission plateau cross sections /7.12/ and Hill Wheeler type barries penetration factor /11/. Fission barrier parameters were taken from Weigmann /13/. MT 102 Capture cross section Calculated by optical and statistical model with Gam g of 41.5 milli eV and D of 6.3 eV. MT:251 Mu bar Calculated with optical model. MF-4 Angular Distribution of Secondary Neutrons MT 2 Based on optical and statistical model calcula tion. MT 51 54 Isotropic in the center of mass system. MT 16 21.91 Isotropic in the laboratory system. MF 5 Energy Distribution of Secondary Neutrons MT 16.17.91 Evaporation spectrum assumed MT=18,19,20,21 Fission spectrum of Maxwellian form adopted. Theta taken from evaluation of Terrel1/14/. References 1) Hojuyama T.: Proc. '79 Fall Meet. of A.E.S.J., Tokai (1979) C43. 2) Manero F. and Konshin V.A.: At. Energy Rev., 10, (1972) 637. 3) Howerton R.J.: Nucl. Sci. Eng., 62, (1977) 438 4) Mughabgab S.F. and Garber D.I.: BNL 325,3rd Ed.,1,(1973). 5) Musgrove A.R.de L.: AAEC/E 277 (1973). 6) Gindler J.E. et al.: Phys. Rev., 115, (1959) 1271. 7) Matsunobu H. et al.: Proc. Int. Conf. on Nuclear Cross Sections for Technology, Knoxville (1979) 715. 8) Schmorak M.R.: Nucl. Data Sheets.20, (1977) 165. 9) Lynn J.E.: The Theory of Neutron Resonance Reactions (1968). Oxford University Press. 10) Pearlstein S.: Nucl. Sci. Eng. 23, (1965)238. 11) Hill D.L. and Wheeler J.A.: Phys. Rev., 89, (1953) 1102.

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94 Pu 238 MAT number 2942

94 Pu 238 PNC Eval Mar79 T.Kavakita Dist Mar83 Rev1 Feb84

History

79 03 New evaluation was made by T.Kawakita (PNC).

- 84 02 Neutron widths of two low energy resonances were corrected. The n.2n⁺ cross section was also corrected. Comment was added.
- MF 1 General Information

MT 451 Descriptive data and dictionary

MT 452 Number of neutrons par fission The thermal value of prompt neutrons was based on experimental data of Jaffey 1 and Nu d was taken from semi empirical formula by Manero 2. The energy dependent term was estimated from Howrton's formula 3. Nu p and Nu d are not given in JENDL 2.

MF 2 Resonance Parameters

MT 151 Resolved resonance parameters for MLBW formula. Energy range is from 1.0E 5 eV to 500 eV. Parameters were taken adopted from the following experimental data. 49 resonances above 10 eV : Silbert 4. 4 resonances below 10 eV : Young 5

Calculated 2200 m s cross sections and resonance integrals 2200 m s Res. Integ.

elastic	27.60	b		
fission	16.55	b	32.4	b
capture	548.8	b	156	b
total	592.9			

MF 3 Neutron Cross Sections

Below 500 eV is the resonance region. Above 500 eV, the cross sections were evaluated as follows.

MT-1 Total Sum of partial cross sections. MT=2,4,51-78.91 Elastic and inelastic scattering Calculated with optical and statistical models.

Optical potential parameters: Real well depth was obtained by mass fitting. Other parameters were taken from Murata's evaluation (6).

V = 41.0 - 0.05 En	(MeV)
Ws ≕ 6.5 ~ 0.15 En	(MeV)
Vso≔ 7.0	(MeV)
a = b = aso= C.47	(វ តា)
r = rso = 1.32	(fm)
rs = 1.38	(fm)

Statistical model calculation with CASTHY code 7%

The level scheme	taken from I	Ref, ∞8	
No.	Energy (kev.)	Spin Parity	
g.s.	0.0	0 +	
1	44.08	2 .	
2	145.98	4 (
3	303.4	6 (
4	514.0	8 +	
5	605.1	1 -	
6	661.4	3	
7	763.2	5	
8	941.5	0.4	
9	962.77	1 .	
10	968.2	2	
11	983.0	2 ·	
12	955.5	2	
1:3	1028,65	2 '	
,4	1069,95	3	
10	1082.57	4	
10	1125.8	4	
17	1174.5	2 +	
10	1202.1	3 ··	
18	1223.6		
20	1204.2	2 +	
ZI C	1310.3	2 1	
22	1420.6	1	
23	1441.3		
64 05	1400.0	<u> </u>	
20	1000.0	0	
20	1090.0	<u> </u>	
15	1021.4	1	
Continuity los		share 1 65 MeV	
The level density	vers assumed	above 1.00 mey.	'amanan O
Mo compating another	parameters o	ng Gilbert and C	ameron 9,
no competing proc	ess vas cons	laerea.	
MT-16-17 on 2n and	in Sn i		
Calculation based	on the Pear'	letain's method	/10/
MT-18 Fission	on the rear	Latern a method	/10/.
Evaluated on the l	hasis of meas	sured data (A/.	/11/ - /17/
MT=102 Capture		Jul Ca Guera / 4/ 1	/ • • / / / / 4
In the low energy	region, the	cross section w	as calculated
from the following	average res	conance paramete	rs.
S0 = 1.27E-4.5	S1 = 2.26F-4	D-obs = 9.5 eV	
gamma width = $($).040 eV, fis	sicn width = $0.$	013 eV.

MI-251 Mu-bar

Calculated with optical model.

MF=4 Angular Distributions of Secondary Neutrons MT=2

Calculated with optical model.

MT=51--78

Isotropic in the center of mass system.

MT=16,17.18,91

Isotropic in the laboratory system.

- MF 5 Energy Distributions of Secondary Neutrons MT 16.17.91 Evaporation spectrum was assumed. MT 18
 - Maxwellian type fission spectrum. Temperature was estimated from $Z(\cdot, Z, A)$ systematics by Smith et al. 18.

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1 of Plutonium 239

94 Pu 239 MAT number - 2943

94 Pu 239 NAIG Eval Aug79 M.Kawai .T.Yoshida .Y.Kikuchi Dist Mar83 Rev1 Jan84

History

79 08 Evaluation was made by

M.Kawai (NAIG) for cross sections above resonance region and other quantities.

T.Yoshida NAIG for resonance parameters and background cross sections.

Y.Kikuchi JAERI for unresolved resonance parameters.

84 0) Small modification. Comment data were added.

MF 1 General Information

MT 451 Descriptive data and dictionary

- MT 452 Number of neutrons per fission
 - Sum of nu p MT 456 and nu d MT 455.
- MT 454 Fission product yields ENDF B IV data were adopted.
- MT 455 Delayed neutron data

Evaluated data by Mancro and Konshin (1) were adopted.

MT 456 Number of prompt neutrons per fission Same as JENDL 1 2. Least squares fit to experimental data mainly Refs. 3 5 7.

MF 2 Resonance Parameters

MT 151 Resolved and unresolved resonance parameters

Resolved resonances for MLBW formula : from 1.0 to 598 eV. Parameters were evaluated by Yoshida 6 modifying parameters recommended by Ribon 7. Background cross sections were added so as to reproduce the measured fission cross section by Derrien et al. 8 and capture cross section by Gwin et al. 9.10

Unresolved resonances : from 598 eV to 30 keV.

The energy dependent SO. S1 and fission width were determined by Kikuchi so as to reproduce the total. capture and fission cross sections evaluated by Kawai. Fixed parameters were R-9.054 fm. capture width-41.5 milli eV.

2200 m sec cross sections and calculated resonance integrals. 2200 m/sec res. integ.

elastic	8.000	Ъ	
capture	270.2	b	195 b
fission	741.7	b	302 b
total	1019.9	b	

MF-3 Neutron Cross Sections

Below 1.0 eV.

The elastic and fission cross sections were taken from ENDF B-IV. The capture cross section was also taken from ENDF/B-IV and small modification was made above 0.6 eV. The total cross section is sum of these three. Between 1.0 eV and 30 keV.
fm.

Background cross sections for resonance parameters were given.

Above 598 eV.

Evaluated cross sections were replaced with unresolved resonance parameters and background cross sections in the energy region between 598 eV and 30 keV.

MT 1 Total

Calculated with optical and statistical model code CASTHY 16. Sphrical optical potential parameters were obtained by fitting the experimental data for total cross sections of Refs. 11-15.

٧	40.72 0.05 En	(MeV)
Ws	6.78 + 0.29 En	(MeV
Vso	7.0	MeV
r	rso 1.32	ſm
rs	1.357 derivative	• Woods Saxon type
a	aso b 0.47	f m

MT 2 Elastic scattering

Calculated as Total Partial cross sections ...

MT 4.51 78.91 Inelastic scattering

Calculated with CASTHY 16. The fission. (n.2n), (n.3n) and n.4n were considered as competing processes. Level fluctuation and interference effects were also taken into account. Direct inelastic components were added on the basis of coupled channel calculation carried out out by Prince 17. The level scheme shown below is taken from Ref. /18/.

TEAGT PETICING SHOW	I DOTOM TO FO
Energy (keV)	Spin Parity
0.0	12+
7.86	32+
57.273	52 ×
75.701	72+
163.75	9 Z +
194.	11.2 -
265.46	52 -
317.	13 2 🕔
330.13	7.2 .
360.	15.2
387.41	92+
391.6	7/2
428.	(13 /2 +)
435.	92 -
452.	(11/2)
462.	11,2 +
469.8	1/2 -
482.	(13/2 -)
488.	11/2 -
492.1	3/2 -
505.5	52-
511,83	72+
538.	72+
555.	72
565.	92+
583.	92-
620.	15/2 -
634.	11/2 +
659.	11/2 -
	Energy (keV) 0,0 7,86 57,273 75,701 163,75 194, 265,46 317, 330,13 360, 387,41 391,6 428, 435, 452, 462, 462, 462, 462, 462, 462, 555,5 511,83 538, 555,5 511,83 538, 565,5 583,6 620, 634, 659,

Continuum levels were assumed above 670 keV. The values of spin in parenthesis were assumed from those of neighbouring nuclide. Level denbsity parameters were taken from Murata's evaluation 7287.

	a	spin cut	E pair	CO	E joint
	(1.:MeV)	off factor	(MeV)		(MeV)
Pu 239	26,93	17.6156	0.61	4130.0	3.61
Pu 240	26.53	17,7964	1.64	5480.0	3.53
MT-16.17.37	(n.2n). (n.3	n) and (n.4n	0		

Calculated with Perlstein's method 28 by using the neutron emission cross sections which derived by subtracting the presently evaluated fission and capture cross sections from compound nucleus formation cross section obtained with CASTHY. The results agree with the experimental values by Mather et al. 27

MT 18 Fission

Evaluated on the basis of the following measured data. Absorute measurements

Kari and Cierjacks 19, Cance and Grenier 20, Gwin et al. 10 and Gayther 21.

Relative measurments to the U 235 fission cross section: Carlson and Behrens 22°, Meadows 23°, Kari and Cierjacks 19°, Cance and Grenier 20° and Fursov et al. 24

The relative data were converted into cross sections by multiplying the U 235 fission cross section in JENDL 2 /25/. MT 102 Capture

The cross section in the energy range below 1 MeV was derived as a product of the evaluated fission cross section and alpha values. The alpha values are the same as JENDL-1 $/2^{-1}$. Above 1 MeV, results of statistical model calculation with CASTHY 16 was applied.

Normalization parameters for CASTHY calculation:

Gamma g 43.0 milli eV.

D obs 2.36 eV.

Capture cross section 284 mb at 100 keV.

MT 251 Mu bar

Calculated with CASTHY 16 .

MF-4 Angular Distributions of Secondary Neutrons

MT=2	Calculated with CASTHY code /16/.
MT=51-78	Isotropic in the center-of-mass system.
MT=16,17,18,37,91	Isotropic in the laboratory system.

MF=5 Energy Distributions of Secondary Neutrons

MT=16,17,37,91 Evaporation spectrum.

MT=18⁻ Maxwellian fission spectrum estimated from Z+42/A systematics 29^{-/}. MT=455 Takenfrom ENDF B-IV.

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94-Pu-240 MAT number = 2944

94 Pu 240 NAIG+ Eval Mar79 T.Murata, A.Zukeran, Y.Kikuchi Dist Mar83 Rev1 Nov83 History 79 03 New evaluation was made by the following evaluaters. A.Zukeran (Hitachi) Resolved resonance parameters Y.Kikuchi (JAERI) Unresolved resonance parameters T.Murata (NAIG) Other quantities 83-03 MF-5 was added. 83-11 Comment was added. MF-1 General Information MT-451 Comments and dictionary MT 452 Number of neutrons per fission Sum of MT 455 delayed neutrons and MT 456 prompt neutrons). MT-455 Delayed neutron data Taken from ENDF/B-IV. MT=456 Number of prompt neutrons Linear least-squares fitting to the experimental data of Freaut et al. /1/ renormalized to Cf-252 nu-p=3.756. MF 2 Resonance Parameters MT 151 Resolved and unresolved resonance parameters 1 Resolved resonances for MLBW formula Energy range : 1.0E 4 to 4 keV. The first resonance at 1.056 eV was evaluated on the basis of Pattenden and Rainey 2. The neutron and capture widths were based on the experimental data by Hockenbury et al. /3/in the energy range from 20 to 500 eV, and Kolar and Boeckhoff 4 from 500 eV to 4 keV. The average capture width of 29.5 milli eV was assumed for the resonances whose J values were unknown. The sub threshold fission widths were taken from the data by Auchampaugh and Weston /5/. A negative resonance was adopted from ENDF B-IV. Details are given in Ref. 76/. 2) Unresolved resonances Energy range : 4 to 40 keV. Energy dependent parameters were determined to reproduce the presently evaluated cross sections in this energy region. Calculated 2200-m/sec cross sections and res. integerals. 2200-m/sec res. integ. 1.509 b elastic 1.509 b -0.06761b 10.1 b 288.5 b 8450. b fission capture 290. i b total MF=3 Neutron Cross Sections Below 4 keV: Background cross sections are given.

Above 4 keV: Evaluated as follows. In the energy range from 4 to 40 keV, the cross sections are represented with the unresolved resonance parameters, and the back-

ground cross sections are given in MF-3. MT=1 Total Pptical model calculation with the following parameters. which were determined to reproduce the experimental data of Smith et al. /7/, strength functions and calculated cross sections from resoleved resonance parameters. The spherical optical potential parameters V = 40.6 0.05te , Ws = 6.5 + 0.15te (MeV) Vso= 7.0 (MeV) , rs 1.38 r = rso =1.32 (fm) a = as = aso = 0.47(ព្រា) Optical and statistical model calculations were made with CASTHY code 8 . MT-2Elastic scattering Obtained by subtracting the other cross sections from total cross section below 4 MeV. Above 4 MeV. optical model calculation. MT-4 Total inelastic scattering Sum of partial inelastic scattering cross sections (MT=5) to MT-9!). MT-51 79, 91 Partial inelastic scattering CASTHY code calculation for almost levels. For some levels, for which Smith's experimental data 77 were available. renomalization was performed (for 1st, 2nd, 3rd, 5th and 9 to 11th levels). Level scheme (taken mainly from Ref. 9_{2}) No. Energy (MeV) Spin Parity 0.0 0 + g.s. 2 . 1 0.04282 0.1417 Δ. 3 0.2947 6 · 4 0.4976 8 . 5 0.5974 1 6 3 0.6489 7 0.7425 5 -8 0.7514 10 + 9 0.8607 0 + 10 0.9003 2 +0.9381 11 1 ---12 0.9589 2 -13 0.9926 4 + 3 ---14 1.002 15 1.031 3 + 1.038 4 -16 17 1.076 4 + 18 1.090 0 ÷ 19 1.116 5 -20 1.138 2 + 6 --21 1.161 3 + 22 1.178 23 1.180 2 + 1.223 2 + 24 25 1.232 4 +

26	1,241	2 -
27	1.262	3 +
28	1.282	3 -
29	1.309	5 -

Levels above 1.309 MeV were assumed to be continuum. Level density parameters were determined to reproduce the resonance-level spacing and level-scheme staircase.

MT 18 Fission

Below 9 keV : Shape of fission cross section based on the experimental data of Byers et al. 11 and the calculated cross section with resonance parameters of Migneco and Theobald 12. Then normalized to the value of higher energy region. Above 9 keV : Fission ratios to U 235 fission based on the

experimental data of Behrens et al. /13/ and Wisshak and Kaeppeler (14/ were multiplied by U-235 fission cross section of JENDL-2 /15/.

MT 102 Capture

Below 0.35 MeV : Based on the experimental data of Hockenbury et al. $\langle 3 \rangle$. Weston and Todd $\langle 16 \rangle$ and the ratio data of Wisshak and Kaeppeler $\langle 17 \rangle$ with the capture cross section of Au-197 $\langle 18 \rangle$. As a guide line, the result of CASTHY calculation was normalized to the low energy region. direct and collective capture were included in high energy region using the value for U-238 given by Kitazawa et al. 19.

MT-251 Mu bar

The same as JENDL 1 20 .

MF=4Angular Distributions of Secondary NeutronsMT=2Taken from JENDL-1 /20/.MT=16,17,18,37,91Isotropic in the laboratory system.MT=51-79Isotropic in the center-of-mass system.

MF=5Energy Distributions of Secondary NeutronsMT=16.17.37.91Evaporation spectrumMT=18Fission spectrum. Temperature was estimated
from Z+>2/A systematics by Smith et al. /21/.MT=455Taken from ENDF/B-IV.

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94-Pu-241 MAT number = 2945

94-Pu-241 JAERI Eval-Oct79 Y.Kikuchi.N.Sekine Dist-Dec79 Rev1-Nov83

History

- 79-10 New evaluation was made by Y.Kikuchi (JAERI) and N.Sekine (HEC). Data of JENDL-1 /1/ were superseded.
- 79-12 Files 2, 3 and 4 were released as JENDL-28 /2/.
- 83-03 Files 1 and 5 were added.
- 83-11 Comment was added.

MF=1 General Information

- MT=451 Comment and dictionary
- MT-452 Number of neutrons per fission

Sum of Nu p (MT-456) and Nu d (MT=455).

MT-454 Fission yield data

No evaluation done. Data of ENDF/B-IV were adopted. MT=455 Delayed neutron data

- Data of Benedetti + /3/
- MT=456 Number of prompt neutrons per fission Data of Boldeman and Frehaut /4/ for thermal fission Nu-p(Cf-252 spontaneous fission) = 3.753 assumed. Energy dependence : Frehaut + /5/

MF-2, MT-151 Resonance Parameters

Resolved resonances : 1 100 eV

JENDL-1 data /1/ modified for better fit to experiments. A negative resonance addedd. Background cross section applied for fission and capture.

Unresolved resonances : 100 eV 30 keV Obtained by fitting evaluated Sig-f and Sig-c. Energy dependent parameters : So, S1 and Gam-f. Fixed parameters : R-9.8 fm , Gam-g = 0.040 eV, D-obs = 0.85 eV

2200-m/sec cross sections and calculated resonance integrals. 2200 m/sec Res. Integ.

elastic	10.23	b	
fission	1015.	b	590 b
capture	363.0	ь	187 b
total	1398.2	b	-

MF=3 Neutron Cross Sections
Point-wise data below 1 eV down to 1.0E-5 eV
Sig-t : on the basis of the data of Smith + /6/
Sig-f : on the basis of the data of Wagemans + /7/
Sig-e : calculated from resonace parameters
Sig-c : Sig-t - /Sig-f + Sig-e;
2200 m/s values :
Sig-t = 1398.2 b , Sig-f = 1015 b , Sig-c = 363 b.

Background cross sections for resolved resonances (1 - 100 eV).

No background cross sections for unresolved resonances.	
Above 30 keV, smooth cross sections given as follows.	
$\begin{array}{rrrr} MT \approx 1.2, 4, 51 \approx 61.91.251 & : Sig-t, Sig-e, Sig-in, mu-bar\\ Calculated with optical and statistical models.\\ Optical potential parameters obtained from systematics /8 \\ V \approx 40.25 \approx 0.05 \pm En , Ws \approx 6.5 , Vso \approx 7.0 & (MeV)\\ r \approx rso \approx 1.32 , rs \approx 1.38 & (fm)\\ a \approx b \approx aso \approx 0.47 & (fm)\\ Statistical model calculation with CASTHY code /9/.\\ Competing processes : fission, (n,2n), (n,3n), (n,4n).\\ Level fluctuation considered.\\ The level scheme taken from Ref. /10/.\\ No & Energy(keV) & Spin-Parity \\ \end{array}$	/
g.s. $0 - 5/2 + 1 - 1 - 2/2 + 1 - 2/2 - 1 - 2/2 - 1 - 2/2 - 1 - 2/2 - 1 - 2/2$	
1 + 41.8 + 7/2 + 2 + 9/10 + 9/2 +	
3 161.5 1.2 +	
4 170.8 3/2 +	
5 223.1 5/2 +	
6 230.0 9/2 + 7 242 7 7.0 -	
8 300 11/2 +	
9 335 9/2 +	
10 368 13/2 +	
Continuum levels assumed above 490 keV. The level density parameters : Gilbert and Cameron /11/.	
$\begin{array}{llllllllllllllllllllllllllllllllllll$	
MT-18 Fission Simultaneous evaluation with U-235,U-238,Pu-240,Pu-241 /8/ mainly based on the data of Carlson +/12/. Kaeppeler+/13/. Fursov+ 14/ and Szabo+/15,16/.	,
MT=102 Capture Based on the data of Alpha by Weston+ /17/ up to 250 keV. Calculated with the statistical model above 250 keV. The gamm-ray strength function was determined so that Sig-c =269 mb at 250 keV.	
MF=4Angular Distributions of Secondary NeutronsMT=2: Calculated with the optical model.MT=51-61: Isotropic in the center-of-mass system.MT=16,17,18,37,91: Isotropic in the laboratory system.	
MF=5 Energy Distributions of Secondary Neutrons MT=16,17,18,37,91 : Evaporation spectrum. MT=18 : Maxwellian firsion spectrum. Temperature estimated from Z**2/A values. MT=455 : Beta-i from the data of Benedetti+ /3/. ENDF/B-IV data for delayed neutron spectrum	

No background cross sections for unresolved resonances. Above 30 keV, smooth cross sections given as follows. MT-1.2.4.51-61.91.251 : Sig-t.Sig-e.Sig-in.mu-bar Calculated with optical and statistical models. Optical potential parameters obtained from systematics /8/ $V=40.25=0.05\pm En$, Ws ≈6.5 , Vso ≈7.0 (MeV) r rso 1.32 . rs = 1.38 (**fm**) a = b = aso = 0.47 (fm) Statistical model calculation with CASTHY code /9/. Competing processes : fission, (n.2n), (n.3n), (n.4n). Level fluctuation considered. The level scheme taken from Ref. /10/. No Energy (keV) Spin-Parity 0 52+ g.s. 7.2 + 1 41.8 2 94.0 92 + З 161.5 1.2 (4 170.8 32+ 5 223.1 5/2 + 6 230.0 9/2 + 7 7/2 + 242.7 8 300 11/2 + 9 335 9/2 + 10 368 13/2 + 11 445 11/2 -Continuum levels assumed above 490 keV. The level density parameters : Gilbert and Cameron /11/. MT=16.17.37 (n,2n), (n,3n), (n,4n)Calculated with evaporation model. MT 18 Fission Simultaneous evaluation with U-235.U 238.Pu-240.Pu-241 /8/ mainly based on the data of Carlson + 12/, Kaeppeler+/13/, Fursov 14/ and Szabo+/15.16/ MT=102 Capture Based on the data of Alpha by Weston+ /17/ up to 250 keV. Calculated with the statistical model above 250 keV. The gamm-ray strength function was determined so that Sig-c =269 mb at 250 keV. MF=4 Angular Distributions of Secondary Neutrons MT=2 : Calculated with the optical model. MT=51-61 : Isotropic in the center-of-mass system. MT=16,17,18,37,91 : Isotropic in the laboratory system. MF=5 Energy Distributions of Secondary Neutrons MT=16,17,18.37.91 : Evaporation spectrum. : Maxwellian fission spectrum. MT=18 Temperature estimated from Z**2/A values. MT=455 : Beta-i from the data of Benedetti+ /3/. ENDF/B-IV data for delayed neutron spectrum.

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94-Pu-242 MAT number = 2946

Chierry and an annual state of the state

94 - Pu-242 NAIG Eval-Mar81 M.Kawai and T.Murata Dist Mar83 Rev1 Nov83 History 81-03 New evaluation was made by M. Kawai (NAIG) for resonance parameters /1/ and T. Murata (NAIG) for smooth cross sections. 83-03 MF-5 was added. 83.11 Q values and threshold energies and MF-4 were modified. MF 1 General Information MT 451 Descriptive data and distionary MF 452 Total number of neutrons per fission Taken from ENDF B 1V 2 . MF-2 Resonance Parameters MT 151 Resolved resonances for MLBW formula : 1.0E 5 eV - 1290 eV. Res. energies : BNL-325 3rd edition /3/ Gam n. Gam g : Poortmans et al. /4/.Auchampaugh et al. /5/ Gam f : Deduced from fission areas given by Berman et al. -6/ and Auchampaugh et al. 5. : 9.6 fm 3 R scat Average Gam g : 24.2 milli eV Background cross section of 0.67 barns was added to elastic scattering cross section so as to reproduce the measured data of the elastic 7 and total cross sections /8, 9/. Calculated 2200 m s cross sections and res. integrals 2200 m s res. integ. 8.111 b elastic capture 18.42 b 1117 b 0.1212b fission 6.35 b 26.65 h total MF-3 Neutron Cross Sections MT-1 Total + Below 6 keV : Experimental data of Young and Reeder /8/ were averaged over some keV intervals. + Above 6 keV : The results of optical model calculation with the following parameters were adjusted in the energy region from 6 keV to 100 keV to connect smoothly with the total cross section below 6 keV. V = 40.1 - 0.05ie, Ws = 6.5 + 0.15ie (MeV) (MeV) Vso≈ 7.0 r = rso = 1.32 , rs = 1.38(fm) a = as = aso = 0.47(fm) Optical and statistical model calculations were made with CASTHY code / 10/. MT=2 Elastic scattering * Below inel. threshold : Obtained by subtracting the nonelastic scattering from the total cross section.

- + Above the threshold : Sum of the shape elastic scattering cross section calculated with optical model and the compound elastic scattering cross section obtained using the other cross sections and results of statistical model calculation.
- MT-4 Total inelastic scattering Obtained by subtracting the others from the total cross section.
- MT-51-67.91 Partial and continuum inelastic scattering The total inelastic scattering (MT-4) was multiplied by branching ratio to each level calculated with CASTHY code.

Level scheme (taken mainly from Ref. (11.))

No.	Energy (MeV)	Spin Parity
g. s .	0.0	0 +
1	0.0445	2 +
2	0.1472	4
3	0.3059	6 ,
4	0.5176	8 +
5	0,7787	10 +
6	0.7803	1
7	0.8323	3
8	0,865	1
9	0.927	5
10	0.956	0 +
11	0.995	2 +
12	1.019	3
13	1.064	4
14	1.087	12 +
15	1,102	2 +
16	1.122	5
17	1.152	2

Levels above 1.152 MeV were assumed to be continuum.

Level density parameters were determined to reproduce the resonance level spacing and level scheme staircase. $MT=16,17,37=(n,2n_{\rm cl},-n,3n_{\rm cl})$ and $(n,4n_{\rm cl})$

Neutron emission cross section obtained by subtracting the elastic (MT-2), fission (MT-18) and capture (MT-102) from the total cross section (MT-1) were multiplied by branching ratio to reaction channel calculated with Pearlstein's method /12/.

- * Below 100 keV : Shape of fission cross section was determained on the basis of fission area data of Auchampaugh et al. /13/. Then normalized to the value of higher energy region.
- * Above 100 keV : Fission ratios to U-235 fission cross section were obtained from the experimental data of Behrens et al. /14/ and multiplied by the U-235 fission cross section of JENDL-2 /15/.

- * Energy region from 6 keV to 21C keV : Determined from the experimental data of Hokenbury et al. /16/ and Wisshak and Kaeppeler /17/.
- * Other energy region : Calculated results with CASTHY code

MT=18 Fission

MT=102 Capture

were normalized to the capture cross section in the region of 6 to 210 keV. Direct and collective captures were included in high energy region using the value for U-238 given by Kitazawa et al. >18/.

MT=251 Mu=bar

Calculated with optical model.

MF-4 Angular Distributions of Secondary Neutrons

MT-2 Taken from Pu-240 data /19/ calculated with optical model.

MT-51-67 Assumed to be isotropic in the center-of-mass system. MT-16.17.18.37.91 Isotropic in the lab system.

MF=5 Energy Distributions of Secondary Neutrons

MT=16,17.37,91

MT=18

Evalporation spectrum. Fission spectrum. Temperature was estimated Z++2 A systematics by Smith et al. 201.

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95-Am-241 MAT number = 2951

95-Am 241 JAERI Eval Mar82 Y.Kikuchi JAERI - M82-096 Dist Mar83 Rev1-Nov83 History 82-03 Complete reevaluation for JENDL-2 was made by Y.Kikuchi (JAERI). Details are given in Ref. 17. 83-11 Comment was added. MF | General Information MT 451 Comment and dictionary MT-452 Number of neutrons per fission Sum of Nu p MT 456 and Nu d MT 455 . MT 454 Fission product vield data Taken from ENDF B IV. and renormalized to 2.0. MT 455 Delayed neutron data Estimated with semi empirical formula by Tuttle /2/. MT: 456 Number of prompt neutrons Experimental data of Jaffey and Lerner /3/.MF-2.MT-151 Resonance parameters Resolved resonances for MLBW formula : 1.0E 5 - 150 eV Data of Derrien and Lucas /4/. Same as JENDL-1 /5/. 5 negative resonances added. Unresolved resonances : 150 eV - 30 keV The evaluated Sig-t. Sig-c and Sig f were fitted by adjusting SO. SI and Cam f. Fixed parameters: R 9.37 fm . Gam-g 43.77 MeV . Dobs 0 432 eV Calculated 2200 m s cross sections and resonance integrals 2200 m s value Res. Int. elastic 11.26 b 600.4 b 1299 b 3.018 b 14.7 b capture fission total 614.7 b MF=3 Neutron Cross Sections MT=1,2,4,51-66.91,251 Sig-t,Sig-el,Sig-in,Mu-bar Calculated with optical and statistical models. Optical potential parameters were obtained by fitting the data of Phillips and Howe /6/ : (MeV) V = 43.4 - 0.107 + En(MeV) Ws= 6.95 - 0.339+En + 0.0531+En+42 , Vso = 7.0 Wv≕ O (MeV) r = rso = 1.282, rs = 1.29(**f**m) a = aso = 0.60, b = 0.5(fm) Statistical model calculation with CASTHY code /7/. Competing processes : fission, (n, 2n), (n, 3n), (n, 4n). Level fluctuation considered. The level scheme taken from Ref. /8/ energy(keV) spin-parity No

g.s.	0	52
1	41.2	7/2
2	93.6	9.2
3	158.0	11-2
4	205.9	5/2 +
5	234.0	7/2 (
6	271.0	9/2 (
7	319.0	11 2 +
8	375.0	13/2 +
9	471.8	3/2 -
10	504.5	5/2 -
11	549.0	7./2
12	623.1	1/2 +
13	636.9	3 2 -
14	652.1	12
15	653.2	32+
16	670.2	32+

Continuum levels assumed above 732 keV.

The level density parameters: Gilbert and Cameron /9/.

MT = 16.17.37 (n,2n), (n,3n), (n,4n) Calculated with evaporation model.

Capture

MT 18 Fission

Evaluated on the basis of the following measured data : Knitter and Budtz-Jorgensen /10/ : 150 eV - 10 keV Wisshak and Kaeppeler /11/ : 10 - 300 keV Behrens and Browne /12/ : 300 keV - 20 MeV High sub-threshold cross-section values of Seeger + /13/ were abandoned.

MT 102

Evaluated on the basis of the measured data of Gayther and Thomas 14 up to 350 keV. Calculated with the statistical model above 350 keV. The gamma ray strength function was determined so that Sig-c 830 mb at 350 keV.

- MF=4 Angular Distributions of Secondary Neutrons MT=2.51-66.91 Calculated with optical model. MT=16.17.18.37 Isotropic in the lab system.
- MF=5 Energy Distributions of Secondary Neutrons MT=16,17,37,91 Evaporation spectrum. MT≔18 Maxwellian fission spectrum. Temperature was estimated from Z++2/A values /14/.

- 1) Kikuchi Y. : JAERI-M82-096 (1982).
- 2) Tuttle R.J. : INDC(NDS)-107/G+Special , p.29 (1979).
- 3) Jaffey A.H. and Lerner J.L. : Nucl. Phys., A145,1 (1970).
- 4) Derrien H. and Lucas B. : 1975 Washington Conf. p.637,NBS-Sp-425 (1975).
- 5) Nakagawa T. and Igarasi S. : JAERI-M6636 (1976). 6) Phillips T.W. and Howe R.E. : Nucl.Sci.Eng., 69,375 (1979).
- 7) Igarasi S. : J.Nucl.Sci.Technol., 12,67 (1975).

- 8) Lederer C.M. and Shirley V.S. : Table of Isotopes , 7th Ed. 9) Gilbert A. and Cameron A.G.W. : Can.J.Phys.,43,1446 (1965).
- 10) Knitter H.H. and Budtz-Jorgensen C. : Atomkernenergie.33.205 (1979).
- 11) Wisshak K. and Kaeppeler F. : Nucl.Sci.Eng., 76, 148 (1980).
- 12) Behrens J.W. and Browne J.C. : Nucl.Sci.Eng., 77,444 (1981).
- 13) Seeger P.A. et al. : Nucl.Phys. A96,605 (1967). 14) Smith A.B. et al. : ANL/NDM-50 (1979).

1 of Americium-242

95-Am-242 MAT number = 2952

95-Am-242 JAERI Eval-Mar80 T.Nakagawa, S.Igarasi JAERI-M 8903 (1980) Dist-Mar83 Rev1 Nov83 History 80-03 New evaluation was made by T.Nakagawa and S.Igarasi (JAERI). Details are given in Ref. 11. 83-11 Comment was added. MF=1 General Information MT=451 Comment and dictionary MT 452 Number of neutrons per fission Sum of prompt and delayed neutrons. MT-455 Delayed neutron data Estimated from Tuttle's semi empirical formula 2. MT 456 Number of prompt neutrons per fission Semi empirical formula by Howerton 3 Nu = 3.268 + 0.172 + E(MeV). MF=2 Resonance Parameters MT 151 No resonance parameters 2200m s cross sections and calculated resonance integrals. 2200 m/sec Res. Integ. 5500.0 b 391 Ь capture 2100.0 b fission 1260 b elastic 11.44 b total 7611.44 b MF 3 Neutron Cross Sections MT-i.2.4.51-72.91.102.251 Sig-t.Sig-el.Sig-in.Sig-c.Mu-bar Below 0,225 eV: 1 v form was assumed for fission and capture cross sections. Effective scattering radius of 9.54 fm was used for elastic scattering cross-section calculation. Above 0.225 eV: Optical and statistical models were used. The spherical optical potential parameters (MeV, fm) : V = 42.0 - 0.107 + E, r = 1.282, a = 0.6Ws = 9.0 - 0.3394E + 0.05314E442, r= 1.29, a= 0.5 Vso = 7.0, r = 1.282, a = 0.6Statistical model calculation with CASTHY code /4/. Competing processes : fission, (n.2n) and (n,3n). Level fluctuation considered. Gam-g = 0.05 eV and D = 0.45 eV used for capture cross section calculation The level scheme taken from the compilation by Ellis and Haese 5. Energy (MeV Spin Parity Ne. 0.0 g.s. 1 0 -3 -5 -0.044 1 0.049 0.049 2 3 2 -4 0.074

5	0.113	6	a
6	0.148	4	~
7	0.148	5	
8	0.190	7	
9	0.242	3	
10	0.263	6	
11	0.263	7	
12	0.288	4	
13	0.288	2	
14	0.325	3	
15	0.341	5	
16	0.372	4	
17	0.410	6	
18	0.430	5	•
19	0.488	7	
20	0.500	6	
21	0.581	7	
22	0.679	8	

Overlapping levels are assumed above 0.681 MeV. The level density parameters of Gilbert and Cameron /6/.

MT (16.17 (n.2n) and (n.3n) cross sections Calculated with the evaporation model by Pearlstein /7/. MT=18 Fission cross section

The empirical formula used for the Am-242m data was applied by shifting the energy origin to -49 keV.

M⁺ 4 Angular Distributions of Secondary Neutrons

MI-2 Legendre coefficients are given by the optical and statistical model calculations.

- MT-16.17.18.91 Isotropic distributions in the center-of-mass system.
- MT 51 72 Isotropic distributions in the laboratory system.

MF-5 Energy Distributions of Secondary Neutrons

- MT-16.17.91 Evaporation spectrum

- 1) T. Nakagawa and S. Igarasi : JAERI-11 8903 (1980), in Japanese.
- 2) R.J. Tuttle : INDC(NDS)-107/G+Special, 29 (1979).
- 3) R.J. Howerton : Nucl. Sci. Eng., 62, 438 (1977).
- 4) S. Igarasi : J. Nucl. Sci. Technol., 12, 67(1975).
- 5) Y.A. Ellis and R.L. Haese : Nucl. Data Sheets 21, 615 (1977).
- 6) A. Gilbert and A.G.W. Cameron : Can. J. Phys., 43, 1446 (1965).
- 7) S. Pearlstein : Nucl. Sci. Eng., 23, 238 (1965).
- 8) A.B. Smith et al. : ANL/NDM-50 (1979).

1 of Americium-242m

95-Am-242m MAT number = 2953

95 Am 242m JAFRI Eval-Mar80 T.Nakagawa.S.Igarasi JAERI-M 8903 (1980) Dist-Mar83 Rev1 Feb84 History 80-03 New evaluation was mode by T.Nakagawa and S.Igarasi (JAERI), Details are given in Ref. /1/.83-11 Comment was added. 84-02 Cross sections were corrected around 3.5 eV. MF-1 General Information MT-451 Comment and dictionary MT=452 Number of neutrons per fission Sum of prompt and delayed neutrons. MT-455 Delayed neutron data Estimated from Tuttle's semi empirical formula /2/. MT-456 Number of prompt neutrons per fission Based on the relative measurements /3,4/ to the U-235 data, and on the empirical formula by Howerton $\sqrt{5}$. The U-235 data of JENDL-2 were used. Nu p 3,268 + 0,172 (MeV). MF-2 Resonance Parameters MT~151 Resonance parameters : below 3.5 eV. Single level B-W. Parameters by Bowman et al. 767 were adopted. Average Gam - g = 0.05 eV, level spacing = 0.45 eV, s wave neutron strength function = 1.4E-4. Calculated 2200m s cross sections and resonance integrals. 2200 m sec Res. Integ. 1342. b 207 b 6620. b 1530 b capture fission 6,698 b elastic 7969. b total MF=3 Neutron Cross Sections The resonance region: Null value is given for the total, elastic scattering, capture

and fission cross sections.

From 3.5 eV to 1.5 keV:

The fission cross section was evaluated by fitting spline functions to the experimental data by Bowman et al. /6/ and Seeger et al. /7/. The capture and elastic scattering cross sections were estimated by assuming that the cross sections have the same structure as that of the fission cross section. Calculations were made by using the radius parameters of 9.45 fm. average fission and capture widths of 0.385 eV and 0.05 eV respectively.

Above 1.5 keV:

MT=1.2.4.51-73.91.102.251 Sig-t.Sig-el.Sig-in.Sig-c.Mu-bar Calculated with optical and statistical models above 1.5 keV.

The spherical optical potential parameters (MeV, fm) :

V = 42.0 - 0.107 * E, r = 1.282, a = 0.6Ws= 9.0 - 0.339 + E + 0.0531 + E + + 2, r= 1.29, a= 0.5Vso = 7.0, r = 1.282, a = 0.6Statistical model calculation with CASTHY code /8/. Competing processes : fission, (n, 2n) and (n, 3n), Level fluctuation considered. $Gam \cdot g = 0.05 \text{ eV}$ and D =0.45 eV used for capture cross section calculation. The level scheme taken from the compilation by Ellis and Haese /9/, with shifted energy origin at -49 keV. No. Energy (MeV) Spin-Parity g.s. 0.049 1 -0.05 0 1 2 0.0 3 ---3 0.0 (meta stable) 5 4 0.025 2 5 0.064 6 6 0.099 4 ---7 0.099 5 -7 8 0.141 _ 3 9 0,193 ----6 10 0.214 -7 11 0.214 ••• 0.239 12 4 13 0.239 2 35 14 0.276 -15 0.292 16 0.323 4 6 17 0.361 -5 18 0.381 ... 7 19 0.439 0.451 6 20 ----7 21 0.532••• 22 0.630 8 Overlapping levels are assumed above 0.632 MeV. The level density parameters of Gilbert and Cameron /8/. (n,2n) and (n,3n) cross sections

Calculated with the evaporation model by Pearlstein /9/. MT=18 Fission cross section

Smooth cross section above 1.5 keV was obtained by fitting a semi-empirical formula to the averaged experimental data.

MF=4 Angular Distributions of Secondary Neutrons

MT=2 Legendre coefficients are given by the optical and statistical model calculations.

MT=16,17,18,91 Isotropic distributions in the laboratory system. MT=51-72 Isotropic distributions in the center-of-mass system.

MF=5 Energy Distributions of Secondary Neutrons

MT=16.17.91 Evaporation spectrum

MT=18 Fission spectrum estimated from Z**2/A systematics by Smith et al. /10/ by assuming E(Cf-252) = 2.13 MeV.

References

MT=16.17

1) T. Nakagawa and S. Igarasi : JAERI-M 8903 (1980), in Japanese. 2) R.J. Tuttle : INDC(NDS)-107/G+Special, 29 (1979).

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- 3) A.H. Jaffey and J.L. Lerner : Nucl. Phys., A145, 1 (1970).
- 4) N.I. Kroshkin and Yu.S. Zamyatnin : Atom. Energ., 29, 95
- (1970), Sov. Atom. Energy, 29, 790 (1970). 5) R.J. Howerton : Nucl. Sci. Eng. 62, 438 (1977).
- 6) C.D. Bowman et al. : Phys. Rev., 166, 1216 (1968).
- 7) P.A. Seeger et al. : Nucl. Phys., A96, 605 (1967).
- 8) S. Igarasi : J. Nucl. Sci. Technol., 12, 67 (1975).
 9) Y.A. Ellis and R.L. Haese : Nucl. Data Sheets 21, 615 (1977).
- 10) A. Gilbert and A.G.W. Cameron : Can. J. Phys., 43, 1446 (1965).
- 11) S. Pearlstein : Nucl. Sci. Eng., 23, 238 (1965).
- 12) A.B. Smith et al. : ANL/NDM-50 (1979).

95-Am-243 MAT number = 2954

95 Am 243 JAERI Eval Mar82 Y.Kikuchi JAERI M82 096 Dist Mar83 Revi-Nov83 History 77-03 New evaluation was made by S.Igarasi and T.Nakagawa (JAERI). Details are given in Ref. /1/.82-03 Complete reevaluation for JENDL-2 was made by Y.Kikuchi (JAERI). Details are given in Ref. $\frac{2}{.}$ 83-11 Comment was added. MF-1 General Information MT=451 Comment and dictionary MT-452 Number of neutrons per fission Sum of Nu p MT 456 and Nu d (MT-455). Fission product yield data MT: 454 Taken from ENDF B IV and renormalized to 2.0. MT=455 Delayed neutron data Estimated with semi-empirical formula by Tuttle /3/. MT=456 Number of prompt neutrons Estimated from systematics. Same as previous evaluation /1/. MF-2,MT-151 Resonance parameters Resolved resonances for MLBW formula : 1.0E-5 - 215 eV. Based on the data of Simpson $+ \frac{1}{4}$. The results are the same as the previous evaluation /1/ except fission widths which were determined to reproduce the fission cross section of 0.225 barns at 0.0253 eV. The neutron width of a negative resonance was adjusted to the thermal capture and total cross sections. Unresolved resonances : 215 eV 30 keV Obtained from optical model calculation: S0=0.93E-4, S1=2.44E-4, R=9.34 fm Estimated from resolved resonances: Dobs=0.67 eV, Gam-g=0.039 eV, Gam-f=0.00012 eV Calculated 2200-m/s cross sections and resonance integrals 2200 m/s value Res. Int. 7.528 b elastic capture 78.50 b 1820 b 0.2281 b fission 11.4 b 86.26 b total _ MF=3 Neutron Cross Sections MT=1.2.4.51-59.91.102.251 Sig-t.Sig-el.Sig-in.Sig-c.Mu-bar Calculated with optical and statistical models. Optical potential parameters were obtained /1/ by fitting the data of Phillips and Howe 5 for Am-241: V = 43.4 - 0.107 En (MeV) Ws= 6.95 - 0.339*En + 0.0531*En**2 (MeV)

, Vso = 7.0

(MeV)

Wv= 0

r = rso = 1.282 , rs = 1.29(fm) a = aso = 0.60 , b = 0.5(fm) Statistical model calculation with CASTHY code /6/. **Competing** processes : fission, (n, 2n), (n, 3n), (n, 4n). Level fluctuation considered. The level scheme taken from Ref. /7/ No Energy(keV) Spin-Parity g.s. Û. 5/2 --42.2 7/2 -1 2 5/2 + 84.0 3 96.4 9/2 -4 109.3 7/2 + 5 143.5 9/2 + 6 189.3 11/2 + 7 267.0 32 -8 298.0 5 2 -9 344.0 7.2 -

Continuum levels assumed above 383 keV.

The level density parameters : Gilbert and Cameron /9/. Gamma-ray strength function deduced from resonance parameters

- MT=18 Fission Evaluated on the basis of the measured data of Behrens and Browne /9/ above 200 keV. The curve vas smoothly connected to the unresolved resonance region below 200 keV. High sub-threshold cross-section values of Seeger +
 - /10/ were abandoned.
- MF=4Angular Distributions of Secondary NeutronsMT=2.51-66.91Calculated with optical model.MT=16.17.18.37Isotropic in the laboratory system.

MF=5 Energy Distribu	tions of Secondary Neutrons	
MT=16.17.37.91	Evaporation spectrum.	
MT=18	Maxwellian fission spectrum. was estimated from the Z*42/A	Temperature systematics
	of Smith + $/11/$.	

- 1) Igarasi S. and Nakagawa T. : JAERI-M7174 (1977).
- 2) Kikuchi Y. : JAERI-M82-096 (1982).
- 3) Tuttle R.J. : INDC(NDS)-107/G+Special , p.29 (1979).
- 4) Simpson O.D. et al. : Nucl.Sci.Eng., 55, 273(1974)
- 5) Phillips T.W. and Howe R.E. : Nucl.Sci.Eng., 69,375 (1979).
- 6) Igarasi S. ; J.Nucl.Sci.Technol., 12,67 (1975).
- 7) Lederer C.M. and Shirley V.S. : Table of Isotopes , 7th Ed.
- 8) Gilbert A. and Cameron A.G.W. : Can.J.Phys. 43,1446 (1965).
- 9) Behrens J.W. and Browne J.C. : Nucl.Sci.Eng. .77.444 (1981).
- 10) Seeger P.A. et al. : Nucl.Phys., A96, 605 (1967).
- 11) Smith A.B. : ANL/NDM-50 (1979).

96-Cm-242 MAT number - 2981

96 Cm 242 JAERI Eval-Mar79 S. Igarasi, T. Nakagawa JAERI-M 8342 (1979) Dist-Mar83 Rev1-Nov83 History 79-03 Evaluation was made by S.Igarasi and T.Nakagawa (JAERI) /1/. 83-11 Comment was added. MF-1 General Information MT=451 Descriptive data MT: 452 Number of neutrons per fission Based on the empirical formula by Howerton 2 Nu bar 3.50 ± 0.17 E.MeV MT 455 Delayed neutron data Estimated from the systematics by Tuttle 3. MF-2 Resonance Parameters MT=151 Resonance parameters : Below 275 eV. Multi-level B-W. Parameters by Artamonov et al. /4/ plus one negative level at -3,45 eV. No fission width is given for all the resonances. Fission cross section in this region is assumed to be 1 v normalized to 5.0 barns at 0.0253 e¥. Average Gam g = 0.04 eV Effective scattering radius - 9.38 fm Calculated 2200m s cross sections and resonance integrals. 2200 m sec Res. Integ. 15,92 b 116 b capture fission 5.00 b 11.1 b 11.61 b elastic 32.53 b total MF=3 Neutron Cross Sections MT=1,2,4,51-53,91,102,251 Sig-t,Sig-el,Sig-in,Sig-c,Mu-bar Calculated with optical and statistical models. The spherical optical potential parameters (MeV, fm) : V = 43.4 - 0.107 + E, r= 1.282, a= 0.6, Ws=6.95-0.3394E+0.05314E442 , r= 1.29 , a= 0.5 , Vso = 7.0. r = 1.282, a = 0.6This potential reproduces well the total cross section of Am-241 by Phillips and Howe /5/ Statistical model calculation with CASTHY code /6/. Competing processes : fission. (n, 2n) and (n, 3n). Level fluctuation considered. Gam-g = 0.036 eV and D = 16 eV used for capture cross section calculation. The level scheme taken from the compilation by Ellis and Haese 7: Spin-Parity No. Energy (MeV) 0.0 0 + g.s. 1 0.0422 2 + 2 0.138 4 + 3 6 + 0.284

Overlapping levels are assumed above 0.35 MeV. The level density parameters of Gilbert and Cameron 787.

- MT = 16, 17 (n.2n) and (n.3n) cross sections
- Calculated with the evaporation model by Pearlstein 79%. MT-18 Fission cross section

Determined from the evaluated fission cross section of Cm-244 /10/ and the empirical formula on the fissioncross-section systematics around 4 MeV by Behrens and Howerton /11/.

MF-4 Angular Distributions of Secondary Neutrons

Calculated with the optical and statistical models. ML-5 Legendre coefficients are given.

- MT=51-53
- Isotropic distributions in the center of mass system. MT-16.17.18.91

Isotropic distributions in the laboratory system.

- MF-5 Energy Distributions of Secondary Neutrons
 - MT-16,17.91 Evaporation spectrum.
 - Estimated from $Z \mapsto 2/A$ systematics by Smith et al. /12/, MT - 18 assuming E(Cf-252) = 2,13 MeV.

- 1 S. Igarasi and T. Nakagawa: JAERI-M 8342 (1979), in Japanese.
- 2) R.J. Howerton : Nucl. Sci. Eng. 62, 438 (1977).
- 3) R.J. Tuttle: INDC(NDS)-107/G(Special, 29 (1979),
- 4) V.S. Artamonov et al. : Proc. of 4th All Union Conf. on Neutron Physics, Kiev (1977), Vol. 2, 257.
- 5) T.W. Phillips and R.E. Howe : Nucl. Sci. Eng. 69, 375 (1979).
- 6) S. Igarasi : J.Nucl.Sci.Technol. 12, 67 (1975). 7) Y.A. Ellis and R.L. Haese : Nucl. Data Sheets 21, 615 (1977).
- 8 A. Gilbert and A.G.W. Cameron : Can. J. Phys. 43, 1446 (1965).
- 9 S. Pearlstein : J. Nucl. Energy 27, 81 (1973).
- 10 S. Igarasi and T. Nakagawa : JAERI M 7175 1977).
- 11 J.W. Behrens and R.J. Howerton : Nucl. Sci. Eng. 65, 464 1978
- 12) A.B. Smith et al. : ANL NDM 50 (1979).

96-Cm-243 MAT number = 2962

96-Cm-243 JAERI Eval-Mar81 T.Nakagawa, S.Igarasi JAERI-M 9601 (1981) Dist-Mar83 Rev1-Apr84 History 81-03 Evaluation was made by T.Nakagawa and S.Igarasi (JAERI) /1/. 83-11 Comment was added. 84-04 The capture and t tal cross sections were corrected in the energy range from 2/ eV to 1 keV. MF=1 General Information MT=451 Descriptive data MT=452 Number of prompt neutrons per fission Based on the experimental data at thermal energy by Jaffey and Lerner $\frac{1}{2}$, and Zhuravlev et al. $\frac{3}{3}$, and on the empirical formula by Howerton /4/. Nu-bar = 3.43 + 0.178 + E(MeV)MT-455 Delayed neutron data Estimated from the systematics by Tuttle 757. MF-2 Resonance Parameters MT=151 Resonance parameters : Below 27 eV. Reich-Moore formula. Parameters by Berreth et al. /8/ plus one negative level at -0.7 eV. Average Gammag = 0.04 eV, level spacing = 2.2 eV, Gam=n = 0,00041 eV S-wave strength function = 2.20E-4 Effective scattering radius = 9.81 fm Calculated 2200m/s cross sections and resonance integrals. 2200 m/sec Res. Integ. 436 b 1750 b 131.3 b capture fission elastic 612.3 b 9.658 b 753.3 Ь total -MF=3 Neutron Cross Sections Null value is given for the total, elastic scattering, capture and fission cross sections in the resonance region. The capture and elastic scattering cross sections from 27 eV to 1 keV were estimated by assuming that the cross sections have the same structure as that of the fission cross section which was made by averaging the experimental data by Silbert /7/ with Gaussian weight factor. Calculations were made by using the radius parameter of 9.81 fm. average fission and capture widths of 0.37 eV and 0.04 eV, respectively. MT=1,2,4,51-64,91,102,251 Sig-t.Sig-el,Sig-in,Sig-c,Mu-bar Calculated with optical and statistical models above 1 keV. The spherical optical potential parameters (MeV, fm) : Y= 42.0 − 0.107*E , r= 1.282, a= 0.6, Ws = 9.0 - 0.339 *E + 0.0531 *E**2, r = 1.29, a = 0.5, $V_{so} = 7.0$, r = 1.282, a = 0.6

Statistical model calculation with CASTHY code /8/. Competing processes : fission, (n,2n), (n,3n) and (n,4n). Level fluctuation considered. Gam-g = 0.04 eV and D = 2.2 eV used for capture cross section calculation. The level scheme taken from the compilation by Ellis /9/ Spin-Parity No. Energy (MeV) 0.0 5/2 g.s. 0.042 7/2 + 1 2 0.087 1/2 + 9/2 + 3 0.094 4 3/2 0.094 + 5 0.133 7/2 + 6 0.153 11/2 +7 9/2 + 0.164 8 0.219 13/2 + 9 0.228 11/2 -+-10 0.260 9/2 + 11 0.530 15/2 -12 0.729 1/2 -13 0.769 3/2 -14 0.798 5/2 + Overlapping levels are assumed above 0.82 MeV. The level density parameters of Gilbert and Cameron /10/ vere used. MT = 16, 17, 37 (n, 2n), (n, 3n) and (n, 4n) cross sections Calculated with the evaporation model by Pearlstein/11/. MT=18 Fission cross section Smooth cross section above 1 keV was obtained by fitting a semi-empirical formula to the averaged experimental data. MF=4 Angular Distributions of Secondary Neutrons MT=2.51-64.91 Legendre coefficients calculated with CASTHY /B/. MT=16.17.18.37 Isotropic distributions in the laboratory system. MF=5 Energy Distributions of Secondary Neutrons MT=16,17,37,91 Evaporation Spectrum. Fission spectrum estimated from Z*+2/A systematics by MT=18 Smith et al. /12/ by assuming E(Cf-252) = 2.13 MeV. References 1) T. Nakagawa and S. Igarasi: JAERI-M 980! (1981). 2) A.H. Jaffey and J.L. Lerner : Nucl. Phys., A145, 1, (1970). 3) K.D. Zhuravlev et al. : Proc. 2nd Nat. Soviet Conf. on Neut. Phys., Vol.4, 57 (1974). 4) R.J. Howerton : Nucl. Sci. Eng., 62, 438 (1977). 5) R.J. Tuttle: INDC(NDS)-107/G+Special, 29 (1979). 6) J.R. Berreth et al. : Nucl. Sci. Eng., 49, 145 (1972). 7) M.G. Silbert : LA-6239 (1976). 8) S. Igarasi : J. Nucl. Sci. Technol., 12, 67 (1975). 9) Y.A. Ellis : Nucl. Data Sheets, 19, 103 (1976). 10) A. Gilbert and A.G.W. Cameron : Can. J. Phys., 43, 1446 (1965). 11) S. Pearlstein : J. Nucl. Energy 27, 81 (1973).

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12) A.B. Smith et al. : ANL/NDM-50 (1979).

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96-Cm-244 MAT number = 2963

96-Cm-244 JAERI Eval-Mar77 S.Igarasi,T.Nakagawa JAERI-M 7175 (1977) Dist-Mar83 Rev1 Nov83 History 77-03 Evaluation was made by S.Igarasi and T.Nakagawa (JÁERI) /1/, 83-11 Comment was added.

MF=1 General Information

MT=451 Descriptive data

- MT=452 Number of neutrons per fission Determined from semi-empirical formula by Howerton /2/. Nu-bar-3.24 + 0.184+E(MeV)
- MT=455 Delayed neutron data Estimated from semi empirical formula by Tuttle .3/.
- MF-2 Resonance Parameters
 - MT 151 Resonance parameters : Below 1 keV. Multi-level B-W. Above 20 eV, parameters by Moore and Keyworth /4/ were adopted assuming neutron width of 0.2 eV for 646.9, 759.7, 914.0 and 971.5 eV levels, and below 20 eV, evaluation by Benjamin et al. /5/. For fission and capture cross sections, background cross sections proportional to 1/v were added.

Calculated 2200m/s cross sections and rc onance integrals. 2200 m/sec Res. Integ. capture 14.41 b 594 b

capture	[4.4] D	594 D
fission	1.180 b	18.4 b
elastic	6.650 b	
total	22.24 b	

MF 3 Neutron Cross Sections

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MT=1.2.4.51 58.91.102.251 Sig t.Sig el.Sig in.Sig c.Mu bar Calculated with optical and statistical models above 1 keV.

The spherical optical potential parameters (MeV, fm): V= 40.5 + 0.5 +E , r= 1.32 , a= 0.47 Ws= 8.2 + 0.5 +E + 0.5 , r= 1.32 , a= 0.47 Vso= 7.0 , r= 1.32 , a= 0.47 Statistical model calculation with CASTHY code /6/. Competing processes : fission. (n.2n) and (n.3n). Level fluctuation considered. Gam-g = 0.037 eV and D = 14 eV used for capture cross section calculation. The level scheme taken from the compilation by Schmorak .7

No.	Energy (MeV)	Spin-Parity		
g.s.	0.0	0		
1	0.0429	2 🐨		
2	0.1423	4 +		
3	0.296	6 +		
4	0.502	8 +		
5	0.970	3 -		

6	1.038	2 ·
7	1.042	6 +
8	1.187	3

Overlapping levels are assumed above 1.2 MeV.

The level density parameters of Gilbert and Cameron 8. MT 16.17 (n.2n) and (n.3n) cross sections

- Calculated with the evaporation model by Pearlstein 9. MT-18 Fission cross section
 - Smooth cross section above 1 keV was obtained by fitting a semi empirical formula to the averaged experimental data.

MF-4 Angular Distributions of Secondary Neutrons

- MT -2 Legendre coefficients are given by the optical and statistical model calculations.
- MT 51 58 Isotropic distributions in the center of mass system. MT 16.17.18.91

Isotropic distributions in the laboratory system.

MF 5 Energy Distributions of Secondary Neutrons

MT 16,17,91 Evaporation spectrum

MT 18 Fission spectrum estimated from Z++2 A systematics by Smith et al. 10 by assuming E(Cf 252) < 2.13 MeV.

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96 Cm - 245 MAT number 2964

96 Cm 245 JAERI Eval Mar78 S.Igarasi T.Nakagawa JAERI-M 7733 (1978 Dist Mar83 Rev) Nov83 History 78-03 Evaluation was made by S.Igarasi and T.Nakagawa (JAERI 1, 83-11 Comment was added.

MF 1 General Information

MT 451 Descriptive data

MT 452 Number of neutrons per fission Based on the experimental data at thermal energy by Jaffey and Lerner 2. Kroshkin and Zamyatnin 3 and Zhuravlev et al. 4. Energy dependent term was derived from the neutron separation energy. NJ bar 3.83 0.18-E MeV

MT 455 Delayed neutron data Estimated from the systematics proposed by Tuttle 5/.

MF 2 Resonance Parameters

MT 151 Resonance parameters : Below 60 eV. Single level B-W. Parameters by Moore and Keyworth 6 were adopted above 20 eV. and those by Browne et al. 7 below 20 eV with a little modification of a negative resonance so that the thermal cross section could be in agreement with the experimental data. The differences between Reich-Moore and single level B W formulas are treated as the background cross sections.

Average Gam g = 0.04 eV, Level spacing = 1.8 eV, S wave neutron strength function = 1.0 E 4

Calculated 2200m s cross sections and resonance integrals. 2200 m sec Res. Integ. capture 346.4 b 108 b fission 2001. b 800 b elastic 11.59 b total 2359. b

MF-3 Neutron Cross Sections

MT=1,2,4,51-66,91,102,251 Sig-t,Sig-el,Sig-in,Sig-c,Mu-bar Calculated with optical and statistical models above 60 eV. The spherical optical potential parameters (MeV, fm) : r-1.32 . a 0.47 Vso= 7.0 , Statistical model calculation with CASTHY code /8/. Competing processes : fission. n.2n and (n.3n). Level fluctuation considered. Gam g = 0.04 eV and D 14 eV used for capture cross section calculation. The level scheme taken from the compilation by Ellis /9/ No. Energy(MeV) Spin-Parity 7/2 +g.s. 0.0

1	0.05473	92+
2	0.1214	11 2 +
3	0.1971	13 2 +
4	0.25285	52+
5	0.29584	7.2 +
6	0.3505	9.2 1
7	0.35595	121
8	0.3615	32 (
9	0.38795	92
10	0.4170	11 2 +
11	0.4188	52+
12	0.431	7/2 +
13	0.4428	11.5
14	0.498	13 2 +
15	0.5087	13 2
16	0.532	92.

Overlapping levels are assumed above 0.55 MeV.

The level density parameters of Gilbert and Cameron (107) were used.

MT 16.17 n.2n and n.3n cross sections

Calculated with the evaporation model by Pearlstein /11/ MT 18 Fission cross section

Smooth cross section above 60 eV was obtained by fitting a semi empirical formula to the averaged experimental data.

MF 4 Angular Distributions of Secondary Neutrons

- MT 2 Legendre coefficients are given by the optical and statistical model calculations.
- MT-51-66 Isotropic distributions in the center of mass system. MT-16.17.18.91

Isotropic distributions in the laboratory system.

- MF 5 Energy Distributions of Secondary Neutrons
 - MT 16.17.91 Evaporation spectrum
 - MT 18 Fission spectrum estimated from Z-2 A systematics by Smith et al. 12 by assuming E Cf 252 2.13 MeV.

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Appendix 1

Thermal and Fast Neutron Cross Sections

The 2200 m sec and 14 MeV cross sections, resonance integrals, and Maxwellian and fission spectrum average cross sections were calculated from JENDL 2 (Rev1) pointwise data files. Details of the calculation are described in the text.

page 1

Nuclide MAT	Reaction	Thermal cross sections	Fast cross sections	
		2200-m/s Maxv.Avg. Res.Integ	14-MeV Fiss.Avg.	
1-H - 1	2011	total elastic capture	20.77 b 20.74 b 20.44 b 20.44 b 331.9 mb 294.1 mb 149.0 mb	691.9 mb 3.926 b 691.9 mb 3.926 b 29.83 μb 39.27 μb
1-H - 2	2012	total elastic (n.2n) capture	3.390 b 3.390 b 3.389 b 3.390 b Threshold energy = 3.339 MeV 550.0 μb 487.3 μb 286.2 μb	801.4 mb 2.537 b 624.2 mb 2.532 b 177.1 mb 5.276 mb 9.521 μb 7.076 μb
3-Li- 6	2031	total elastic inelastic (n,2n'α) capture (n,p) (n,α)	937.0 b 730.5 b 735.9 mb 736.2 mb Threshold energy = 1.718 MeV Threshold energy = 4.318 MeV 28.00 mb 24.81 mb 12.60 mb Threshold energy = 3.192 MeV 336.2 b 829.7 b 423.0 b	1.247 b 1.908 b 709.9 mb 1.404 b 434.6 mb 149.0 mb 69.00 mb 176.2 μb 1.190 μb 4.257 μb 7.200 mb 4.184 mb 25.74 mb 350.3 mb
3-Li- 7	2032	2 total elastic inelastic (n.2n) (n.2n'α) capture (n.d)	1.094 b 1.091 b i.049 b 1.049 b Threshold energy = 0.546 MeV Threshold energy = 8.300 MeV Threshold energy = 10.000 MeV 45.40 mb 40.24 mb 20.42 mb Threshold energy = 8.880 MeV	1.470 b 1.846 b 999.7 mb 1.643 b 404.8 mb 203.0 mb 22.00 mb 28.36 μb 33.00 mb 10.09 μb 1.930 μb 6.903 μb 10.00 mb 5.207 μb
4-Be- 9	204	l total elastic (n,2n) capture (n,p)	6.008 b 6.008 b 5.000 b 6.000 b Threshold energy = 1.850 MeV 7.600 mb 6.735 mb 3.419 mb Threshold energy = 14.260 MeV	1.502 b 2.801 b 961.7 mb 2.639 b 513.9 mb 125.4 mb 323.0 nb 1.156 μb 31.94 nb

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Nuclide MAT		T Reaction	Thermal cross sections	Fast cross	sections
	MAI		2200-m/s Maxw.Avg. Res.Integ	14-MeV	Fiss.Avg.
4-Be- 9	2041	(n,d) (n,t) (n,α)	Threshold energy = 16.300 MeV Threshold energy = 11.610 MeV Threshold energy = 0.670 MeV	15.47 mb 10.81 mb	19.88 nb 2.722 μb 36.92 mb
5-B - 10	2051	total elastic inelastic (n,2n) capture (n,p) (n,d) (n,a) (n,t,2a)	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	1.450 b 952.7 mb 285.4 mb 26.44 µb 21.38 µb 32.80 mb 29.97 mb 60.22 mb 88.65 mb	2.611 b 2.101 b 33,65 mb 316.3 nb 76.47 μb 8.222 mb 1.134 mb 433.7 mb 32.63 mb
6-C - 12	2061	total elastic inelastic capture (n.α)	4.702 b 4.702 b 4.699 b 4.699 b Threshold energy = 4.800 MeV 3.400 mb 3.013 mb 1.530 mb Threshold energy = 6.320 MeV	1.270 b 737.8 mb 450.2 mb 144.5 nb 81.51 mb	2.370 b 2.358 b 10.86 mb 516.9 nb 1.206 mb
9-F - 19	2091	total elastic inelastic (n,2n) $(n,n' \alpha)$ (n,n' p) capture (n,p) (n,d) (n,t)	3.651 b 3.652 b 3.641 b 3.641 b Threshold energy = 0.116 MeV Threshold energy = 10.990 MeV Threshold energy = 4.227 MeV Threshold energy = 8.418 MeV 9.600 mb 8.519 mb 19.59 mb Threshold energy = 4.251 MeV Threshold energy = 6.075 MeV Threshold energy = 7.957 MeV	1.768 b 848.9 mb 496.3 mb 42.94 mb 216.2 mb 72.80 mb 25.66 μb 14.66 mb 39.50 mb 15.00 mb	3.537 b 2.191 b 1.327 b 8.615 μb 2.009 mb 37.46 μb 213.4 μb 1.163 mb 317.7 μb 28.55 μb

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Nuel 1 de	MAT Dr	Thermal cross sections	Fast cross sections
Nuclide	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
9-F - 19	2091 (n,a)	Threshold energy = 1.603 MeV	21.34 mb 14.65 mb
11-Na- 23	2111 total elastic inelastic (n,2n) capture (n,p) (n,α)	3.700 b 3.673 b 3.170 b 3.189 b Threshold energy = 0.458 MeV Threshold energy = 12.960 MeV 529.9 mb 469.0 mb 329.0 mb Threshold energy = 3.784 MeV Threshold energy = 4.035 MeV	1.690 b 3.132 b 918.0 mb 2.581 b 565.9 mb 549.2 mb 18.00 mb 2.144 μb 213.9 μb 274.5 μb 44.70 mb 1.205 mb 142.9 mb 564.7 μb
13-A1- 27	2131 total elastic inelastic (n,2n) capture (n,p) (n,α)	1.730 b 1.711 b 1.500 b 1.500 b Threshold energy = 0.874 MeV Threshold energy = 13.550 MeV 229.9 mb 203.7 mb 147.1 mb Threshold energy = 1.898 MeV Threshold energy = 3.247 MeV	1.750 b 3.181 b 925.5 mb 2.875 b 614.9 mb 301.9 mb 10.50 mb 4.861 μb 427.2 μb 281.5 μb 77.50 mb 3.775 mb 120.9 mb 732.2 μb
14-Si- 0	2140 total elastic inelastic (n.2n) capture (n.p) (n,α)	2.356 b 2.347 b 2.200 b 2.200 b Threshold energy = 1.319 MeV Threshold energy = 8.769 MeV 155.9 mb 138.2 mb 78.83 mb Threshold energy = 2.999 MeV Threshold energy = 0.034 MeV	1.800 b 3.029 b 622.8 mb 2.786 b 752.8 mb 228.5 mb 8.026 mb 2.652 µb 559.0 µb 524.3 µb 255.4 mb 9.626 mb 160.1 mb 4.456 mb
20-Ca- 0	2200 total elastic inelastic (n,2n)	3.396 b 3.357 b 2.963 b 2.963 b Threshold energy = 0.362 MeV Threshold energy = 8.119 MeV	2.268 b 2.906 b 1.016 b 2.713 b 265.8 mb 59.32 mb 11.61 mb 3.097 μb

Nualida	MAT De-stien	Thermal cross sections	Fast cross sections
Nuclide	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
20-Ca- 0	2200 (n,n' α) (n,n' p) capture (n,p) (n,α) (n,2p)	Threshold energy = 7.218 MeV Threshold energy = 8.540 MeV 430.6 mb 382.9 mb 226.1 mb Threshold energy = 0.543 MeV 2.423 mb 2.172 mb 348.7 mb Threshold energy = 8.317 MeV	79.49 mb 14.92 μb 542.8 mb 183.0 μb 21.06 μb 2.130 mb 204.4 mb 87.33 mb 138.4 mb 43.95 mb 9.694 mb 1.371 μb
20-Ca- 40	2201 total elastic inelastic (n,2n) (n,n' α) (n,n' p) capture (n,p) (n,α) (n,2p)	3.423 b 3.385 b 3.010 b 3.010 b Threshold energy = 3.831 MeV Threshold energy = 16.030 MeV Threshold energy = 7.218 MeV Threshold energy = 8.540 MeV 409.9 mb 364.5 mb 215.8 mb Threshold energy = 0.543 MeV 2.500 mb 2.241 mb 355.2 mb Threshold energy = 8.317 MeV	$\begin{array}{rrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrr$
20-Ca- 42	2202 total elastic inelastic (n,2n) capture (n,α)	1.910 b 1.839 b 1.230 b 1.230 b Threshold energy = 1.561 MeV Threshold energy = 11.750 MeV 679.9 mb 604.8 mb 383.8 mb 252.8-12 b 505.8-12 b 535.0 mb	2.352 b 3.757 b 1.072 b 3.269 b 857.7 mb 423.5 mb 120.0 mb 20.48 μb 57.79 μb 3.421 mb 301.9 mb 59.82 mb
20-Ca- 43	2203 total elastic inelastic (n,2n) capture	9.197 b 8.515 b 2.997 b 2.997 b Threshold energy = 0.382 MeV Threshold energy = 8.119 MeV 6.200 b 5.515 b 3.202 b	2.397 b 3.745 b 1.104 b 3.036 b 395.4 mb 664.1 mb 569.9 mb 513.3 µb 28.09 µb 4.971 mb

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http://	MAT Desetion	Thermal cross sections	Fast cross sections
NUCLIDE	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
20-Ca- 43	2203 (n,p) (n,α)	Threshold energy = 1.059 MeV 0.0 b 0.0 b 369.9 mb	95.24 mb 336.7 µb 232.0 mb 34.69 mb
20-Ca- 44	2204 total elastic inelastic .n.2n capture .n.p n.c	2.203 b 2.109 b 1.323 b 1.323 b Threshold energy = 1.184 MeV Threshold energy = 11.390 MeV 879.8 mb 782.4 mb 428.7 mb Threshold energy = 4.988 MeV Threshold energy = 2.815 MeV	2.441 b 3.651 b 1.136 b 3.114 b 846.0 mb 534.4 mb 400.0 mb 74.30 µb 15.42 µb 1.591 mb 34.52 mb 81.96 µb 23.89 mb 58.07 µb
20-Ca- 46	2205 total elastic inelastic (n,2n) capture (n,p) (n,α)	3.640 b 3.569 b 2.900 b 2.900 b Threshold energy = 1.377 MeV Threshold energy = 10.630 MeV 739.9 mb 655.7 mb 339.0 mb Threshold energy = 7.088 MeV Threshold energy = 5.632 MeV	2.527 b 3.932 b 1.202 b 3.487 b 664.7 mb 443.0 mb 650.0 mb 167.4 µb 896.5 nb 172.1 µb 6.600 mb 2.525 µb 4.000 mb 66.88 µb
20-Ca- 48	2206 total elastic inelastic (n.2n) capture (n.p)	3.990 b 3.876 b 2.900 b 2.900 b Threshold energy = 3.912 MeV Threshold energy = 10.150 MeV 1.090 b 967.2 mb 491.4 mb Threshold energy = 11.450 MeV	2.613 b 3.745 b 1.269 b 3.654 b 443.7 mb 89.13 mb 900.0 mb 325.2 µb 1.349 µb 392.5 µb 112.0 µb 41.15 mb
21-Sc- 45	2211 total elastic inelastic (n,2n)	51.05 b 48.18 b 25.03 b 25.03 b Threshold energy = 0.013 MeV Threshold energy = 11.580 MeV	2.131 b 3.193 b 1.129 b 2.479 b 641.7 mb 679.3 mb 245.0 mb 39.25 µb

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Nu-14d	MAT Desetion	Thermal cross sections	Fast cross sections
Nuclide	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
21-Sc- 45	2211 capture (n,p) (n,α)	26.02 b 23.15 b 11.32 b 0.0 b 0.0 b 168.3 mb Threshold energy = 0.406 MeV	55.26 μb 5.916 mb 59.00 mb 25.64 mb 56.00 mb 2.518 mb
23-V - 51	2231 total elastic n.2n (n.n' α (n.n' p capture (n.p) (n.d) (n.t) (n,α)	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$	2.288 b 3.956 b 936.3 mb 3.144 b 735.8 mb 809.1 mb 544.9 mb 96.33 μb 300.0 μb 107.5 nb 13.00 mb 5.233 μb 26.80 μb 2.210 mb 36.50 mb 514.5 μb 4.500 mb 6.695 μb 1.000 mb 324.5 nb 15.00 mb 175.2 μb
24-Cr 0	2240 total elastic inelastic (n.2n) (n.n' p) capture n.p (n.a)	6.900 b 6.565 b 3.830 b 3.830 b Threshold energy = 0.575 MeV Threshold energy = 8.092 MeV Threshold energy = 9.782 MeV 3.070 b 2.730 b 1.600 b Threshold energy = 0.257 MeV 0.0 b 0.0 b 37.92 mb	2.380 b 3.332 b 1.206 b 2.888 b 695.9 mb 440.1 mb 324.4 mb 101.1 µb 13.87 mb 5.768 µb 97.69 µb 3.141 mb 106.3 mb 978.2 µb 33.22 mb 138.2 µb
24-Cr 50	2241 total elastic inelastic (n.2n) (n.n' p)	$\begin{array}{rrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrrr$	2.310 b 3.272 b 1.142 b 2.726 b 341.8 mb 535.6 mb 8.200 mb 1.805 µb 285.9 mb 37.87 µb

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N	N4.1.77	Dec et i en	Thermal c	ross sectio	ons	Fast cross	sections
NUCLIDE	ra i	Reaction	2200-m/s	Maxv.Avg.	Res.Integ	14-MeV	Fiss.Avg.
24-Cr- 50	2241	capture (n.p)	15.90 b Threshold ene	14.14 b ergy ≈ 0.2	7.765 b 262_MeV	118.4 μb 437.9 mb	7.836 mb 2.980 mb
		(n .α)) 0.0 b	U.U D	76.99 mb	94.00 mb	128.5 µb
24-Cr- 52	2242	total	2.983 b	2.901 b		2.369 b	3.377 ь
		elastic	2.223 b	2.223 b		1.195 b	2.964 b
		inelastic	Threshold ene	ergy ≈ – 1.4	462 MeV	783.2 mb	409.2 mb
		(n.2n)	Threshold end	ergy - 12.1	270 MeV	259.9 mb	31.72 μb
		_ n₁n' p ≠	Threshold ene	ergy 🕤 10.'	710 MeV	1.700 mb	4.844 µb
		capture	759.8 mb	675.8 mb	492.9 mb	dı, 45.84	2.888 mb
		(n,p)	Threshold end	ergy = 3.1	258 MeV	99.60 mb	1.005 mb
		(n.a)	Threshold end	ergy = 1.	234 MeV	30.10 ⊯b	149.5 μb
24-Cr- 53	2243	total	34.51 b	32.54 b		2.390 b	3.506 b
		elastic	16.31 b	16.31 b		1.220 b	2.877 b
		inelastic	Threshold en	ergy = 0.	575 MeV	224.5 mb	625.7 mb
		(n, 2n)	Threshold end	ergy = 8.	092 MeV	869.9 mb	704.9 μb
		(n,n'p)	Threshold en	$\operatorname{ergy} = 11.$	350 MeV	3.000 µb	646.0 nb
		capture	18.20 b	16.19 b	8.859 b	13.69 µb	2.220 mb
		(n , p)	Threshold en	ergy = 2.	691 MeV	36.70 mb	b 70,76
		(n , a)	0.0 Ь	0.0 b	34.14 mb	38.50 ⊞b	76.41 µb
24-Cr- 54	224/	total	2 274 h	2.236 h		2 420 h	3,367 b
		elastic	1.914 b	1 914 b		1.245 b	2.760 b
		inelastic	Threshold en	ergv = 0.	851 MeV	150.4 mb	605.6 mb
		n.2n	Threshold en	ergy = 9.	902 MeV	1.000 b	316.8 ub
		capture	359.9 mb	320.0 mb	193.1 mb	19.37 ub	1.831 ⊏b
		(n.p)	Threshold en	ergy = 6.	336 MeV	13.20 mb	5.486 Pb
		·····	Thus a half do		FT 44.37	10.00	4.070

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Nuclide	MAT Dees		Therma	l cross se	ctions		Fast cro	ss sections
Nuclide	MAI Read		2200-m/s	Maxy.Av	g. R	es.Integ	14-MeV	Fiss.Avg.
25-Mn- 55	2251 tota elas inel (n.2 (n.1 (n.1 cap) (n.1 (n.1) (n.1) (n.1)	al stic clastic 2n) n' α) n' ρ, oture p; α;	15.50 b 2.184 b Threshold Threshold Threshold Threshold 13.32 b Threshold Threshold	14.03 2.184 energy = energy = energy = energy = 11.84 energy = energy =	b 0.128 10.420 8.081 8.216 b 1.855 0.636	3 MeV) HeV MeV 5 MeV 14.63 b 14.63 b 5 MeV 5 MeV	2.673 1.401 415.5 770.9 3.440 9.510 664.9 46.90 24.60	b 3.648 b b 2.806 b mb 836.7 mb mb 208.0 μb mb 2.752 μb mb 3.847 μb μb 2.924 mb mb 1.325 mb mb 202.5 μb
26-Fe- 0	2260 tota elas ine. (n.: cap (n.) (n.)	al stic elastic 2n) oture p) a)	14.95 t 12.44 t Threshold Threshold 2.514 t 0.0 t 0.0 t	14.69 12.41 energy = energy = 2.279 0 0.0 0 0.0	b b 0.015 7.775 b b b b	5 MeV 5 MeV 1.349 b 119.5 mb 95.97 mb	2.520 1.245 621.3 422.4 147.8 125.4 105.2	b 3.180 b b 2.521 b mb 649.3 mb mb 106.6 μb μb 3.452 mb mb 5.313 mb mb 1.273 mb
26-Fe- 54	2261 tot. ela ine (n.; cap (n. (n.	tal astic elastic ,2n) pture ,p) ,a)	2.649 492.8 m Threshold Threshold 2.156 0.0 0.0	2.411 492.8 energy = energy = b 1.918 b 0.0 b 0.0	b mb 1.433 13.870 b b b	5 MeV 0 MeV 1.328 b 755.9 mb 98.45 mb	2.374 1.198 704.2 3.230 452.1 359.9 108.0	b 3.331 b b 2.775 b mb 473.8 mb mb 1.906 µb µb 6.415 mb mb 74.29 mb mb 1.306 ab
26-Fe- 56	2262 tot ela ine (n,	tal astic elastic .2n)	15.27 12.46 Threshold Threshold	b 14.96 b 12.46 energy = energy =	b b 0.86 11.40	2 MeV 0 MeV	2.485 1.196 626.6 439.9	b 3.191 b b 2.534 b mb 650,8 mb mb 75.44 μb

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Nuolido	MAT Departion	Thermal cross sections	Fast cross sections		
nucilde	MAI Neaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.		
26-Fe- 56	2262 capture (n,p) (n,α)	2.813 b 2.502 b 1.444 b Threshold energy = 2.972 MeV 0.0 b 0.0 b 98.45 mb	1/9.9 μb 3.236 mb 113.9 mb 1.079 mb 108.0 mb 1.306 mb		
26-Fe- 57	2263 total elastic inelastic (n.2n) capture	2.664 b 2.393 b 202.1 mb 202.1 mb Threshold energy = 0.015 MeV Threshold energy = 7.775 MeV 2.462 b 2.190 b 1.452 b	2.522 b 3.649 b 1.377 b 2.484 b 274.0 mb 1.160 b 869.9 ab 1.707 mb 116.9 μb 4.215 mb		
26-Fe- 58	2264 total elastic inelastic (n,2n) capture (n,p) (n,α)	4.101 b 3.964 b 2.821 b 2.821 b Threshold energy = 0.825 MeV Threshold energy = 10.210 MeV 1.290 b 1.139 b 1.828 b Threshold energy = 5.413 MeV Threshold energy = 1.414 MeV	2.636 b 3.767 b 1.267 b 3.150 b 361.5 mb 615.2 mb 976.9 mb 310.1 µb 40.24 µb 1.634 mb 15.00 mb 5.584 µb 15.00 mb 30.68 µb		
27-Co- 59	2271 total elastic inelastic (n.2n) capture (n.p) $(n.\alpha)$	$\begin{array}{c ccccccccccccccccccccccccccccccccccc$	2.775 b 3.800 b 1.573 b 3.391 h 486.2 mb 399.2 mb 639.9 mb 170.7 µb 79.08 µb 7.760 mb 45.90 mb 1.321 mb 30.00 mb 164.8 µb		
28-Ni- 0	2280 total elastic inelastic (n,2n)	21.20 b 20.71 b 16.77 b 16 77 b Threshold energy = 0.069 MeV Threshold energy = 7.950 MeV	2.716 b 3.633 b 1.386 b 3.115 b 390.7 mb 434.6 mb 162.0 mb 46.72 μb		

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N	MAT D+:	Thermal cross sections	Fast cross sections
Nuclide	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
28-Ni- 0	2280 (n.3n) (n,n' α) (n,n' p) capture (n,p) (n,α)	Threshold energy = 16.760 MeV Threshold energy = 6.509 MeV Threshold energy = 8.313 MeV 4.429 b 3.939 b 2.221 b 0.0 b 0.0 b 627.0 mb 2.4-12 b 4.9-12 b 115.5 mb	1.737 nb 20.36 mb 5.132 μb 341.7 mb 70.52 μb 97.08 μb 7.940 mb 307.8 mb 71.04 mb 106.8 mb 4.418 mb
28-Ni- 58	2281 total elastic inelastic (n,2n) $(n,n^{*}\alpha)$ $(a,n^{*}p)$ capture (n,p) (n,α)	30.62 b 30.12 b 26.02 b 26.02 b Threshold energy = 1.480 MeV Threshold energy = 12.410 MeV Threshold energy = 6.509 MeV Threshold energy = 8.313 MeV 4.605 b 4.096 b 2.206 b 0.0 b 0.0 b 870.8 mb 3.5-12 b 7.0-12 b 149.5 mb	2.662 b 3.869 b 1.351 b 3.400 b 248.8 mb 370.2 mb 21.50 mb 2.752 µb 30.00 mb 7.561 µb 479.9 mb 100.2 µb 66.88 µb 8.743 mb 400.0 mb 103.3 mb 130.9 mb 6.113 mb
28-Ni- 60	2282 total elastic inelastic (n.2n) (n.n'p) capture (n.p) (n.e)	3.813 b 3.503 b 1.013 b 1.013 b Threshold energy = 1.355 MeV Threshold energy = 11.580 MeV Threshold energy = 9.693 MeV 2.801 b 2.490 b 1.503 b Threshold energy = 2.075 MeV 151.6-15 b 303.4-15 b 48.24 mb	2.760 b 3.694 b 1.398 b 3.135 b 737.8 mb 549.0 mb 370.5 mb 58.59 µb 60.00 mb 9.007 µb 56.11 µb 5.030 mb 130.9 mb 3.187 mb 63.00 mb 838.2 µb
28-Ni- 61	2283 total elastic inelastic (n.2n)	12.12 b 11.84 b 9.611 b 9.611 b Threshold energy = 0.089 MeV Threshold energy = 7.950 MeV	2.814 b 3.814 b 1.501 b 2.723 b 319.9 mb 1.078 b 843.4 mb 1.432 mb

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N	MAT Desetion	Thermal cross sections	Fast cross sections
Nucilde	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
28-Ni- 61	2283 (n,n' p) capture (n,p) (n,α)	Threshold energy = 10.020 MeV 2.506 b 2.229 b 2.438 b Threshold energy = 0.549 MeV 758.7-15 b 1.5-12 b 62.91 mb	7.400 mb 1.137 µb 29.40 µb 5.621 mb 97.00 mb 1.507 mb 45.00 mb 3.851 mb
28-Ni- 62	2284 total elastic inelastic (n.2n) (n.n' p) capture (n.p) (n,α)	23.70 b 22.14 b 9.505 b 9.505 b Threshold energy - 1.192 MeV Threshold energy = 10.770 MeV Threshold energy = 11.320 MeV 14.20 b 12.64 b 6.908 b Threshold energy = 4.532 MeV Threshold energy = 0.444 MeV	2.869 b 3.810 b 1.475 b 3.211 b 578.8 mb 594.8 mb 771.2 mb 185.3 μb 700.0 μb 1.047 μb 19.63 μb 3.526 mb 23.00 mb 73.33 μb 20.20 mb 73.38 μb
28-Ni~ 64	2285 total elastic inelastic (n.2n) (n,3n) capture (n,p) (n,α)	1.515 b 1.351 b 34.64 mb 34.64 mb Threshold energy = 1.367 MeV Threshold energy = 9.809 MeV Threshold energy = 16.760 MeV 1.480 b 1.317 b 819.3 mb Threshold energy = 6.627 MeV Threshold energy = 2.480 MeV	2.984 b 3.982 b 1.546 b 3.513 b 312.7 mb 466.2 mb 1.115 b 1.115 b 524.7 μb 160.8 mb 2.633 μb 2.731 mb 4.500 mb 3.415 μb 5.700 mb
29-Cu- 0	2290 total elastic inelastic (n.2n) (n.n' α) capture (n,p)	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	3.132 b 4.048 b 1.623 b 3.385 b 812.8 mb 653.2 mb 650.4 mb 153.2 µb 11.99 mb 3.501 µb 82.65 µb 9.872 mb 6.489 mb 188.6 µb

Nualida	MAT Depation	Thermal cross sections	Fast cross sections
Nucitae	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
29-Cu- 0	2290 (n,α)	0.0 b 0.0 b 23.17 mb	26.95 mb 379.3 μb
29-Cu- 63	2291 total elastic inelastic (n.2n) $(n.n' \alpha)$ capture $(n.\alpha)$	9.471 b 8.975 b 4.979 b 4.979 b Threshold energy = 0.680 MeV Threshold energy = 11.030 MeV Threshold energy = 5.870 MeV 4.492 b 3.996 b 5.410 b 0.0 b 0.0 b 33.53 mb	3.105 b 4.060 b 1.602 b 3.374 b 917.2 mb 674.2 mb 529.9 mb 85.27 µb 17.00 mb 4.937 µb 111.2 µb 11.28 mb 39.00 mb 548.9 µb
29-Cu- 65	2292 total elastic inelastic (n.2n) (n.n' α) capture (n.p)	16.47 b 16.23 b 14.30 b 14.30 b Threshold energy = C.783 MeV Threshold energy = 10.060 MeV Threshold energy = 6.895 MeV 2.170 b 1.931 b 2.130 b Threshold energy = 1.376 MeV 1.376 MeV	3.191 b 4.024 b 1.670 b 3.409 b 579.2 mb 606.1 mb 919.9 mb 305.0 µb 800.0 µb 288.0 nb 18.73 µb 6.711 mb 21.00 mb 610.7 µb
41-Nb- 93	2411 total elastic inelastic (n.2n) (n.3n) capture (n.p) $(n.\alpha)$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	4.190 b 5.668 b 2.410 b 4.549 b 497.0 mb 1.082 b 1.250 b 1.174 mb 203.3 nb 6.177 μb 31.38 mb 25.00 mb 1.173 mb 8.850 mb 271.8 μb
42-Mo- 0	2420 total elastic inelastic	8.037 b 7.753 b 5.486 b 5.485 b Threshold energy = 0.206 MeV	4.192 b 5.683 b 2.433 b 4.619 b 585.6 mb 1.022 b

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Nu.1:4-	MAT Deset	Thermal	Thermal cross sections			Fast cross sections	
Nuclide	FMI Reacti	2200- m /s	Maxw.Avg.	Res.Integ	14-MeV	Fiss.Avg.	
42-Mo- 0	2420 (n,2n) (n,3n) captur (n,p) (n,q)	e 2.551 b 0.0 b 0.0 b	energy = 6.8 energy = 14.3 2.268 b 0.0 b 0.0 b 0.0 b	392 MeV 360 MeV 25.42 b 29.52 mb 8.136 mb	1.136 b 87.27 μb 25.65 mb 11.36 mb	1.655 nb 169.7 nb 37.54 nb 1.877 mb 217.6 μb	
42-Mo- 92	2421 total elasti inelas (n,2n) captur (n,p) (n,α)	c 5.546 b c 5.545 b tic Threshold e 20.75 mb 0.0 b 0.0 b	5.564 b 5.545 b energy = 1.5 energy = 12.5 18.46 mb 0.0 b 0.0 b	526 MeV 820 MeV 980.7 mb 110.6 mb 16.36 mb	4.190 b 2.550 b 1.491 b 64.30 mb 191.9 μb 61.40 mb 26.90 mb	5.663 b 5.095 b 523.7 mb 21.48 μb 32.67 mb 10.99 mb 355.1 μb	
42-Mo- 94	2422 total elasti inelas (n,2n) (n,3n) captur (n,p) (n,α)	c 5.998 b ttic Threshold Threshold re 13.11 mb Threshold 0.0 b	6.010 b 5.998 b energy = 0.1 energy = 9. energy = 17.5 11.66 mb energy = 1.5 0.0 b	890 MeV 785 MeV 940 MeV 1.425 b 276 MeV 8.588 mb	4.190 b 2.413 b 717.0 mb 1.010 b 177.3 μb 35.10 mb 15.50 mb	5.664 b 4.755 b 864.2 mb 561.4 μb 4.599 nb 42.20 mb 541.0 μb 374.7 μb	
42-Mo- 95	2423 total elasti inelas (n,2m (n,3m captum (n,p)	19.58 b c 5.596 b stic Threshold Threshold Threshold re 13.99 b Threshold	18.02 b 5.584 b energy = 0.3 energy = 7.3 energy = 17.3 12.44 b energy = 0.	206 MeV 450 MeV 230 MeV 118.6 b 145 MeV	4.190 b 2.385 b 326.9 mb 1.440 b 8.643 μb 26.80 mb	5.667 b 4.354 b 1.258 b 2.529 mb 88.92 nb 49.47 mb 403.5 μb	

Nu a 1 é da	MAT Dec-64	Thermal cross sections	Fast cross sections
Miciide	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
42-Mo- 95	2423 (n,α)	0.0 b 0.0 b 9.620 mb	11.90 mb 163.2 μb
42-Mo- 96	2424 total elastic inelastic (n.2n) (n.3n) capture (n.p) (n.α)	5.322 b 5.257 b 4.727 b 4.727 b Threshold energy = 0.787 MeV Threshold energy = 9.251 MeV Threshold energy = 16.700 MeV 595.3 mb 529.7 mb 17.59 b Threshold energy = 2.430 MeV 0.0 b 0.0 b 9.511 mb	4.191 b 5.665 b 2.413 b 4.646 b 487.7 mb 979.5 mb 1.260 b 930.9 μb 59.47 nb 56.81 μb 36.65 mb 20.50 mb 86.89 μb 9.170 mb 483.7 μb
42-Mo- 97	2425 total elastic inelastic (n,2n) (n,3n) capture (n,p) (n,α)	7.953 b 7.720 b 5.853 b 5.853 b Threshold energy = 0.486 MeV Threshold energy = 6.892 MeV Threshold energy = 16.140 MeV 2.100 b 1.867 b 17.25 b Threshold energy = 1.162 MeV 0.0 b 0.0 b 3.959 mb	4.191 b 5.668 b 2.418 b 4.341 b 310.5 mb 1.271 b 1.440 b 4.180 mb 324.4 nb 324.4 nb 7.816 μb 48.67 mb 15.90 mb 63.31 μb 7.100 mb 52.35 μờ
42-Mo- 98	2426 total elastic inelastic (n,2n) (n,3n) capture (n,p) (n,α)	5.772 b 5.757 b 5.642 b 5.642 b Threshold energy = 0.743 MeV Threshold energy = 8.732 MeV Threshold energy = 15.620 MeV 129.9 mb 115.5 mb 6.558 b Threshold energy = 3.842 MeV 0.0 b 0.0 b 4.496 mb	4.191 b 5.663 b 2.415 b 4.589 b 338.0 mb 1.040 b 1.420 b 1.840 mb 414.9 nb 31.71 mb 12.30 mb 11.61 μb 5.530 mb 68.56 μb
42-Mo-100	2427 total	5.499 b 5.477 b	4.191 b 5.666 b

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N	MAT Des - 64	Thermal cross sections	Fast cross sections
NUCLICE MAI	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
42-Mo-100	2427 elastic inelastic (n,2n) (n,3n) capture (n,p) (n,α)	5.300 b 5.300 b Threshold energy = 0.541 MeV Threshold energy = 8.375 MeV Threshold energy = 14.360 MeV 199.0 mb 176.9 mb 3.922 b Threshold energy = 5.501 MeV 0.0 b 0.0 b 2.491 mb	2.429 b 4.449 b 444.4 mb 1.187 b 1.310 b 2.249 mb 167.2 nb 95.09 μb 26.52 mb 7.500 mb 5.042 μb 3.400 mb 6.478 μb
72-Hf-174	2721 total elastic inelastic (n,2n) (n,3n) capture (n,p) (n,α)	397.9 b 353.4 b 8.000 b 8.000 b Threshold energy = 0.092 MeV Threshold energy = 8.682 MeV Threshold energy = 15.730 MeV 399.9 b 345.3 b 491.7 b 0.0 b 0.0 b 2.177 mb 0.0 b 0.0 b 335.3 μb	5.673 b 6.991 b 3.249 b 4.988 b 1.333 b 1.633 b 1.069 b 2284.2 μb 35.69 nb 44.90 μb 365.7 mb 2.446 mb 12.39 μb 327.7 μb 4.761 μb.
72-Hr-176	2722 total elastic inelastic (n,2n) (n,3n) capture (n,p) (n,α)	46.00 b 41.68 b 8.000 b 8.000 b Threshold energy = 0.089 MeV Threshold energy = 8.137 MeV Threshold energy = 14.970 MeV 38.00 b 33.61 b 359.8 b Threshold energy = 0.406 MeV 0.3 5 0.0 b 75.74 μb	5.672 b 6.967 b 3.222 b 5.038 b 521.3 mb 1.635 b 1.928 b 2.426 mb 842.2 nb 307.7 mb 1.183 mb 2.653 μb 69.40 μb 1.102 μb
72-Hf-177	2723 total elastic inelastic (n,2n)	358.9 b 331.8 b 7.000 b 6.991 b Threshold energy = 0.114 MeV Threshold energy = 6.420 MeV	5.678 b 6.984 b 3.253 b 4.806 b 306.6 mb 1.821 b 2.118 b 7.847 mb

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Nuelide	MAT Desetion	Thermal cross sections	Fast cross sections	
Nuclide MA	MAI Reaction	2200-m/s Maxw.Avg. Res.Integ	i4-MeV Fiss.Avg.	
72-Hr-177	2723 (n,3n) capture (n,p) (n,α)	$ \begin{array}{c ccccc} \mbox{Threshold energy} &= & 14.560 \ \mbox{MeV} \\ \mbox{351.9 b} & \mbox{324.7 b} & \mbox{6948. b} \\ \mbox{0.0 b} & \mbox{0.0 b} & \mbox{732.3 } \mbox{μb} \\ \mbox{0.0 b} & \mbox{0.0 b} & \mbox{59.45 } \mbox{μb} \\ \end{array} $	2.252 μb 2.318 μb 344.3 mb 769.0 μb 2.463 μb 57.00 μb 1.062 μb	
72-Hf-178	2724 total elastic inelastic (n.2n) (n.3n) capture (n.p) (n,α)	91.00 b 78.88 b 5.000 b 4.999 b Threshold energy = 0.094 MeV Threshold energy = 7.670 MeV Threshold energy = 14.090 MeV 86.00 b 73.59 b 1915. b Threshold energy = 1.359 MeV 0.0 b 0.0 b 20.82 μ b	5.667 b 6.988 b 3.199 b 5.106 b 448.4 mb 1.803 b 2.019 b 3.315 mb 3.539 μb 757.7 nb 72.12 mb 605.0 μb 564.8 nb 19.80 μb 94.08 nb	
72-Hf-179	2725 total elastic inelastic (n,2n) (n,3n) capture (n,p) (n,α)	$ \begin{array}{cccccccccccccccccccccccccccccccccccc$	5.672 b 6.985 b 3.220 b 4.929 b 368.0 mb 1.925 b 2.084 b 7.487 mb 0.0 b 4.035 µb 235.6 nb 119.5 mb 51.00 µb 64.34 nb 6.400 µb 61.45 nb	
72-Hf-180	2726 total elastic inelastic (n.2n) (n.3n) capture (n.p)	32.00 b 31.07 b 19.43 b 19.85 b Threshold energy = 0.094 MeV Threshold energy = 7.430 MeV Threshold energy = 13.560 MeV 12.60 b 11.16 b 34.59 b Threshold energy = 2.329 MeV	5.667 b 6.986 b 3.161 b 5.132 b 219.8 mb 1.811 b 2.286 b 5.379 mb 0.0 b 5.813 μb 803.0 nb 34.72 mb 54.70 μb 28.75 nb	

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Nuelide		Peactics	Thermal cross sections Fast cross	sections
NUCITOR		Neaction	2200-m/s Maxv.Avg. Res.Integ 14-MeV	Fi ss.Avg .
72-Hf-180	2726	(n. α)	0.0 ь 0.0 ь 21.83 µь 19.40 µь	39 .01 nb
73-Ta-181	2731	total elastic inelastic (n.2n) capture (n.p)	27.32 b 25.03 b 5.370 b 6.110 b 6.101 b 3.012 b Threshold energy = 0.006 MeV 102.3 mb Threshold energy = 7.683 MeV 2.252 b 21.21 b 18.93 b 743.4 b 209.4 mb Threshold energy = 0.242 MeV 4.128 mb	6.992 b 4.944 b 1.949 b 6.467 mb 91.22 mb 454.7 nb
82-Pb- 0	2820	total elastic inelastic (n,2n) (n,3n) $(n,n^{*}\alpha)$ $(n,n^{*}p)$ capture (n,p) (n,α)	11.59 b 11.58 b 5.419 b 11.42 b 11.42 b 2.915 b Threshold energy 0.573 MeV 580.1 mb Threshold energy 6.772 MeV 1.921 b Threshold energy 14.180 MeV 1.921 b O.0 b 0.0 b 4.252 mb 254.9 μb Threshold energy 6.670 MeV 32.43 μb 171.6 mb 157.0 μb 0.0 b 0.0 b 891.9 μb 989.7 μb 0.0 b 0.0 b 3.089 mb 1.154 mb	6.346 b 5.658 b 684.6 mb 1.476 mb 578.5 nb 128.7 nb 30.29 nb 1.741 mb 417.4 nb
82-Pb-204	2821	total elastic inelastic (n,2n) (n,3n) $(n,n' \alpha)$ (n,n' p) capture (n,p) (n,α)	$ \begin{array}{c ccccccccccccccccccccccccccccccccccc$	6.496 b 5.397 b 1.078 b 1.199 mb 88.08 nb 2.136 nb 8.024 nb 19.81 mb 576.0 nb 225.0 nb

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Nuclide	MAI Reaction	2200-m s Maxw.Avg. Res.Integ	14-MeV	Fiss.Avg
82-Pb-206	2822 total	11.37 b 11.37 b	5.348 b	6.421
	elastic	11.34 b 11.34 b	2.855 b	5.329
	inelastic	Threshold energy = 0.807 MeV	634.2 mb	1.089
	(n.2n)	Threshold energy = 8.129 MeV	1.855 b	1.378
	(n.3n)	Threshold energy = 14.890 MeV		201.4
	n n' a	0.0 b 0.0 b 2.720 mb	82.00 µb	68.56
	n.n' p	Threshold energy = 7.283 MeV	2.700 µb	11.04
	capture	28.00 mb 24.90 mb 96.09 mb	1.550 <i>µ</i> b	1.539
	n.p	Threshold energy = 0.747 MeV	2.050 mb	1.020
	(n ,α)	0.0 b 0.0 b 3.766 mb	2.030 mb	813.4
82-Pb-207	2823 total	12.04 b 11.98 b	5.354 b	6.366
	elastic	11.34 b 11.34 b	2.852 b	5.253
	inelastic	Thresbold energy = 0.573 MeV	667.7 mb	1.110
	(n.2 n)	Threshold energy = 6.772 MeV	1.832 b	2.059
	(n , 3n)	Threshold energy = 14.900 MeV		1.556
	(n , n ' a)	0.0 b 0.0 b 963.9 µb	68.10 µb	31.79
	(n . n ' p)	Threshold energy = 7.520 MeV	105.9 µb	49.60
	capture	702.9 mb 625.0 mb 3/4.7 mb	780.7 nb	1.079
	(n.p)	Threshold energy = 0.643 MeV	1.380 mb	5/8.2
	(n.α)	Ι 0.0 5 0.0 5 586.0 μb	434.9 µb	156.2
82-Pb-208	2824 total	11.49 b 11.49 b	5.360 b	6.269
	elastic	11.49 b 11.49 b	2.850 b	5.957
	inelastic	Threshold energy = 2.627 MeV	521.8 mb	308.7
	:n.2n	Threshold energy = 7.404 MeV	1.986 b	1.283
	(n.3n)	Threshold energy = 14.180 MeV		353.2
	(n , n α)	0.0 b 0.0 b 6.455 mb	419.9 μb	200.4
	(n,n p)	Threshold energy = 8.046 MeV	15.00 μb	31.5/
	capture	_ 479.8 μb 426.9 μb 7.833 mb	1.100 mb	1.694

Nuelid.	MAT 1	Bee - 4 i en	Thermal cross	s section	ns	Fast cross	sections
Mucilde	FIA1 1	Reaction	2200-m s Max	₩.Avg.	Res.Integ	14-MeV	Fiss.Avg.
82-Pb-208	28 24	(n,p) (n,α)	Threshold energy 0.0 b	= 4.22 0.0 b	29 MeV 3.911 mb	317.9 µb 1.080 nb	68.52 nb 335.0 nb
90-Th-228	2901	total elastic inelastic in.2n in.3n fission capture	133.0 b 12 12.81 b 12 Threshold energy Threshold energy Threshold energy 300.0 mb 26 119.8 b 10	0.0 b 78 b 7.10 7.11 12.60 6.8 mb 7 0 b	58 MeV 51 MeV 30 MeV 1.024 b 1169. b	5.628 b 3.169 b 4.136 cb 1.637 b 268.9 cb 548.9 cb 240.3-12 b	7.562 b 5.432 b 1.936 b 7.685 mb 42.53 µb 105.8 mb 79.47 mb
90-Th-230	2902	total elastic inelastic (n.2n) (n.3n) fission capture	32.32 b 30 9.774 b 9. Threshold energy Threshold energy Threshold energy 0.0 b 22.55 b 20	0.13 b 750 b = 0.0 = 6.8 = 12.0 0.0 5 0.38 b	54 MeV 21 MeV 90 MeV 1.120 b 1040. b	5.644 b 3.167 b 10.45 mb 1.340 b 528.9 mb 596.9 mb 16.16 nb	7.601 b 5.436 b 1.887 b 10.66 mb 68.64 µb 178.5 mb 87.42 mb
90-Th-232	2903	total elastic inelastic (n.2n) .n.3n) fission capture	19.42 b 18 12.16 b 12 Threshold energy Threshold energy Threshold energy 0.0 b 7.258 b 6.	3.58 b 2.15 b 7 = 0.0 7 = 6.4 7 = 11.6 0.0 b .427 b	50 MeV 65 MeV 610 MeV 636.2 mb 79.93 b	5.740 b 3.266 b 142.9 mb 1.181 b 800.0 mb 250.0 mb 0.0 b	7.561 b 5.257 b 2.125 b 14.46 mb 113.6 µb 78.45 mb 85.74 mb
90-Th-233	2904	total elastic inelastic	1478. b 13.00 b 13 Threshold energy	1312. b 3.00 b 7 = 0.0)17 MeV	5.670 b 3.167 b 15.80 mb	7.614 b 5.296 b 2.054 b

		Thermal cross sections	Fast cross sections
	Reaction	2200-m/s Maxv.Avg, Res.Integ	14-MeV Fiss.Avg.
90-Th-233 290	04 (n.2n) (n.3n) fission capture	Threshold energy = 4.807 MeV Threshold energy = 11.270 MeV 15.00 b 13.34 b 11.08 b 1450. b 1285. b 642.7 b	1.160 b 66.75 mb 916.9 mb 150.7 μb 409.9 mb 109.8 mb 6.071 nb 86.89 mb
90-Th-234 29	05 total elastic inelastic (n.2n) (n,3n) fission capture	14.75 b 14.59 b 13.00 b 13.00 b Threshold energy = 0.048 MeV Threshold energy = 6.219 MeV Threshold energy = 11.030 MeV 0.0 b 0.0 b 260.2 mb 1.750 b 1.551 b 93.68 b	5.680 b 7.646 b 3.168 b 5.497 b 39.56 mb 1.989 b 1.040 b 21.69 mb 1.282 b 228.7 μb 150.0 mb 36.89 mb 154.0 nb 100.9 mb
91-Pa-233 29	11 total elastic inelastic (n.2n) (n.3n) fission capture	53.82 b 48.01 b 11.02 b 11.02 b Threshold energy = 0.007 MeV Threshold energy = 6.545 MeV Threshold energy = 12.130 MeV 0.0 b 0.0 b 4.682 b 42.80 b 36.97 b 779.2 b	5.669 b 7.598 b 3.000 b 4.978 b 103.7 mb 1.496 b 369.5 mb 5.773 mb 418.3 mb 52.40 µb 1.770 b 990.7 mb 7.400 mb i22.1 mb
32-U-233 29	21 total elastic inelastic (n.2n) (n.3n) fission capture	587.8 b 525.0 b 12.70 b 12.73 b Threshold energy = 0.041 MeV Threshold energy = 5.779 MeV Threshold energy = 13.060 MeV 529.8 b 470.7 b 771.4 b 45.30 b 41.43 b 138.6 b	5.770 b 7.686 b 3.278 b 4.806 b 11.30 µb 904.5 mb 134.9 mb 4.152 mb 37.30 mb 6.033 µb 2.320 b 1.888 b 367.9 nb 82.30 mb
92-11-234 29	e2 total	110.1 b 98.40 b	5.742 b 7.455 b

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Nuelide	MAT Decetion	Thermal cross sections	Fast cross sections
	FIAI NOBCLION	2200-m/s Maxw.Avg. Res.Integ	14-MeV Fiss.Avg.
92-U -234	2322 elastic inelastic (n,2n) (n.3n) fission conture	14.72 b 14.58 b Threshold energy = 0.044 MeV Threshold energy = 6.870 MeV Threshold energy = 12.640 MeV 6.387 mb 5.684 mb 6.438 b 96.44 b 83.82 b 608.9 b	3.086 b 5.085 b 393.8 mb 1.082 b 220.9 mb 1.957 mb 127.8 mb 14.39 µb 1.934 b 1.199 b 2 722 µb 1.658 mb
92-U -235	2923 total elastic inelastic (n,2n) (n,3n) fission (n,4n) capture	698.9 b 607.2 b i7.00 b 16.76 b Threshold energy = 0.000 MeV Threshold energy = 5.321 MeV Threshold energy = 12.190 MeV 583.9 b 503.2 b 278.7 b Threshold energy = 17.970 MeV 98.00 b 84.95 b 153.3 b	5.855 b 7.642 b 3.057 b 4.598 b 514.0 mb 1.651 b 191.2 mb 12.78 mb 41.70 mb 7.082 µb 2.051 b 1.248 b 8.449 nb 3.146 µb 130.9 mb
92-U -236	2924 total elastic inelastic (n.2n) (n.3n) fission capture	13.67 b 13.09 b 8.337 b 8.331 b Threshold energy = 0.045 MeV Threshold energy = 6.574 MeV Threshold energy = 11.890 MeV 42.98 mb 38.38 mb 7.611 b 5.295 b 4.717 b 347.0 b	5.826 b 7.752 b 3.110 b 5.450 b 1.144 mb 1.555 b 423.9 mb 8.475 mb 680.9 mb 95.74 µb 1.630 b 595.8 mb 791.4-12 b 141.5 mb
92-U -238	2925 total elastic inelastic (n,2n) (n,3n) fission	11.57 b 11.28 b 8.873 b 8.870 b Threshold energy = 0.045 HeV Threshold energy = 6.170 HeV Threshold energy = 6.170 HeV Threshold energy = 11.320 MeV 3.220 μb 2.888 μb 2.053 b	5.799 b 7.792 b 2.950 b 5.294 b 318.9 mb 2.101 b 909.9 mb 15.36 mb 500.0 mb 82.80 µb 1.117 b 314.6 mb

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Nucl 6 do			Thermal cross sections Fast cross	sections
Nuclide		Neaction	2200-m/s Maxw.Avg. Res.Integ 14-MeV	Fiss.Avg.
92-U -238	2925	capture	2.700 b 2.407 b 279.0 b 3.395 mb	66.72 ∎b
93-Np-237	2931	total elastic inelastic (n,2n) (n,3n) fission capture	208.4 b 185.3 b 5.539 b 27.52 b 27.19 b 2.787 b Threshold energy = 0.033 MeV 105.6 mb Threshold energy = 6.657 MeV 446.2 mb Threshold energy = 12.370 MeV 72.03 mb 19.21 mb 16.68 mb 6.257 b 2.128 b 180.9 b 158.0 b 662.5 b 228.5 mb	7.797 b 5.006 b 1.321 b 3.301 mb 11.70 µb 1.293 b 173.3 mb
93-Np-233	2932	total elastic inelastic (n,2n) (n,3n) fission capture	47.50 b 43.42 b 5.523 t 10.50 b 10.50 b 2.640 t Threshold energy = 0.031 MeV 73.67 mt Threshold energy = 6.253 MeV 142.9 mt Threshold energy = 11.750 MeV 246.2 mt 0.0 b 0.0 b 7.062 b 2.420 t 37.00 b 32.79 b 444.9 b 10.00 µt	7.808 b 4.896 b 1.453 b 3.696 mb 36.09 μb 1.458 b 1.458 b
94-Pu-236	2941	total elastic inelastic (n.2n) (n.3n) fission capture	833.3 b 804.7 b 6.515 t 3.376 b 3.011 b 3.426 t Threshold mergy = 0.045 MeV 93.93 µt Threshold energy = 7.393 MeV 17.37 mt Threshold energy = 13.680 MeV 703.9 µt 65.41 b 63.19 b 100.7 b 764.5 b 738.5 b 1067. b 4.023 µt	8.451 b 5.801 b 418.1 mb 317.4 μb 278.8 nb 2.080 b 149.7 m5
94-Pu-238	2942	total elastic inelastic (n,2n)	592.8 b 508.2 b 10.15 1 27.60 b 27.05 b 3.556 3.556 1 Threshold energy = 0.044 MeV 3.168 1 Threshold energy = 7.028 MeV 419.0 1	i0.79 b 6.300 b 2.154 b 2.830 mb

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Nuelide	мат	Prostian	Thermal cross sections Fast cross	sections
nucitoe		reaction	2200-m/s Maxw.Avg. Res.Integ 14-MeV	Fiss.Avg.
94-Pu-238	2942	(n,3n) fission capture	Threshold energy = 12.930 MeV 155.4 mb 16.55 b 14.10 b 32.42 b 2.722 b 548.7 b 466.9 b 156.3 b 132.8 mb	20.84 μb 2.011 b 318.7 mb
94-Pu-239	2943	total elastic inelastic (n.2n) (n.3n) fission (n.4n) capture	1020. b 973.0 b 5.872 b 8.000 b 7.816 b 2.633 b Threshold energy = 0.008 MeV 402.6 mb Threshold energy = 5.671 MeV 346.9 mb Threshold energy = 12.700 MeV 180.9 mb 741.7 b 694.1 b 301.5 b 2.308 b Threshold energy = 18.600 MeV 1.349 mb	7.709 b 4.429 b 1.362 b 10.63 mb 21.77 μb 1.818 b 45.92 ab 60.75 mb
94-Pu-240	2944	total elastic inelastic (n.2n) (n.3n) fission (n.4n) capture	290.0 b 284.8 b 5.888 b 1.509 b 1.389 b 3.202 b Threshold energy = 0.043 MeV 7.274 mb Threshold energy = 8.561 MeV 427.1 mb Threshold energy = 12.230 MeV 129.7 mb 67.61 mb 61.45 mb 10.09 b Threshold energy = 19,280 MeY 298.4 b 283.3 b	7.844 b 4.984 b 1.420 b 3.920 mb 18.36 µb 1.363 b 5.617 nb 92.71 mb
94-Pu-241	2945	total elastic inelastic (n.2n) (n.3n) fission (n.4n) capture	1388. b ?286. b 5.801 b 10.23 b 9.935 b 3.291 b Threshold energy = 0.042 MeV 221.6 µb Threshold energy = 5.282 MeV 178.0 mb Threshold energy = 11.820 MeV 149.5 mb 1015. b 947.6 b 590.3 b 2.182 b Threshold energy = 17.500 MeV 362.9 b 327.8 b 186.8 b 7.722 mb	7.842 b 5.167 b 879.2 mb 22.66 mb 22.55 μb 1.624 b 19.72 nb 149.0 mb

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Nuclida		Deertin	Thermal cross sections Fast cross	sections
Nucilde		Reaction	2220-m/s Maxv.Avg. Res.Integ 14-MeV	Fi ss.Avg .
94-Pu-242	2946	total	26.65 b 24.74 b 5.918 b	7.849 ъ
1		elastic	8.111 b 8.091 b 3.161 b	5.339 b
		in elas tic	Threshold energy = 0.045 MeV 8.378 mb	1.280 b
ĺ		(n,2 n)	Threshold energy = 6.336 MeV 454.0 mb	6.669 mb
		(n.3n)	Threshold energy = 11.600 MeV 303.0 mb	44.58 µb
		fission	121.2 mb 107.8 mb 6.348 b 1.991 b	1.135 b
		(n,4n)	Threshold energy = 18.160 MeV	116.7 nb
		capture	18.42 b 16.54 b 1117, b 800.0 μb	87.72 ∎b
95-An-241	2951	total	614.6 h 547.5 b 5.742 b	7.779 b
		elastic	11.28 b 10.83 b 2.722 b	4.786 b
		inelastic	Threshold energy = 0.041 MeV 42.21 μ b	1.179 b
		(n,2n)	Threshold energy = 6.610 MeV 282.3 mb	620.3 μb
		(n, 3 n)	Threshold energy = 12.650 HeV 48.74 mb	10. 02 µb
		fission	3.018 b 2.721 b 14.69 b 2.711 b	1.510 b
		(n ,4 n)	Threshold energy = 19.780 MeV	17.0-15 b
	* ******	capture	600.3 b 533.9 b 1299. b 263.5-12 b	297.9 MD
95-An-242	2952	total	7611. b 6747. b 5.906 b	7. 75 8 b
		elastic	11.44 b 11.44 b 2.807 b	4.561 b
		inelastic	Threshold energy = 0.044 MeV 402.2 gb	1.228 b
		(n.2 n)	Threshold energy = 5.515 HeV 130.8 mb	4.387 ∎b
		(n .3 n)	Threshold energy = 12.120 MeV 138.9 mb	17.94 μb
		fission	2100. b 1861. b 1258. b 2.427 b	1.756 b
		capture	i 5500. 5 4872. b 390.4 b i 4.487 µb	206.6 mb
95-An-242M	2953	total	7989. b 7806. b 5.913 b	7.761 b
		elastic	6.698 b 7.429 b 2.807 b	4.527 b
		inelastic	0.0 b 0.0 b 4.597 b 442.2 mb	1.256 b
		(n,2n)	Threshold energy = 5.515 MeV 130.8 mb	4.387 mb

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Nuclida	MAT	Deastion	Thermal cross sections Fast cross	Fast cross sections	
Nucifice		REATION	2200-m/s Maxw.Avg Res.Integ 14-MeV	Fiss.Avg.	
95-Am-242M	253	(n,3n) fission capture	Threshold energy = 12.120 MeV 138.9 mb 6820. b 6480. b 1528. b 2.395 b 1342. b 1321. b 206.8 b 723.0 nb	17.94 μb 1.886 b 106.3 mb	
95-An-243	2954	total elastic inelastic (n.2n) (n.3n) fission (n.4n) capture	86.26 b 78.37 b 5.759 b 7.528 b 7.476 b 2.728 b Threshold energy = 0.042 MeV 42.36 µb Threshold energy = 6.391 MeV 360.4 µb Threshold energy = 11.950 MeV 221.0 mb 228.0 mb 205.2 mb 11.37 b 2.450 b Threshold energy = 18.570 MeV 37.9-12 b	7.853 b 4.798 b 1.577 b 3.728 mb 33.16 μb 1.274 b 2.088 nb 194.5 mb	
96-Cm-242	2961	total elastic inelastic (n,2n) (n,2n) fission capture	32.53 b 30.11 b 5.750 b 11.61 b 11.60 b 2.725 b Threshold energy = 0.042 MeV 318.0 µb Threshold energy = 6.997 MeV 111.5 mb Threshold energy = 13.090 MeV 6.359 mb 5,000 b 4.448 b 11.09 b 2.907 b 15.92 b 14.07 b 116.2 b 3.061 mb	7.818 b 4.954 b 975.4 mb 1.754 mb 4.379 µb 1.798 b 86.97 mb	
96-Cm-243	2962	total elastic inelastic (n.2n) (n.3n) fission (n,4n) capture	753.3 b 570.1 b 5.917 b 9.658 b 9.571 b 2.809 b Threshold energy = 0.042 MeV 26.47 bb Threshold energy = 5.719 MeV 472.4 bb Threshold energy = 12.710 MeV 73.62 bb 612.3 b 545.6 b 1751. b 2.536 bb Threshold energy = 18.830 MeV 131.3 b 115.0 b 404.4 b 17.23 mb	 7.778 b 4.562 b 786.8 mb 15.37 mb 17.69 µb 2.379 b 1.251 nb 34.35 mb 	

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Nualta	MAT	Peachter	Thermal	cross sect:	ions		Fast cross	sections
NUCIIOE	1.141	Reaction	220°-m/s	Maxw.Avg.	Res.Inte	g	14-MeV	Fiss.Avg.
96-C m -244	2963	total elastic inelastic (n.2n) (n.3n) fission capture	22.24 b 6.650 b Threshold e Threshold e Threshold e 1.180 b 14.41 b	20.5! b 6.641 b nergy = 0 nergy = 6 nergy = 12 1.049 b 12.82 b	.043 MeV .828 MeV .550 MeV 18.39 593.5	b	6.026 b 3.273 b 17.51 mb 60.24 mb 39.29 mb 2.636 b 271.4 nb	7.625 b 5.115 b 832.0 mb 576.3 μb 4.776 μb 1.553 b 122.1 mb
96-Cm-245	2964	total elastic inelastic (n.2n) (n.3n) fission capture	2359. b 11.59 b Threshold e Threshold e Threshold e 2001. b 346.3 b	1980. b 11.50 b nergy = 0 nergy = 5 nergy = 12 1680. b 288.3 b	.055 MeV .533 MeV .370 MeV 799.4 107.7	ÞÞ	6.060 b 3.295 b 220.2 mb 176.8 mb 70.66 mb 2.297 b 444.3 nb	7.643 b 4.817 b 890.7 mb 3.080 mt 9.299 µt 1.887 t 45.16 mt

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Appendix 2

Average cross sections

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Appendix 3

Correction of Numerical Data

The corrections of errors found in JENDL-2 (RevO) are described in this appendix. They are classified into two; one is correction commonly done for almost all nuclides and another done case by case.

Common corrections

1. Corrections of Angular Distributions of Secondary Neutrons

In the case of isotropic angular distributions, data are given in the center of mass system for almost all nuclides of JENDL 2 (RevO), because that was considered to be physically correct. However, this representation is not convenient for data applications where the data in the laboratory system are required.

In JENDL 2 Rev1, the angular distribution data of neutrons from multiparticle emission reactions are given in the laboratory system by assuming to be isotropic. In the case where the isotropic distributions in the center of mass system were given for the neutrons from inelastic scattering to continuum levels, the data were transformed into the laboratory system by

 $\frac{d\sigma}{d\Omega_l} = \frac{1 + \gamma^2 \cdot 2\gamma \cos \theta}{1 + \gamma \cos \theta} = \frac{d\sigma}{d\Omega_{l/2}},$ $\gamma = \frac{1}{MR} \sqrt{E_l - E_l + Q},$

where E_t is an incident neutron energy in the center of mass system and Al(R) weight of a target nuclide in the neutron mass unit. A Q value was assumed in this work to be

$$\mathbf{Q} = E_{U,th} + E_U - \mathbf{Z}_{s}$$

where $E_{l,lh}$ is a threshold energy in the center of mass system. At energies near a threshold energy, j is sometimes greater than 1.0. In such cases, isotropic distributions were assumed in the laboratory system. As a result of this calculation, it was found that the transformed distributions in the laboratory systems were almost the same as those in the center of mass system if their mass numbers were greater than 40. Therefore, the distributions calculated from the above equation were used for the nuclides lighter than or equal to calcium. For the others, the isotropic distributions were assumed in the laboratory system.

For some nuclides, the distributions were calculated with optical and statistical model code, $CASTHY^{i_1}$. In such cases, the angular distributions of elastically and inelastically scattered neutrons were represented with Legendre expansion in the center-of-mass system. Only changed for these data were the flag of LCT from '2' to '1', i.e. from the center-of-mass to the laboratory systems, for the neutrons from inelastic scattering to continuum levels.

In the case where Legendre coefficients are given, the number of coefficients should be even according to the ENDF/B-IV rule. However, in JENDL-2 (RevO), the number is sometimes odd. This problem was

automatically solved with CRECTJ5². By this correction, the number of elements of a transformation matrix became incorrect for some nuclides. In Juch cases, the transformation matrix was recalculated and replaced by CRECTJ5.

2. Q values and Threshold Energies

Inconsistency between Q values and threshold energies was found especially in the case of natural element data. The reason is that they were constructed from data of their isotopes and the same Q values and threshold energies as those of isotope data were adopted even if AWR was replaced for natural element.

This inconsistency was removed for almost all cases by recalculating Q values from threshold energies and AWR. It was considered that threshold energies should have been the same as those of JENDL 2 Revoto keep the same curves as JENDL 2 Revo cross sections.

3. Number of Energy Points of The Total Cross Section

In the complete evaluated data file in the ENDF/B format, energy points of the total cross section should cover all energies of partial cross sections. However, in JENDL 2 (RevO), data of many nuclides do satisfy this rule. For such data the total cross section was not reconstructed as a sum of partial cross sections with CRECTJ5 by assuming that all cross section data in JENDL 2 (Revo) were consistent one another. Therefore the results of summation were expected to reproduce completely the same total cross sections as JENDL 2 (RevO). CRECT15 automatically selects energies where partial cross sections are given and which are needed to represent the results of summation within accuracy of 1.0 percent. In the case where inconsistency among cross sections were found and the reason was incorrect compilation of partial cross sections, correction of the partial cross sections was made first, and then the total cross section was recalculated with CRECT.5.

Individual Corrections

In the remaining part of this appendix, the corrections made for each nuclide data are described except the corrections of angular distributions mentioned above.

1. Hydrogen (MAT=2011)

MF-2 was added to give the effective scattering radius which was estimated from the elastic scattering cross section of 20.44 barns at 0.0253 eV.

A transformation matrix of Legendre coefficients of elastically scattered neutrons was calculated and added with CRECTJ5.

2. Deuterium (MAT=2012)

MF 2 was added.

3 Lithium 6 MAT 2031

The Q value and threshold energy of the n.2m cross section were corrected. Q values of the other threshold reactions were replaced with values calculated from threshold energies. The total cross section was recalculated from partial cross sections.

4 Lithium 7 MAT 2032

KF/2 was added. Q values were modified to be consistent with threshold energies. A transformation matrix was replaced with that newly calculated by CKECTJ5.

5 Beryllium 9 MM7 2041

MF-2 was added -Q values were modified. The total cross section was reconstructed from partial cross sections.

6. Boron 10 MAT 2051

The Q volues were related from threshold energies. The total cross section was reconstructed from partial cross sections.

7. Carbon 12 MAT 2061

The Q-values and threshold energies of MT-4.51 and 107 had been inconsistent. The Q-values were modified by keeping the threshold energies

8. Fluorine 19 MAT 2091

The nonclustic scattering cross section was deleted. The total cross section was recalculated from partial cross sections.

The maximum energies of MT 251 and angular distributions of secondary neutrons were not 20 MeV. Sugi did recalculation of these data by using CASTHY In this revision, the calculated data by Sugi³⁰ were adopted for MT 251 and the angular distribution of listically scattered neutrons. For MT's from 51 to 56, isotropic estributions in the center of mass system were assumed.

9. Sodium 23 MAT 2111

The Q values and threshold energies of the n.2n and n.a cross sections were replaced with correct values. The threshold energy of MT-52 was corrected to 2.16917 MeV. The double energy points of 1.5 MeV were found in the total and elastic scattering cross sections. The second energy was changed to 1.505 MeV. After then, the total cross

section was recalculated as a sum of partial cross sections.

10. Aluminium 27 MAT 2131

A transformation matrix of $MT \ge in MF 4$ was recalculated with CRECTJ5, because the immber of elements was not correct

The U-value of energy distribution data for the n.Pn reaction was modified

11 Natural Silicon MAT 2140

The total cross section was reconstructed from partial cross sections. Q values of threshold reactions were corrected to be consistent their threshold energies.

The transformation matrix of MT 2 in $\mbox{MF-4}$ was recalculated with CP4CT \mbox{Ps}

The U values of energy distribution data for the n.2n and inclusive scattering to continuum levels were modified.

12 Natural Calcium MAT 2200

The upper energy of the resonance region for $^{49}C\mu$ was changed to 400 keV

The total cross section was replaced with newly evaluated data from experimental data of Perey et al 4 and Grerjacks⁵⁰ in the energy range trom 400 keV to 5.0 MeV. This evaluation was made by Asami⁶⁰ by means of eye guide method with NDES⁷. The elastic scattering cross section was recalculated by subtracting partial cross sections from the total cross section.

Q values were modified. By modification of Q values, order of inclastic levels. MT 71, 72 and 73 became wrong. Their order was exchanged.

The transformation matrix was replaced with that newly calculated with CEFCTJD

13 Calcium 40 MAT 2201

No modification was made except for angular distribution data.

14. Calcium 42 MAT 2202

No modification was made except for angular distribution data.

15. Calcium 43 MAT 2203

No modification was made except for angular distribution data.

16. Calcium 44 MAT 2204

No modification was made except for angular distribution data.

17. Calcium 46 MAT 2205

MF 2 was added to give an effective scattering radius. The total cross section was calculated as a sum of partial cross sections.

18. Calcium 48 MAT 2206

The upper boundary of the resonance region for ¹⁸Ca was changed to 400 keV.

19 Scandium 45 MAT 2211

The Q values of the n.p and n.a cross sections were modified. The cross section of 0.0 was inserted at 500 keV for the n.p cross section as an effective threshold energy. The total cross section was reconstructed from partial cross sections.

20. Vanadium 51 (MAT 2231)

The Q values and threshold energy of the second inelastic level were modified from 320.1 keV and 326.336 keV to 329 keV and 947.394 keV, respectively. The Q values of the $(n,2n,\ldots,(n,n'\alpha),((n,n'p),((n,p),(n,q),(n,q),(n,q)))$ and (n,t) were adjusted to their threshold energies. The none lastic scattering cross section was deleted from JENDL 2. Background cross sections were corrected below 100 keV to avoid negative values of the elastic and total cross sections. The total and total inelastic cross sections.

21 Natural Chromium MAT 2240

The maximum energy of the elastic scattering cross section was 19 MeV in JENDL 2 -Rev0. Asami⁶ recommended the elastic scattering cross section of 0.947782 barns and the total cross section of 2.16 barns at 20 MeV. These values were adopted for JENDL-2 (Rev1).

All Q values were recalculated from threshold energies. After this modification, the order of the 35 th and 36 th inelastic levels became incorrect. The Q values of these levels were given in JENDL 2 (RevO) as follows.

<u> </u>			500				~				
35	ιn	level	CCC	4≏t	n level		_ب	3.1611	Mev	L _{II} =3.22494	mev
36	th	level	-Cr	7 t	h level	1 (2	3.1617	MeV	Ett. 3.22310	MeV

In JENDL 2 $\,$ Rev1 $_{\odot}$, the order of these two levels was exchanged and Q values were adjusted as follows.

35 th level $\stackrel{52}{\sim}$ Cr 7 th level Q 3.16176 MeV E_{th} 3.22310 MeV 36 th level $\stackrel{59}{\sim}$ Cr 4 th level $_{\rm Q}$ = 3.16357 MeV E_{th} = 3.22494 MeV

The total cross section was reconstructed from partial cross

sections.

22. Chromium 50 (MAT 2241 ;

The value of AWR in MF 2 was corrected. The total cross section was recalculated as a sum of partial cross sections.

23. Chromium 52 MAT 2242

The value of AWR in MF 2 was corrected. The interpolation of the $n_{\rm e}n_{\rm e}$ p cross section was altered.

24. Chromium 53 MAT 2243

The value of AWR in MF 2 was corrected.

25. Chromium 54 MAT 2244

The value of AWR in MF 2 was corrected. The interpolation of the elastic scattering cross section was corrected. The total cross section was recalculated as a sum of partial cross sections.

26. Manganese 55 MAT 2251

In JENDL 2 Rev0, the total cross section calculated with CASTHY was stored. On the other hand, the capture cross section was evaluated on the basis of experimental data and consisted of rather many energy points. To keep consistency among cross sections, the total cross section was recalculated for JENDL 2 Rev1 as a sum of partial cross sections. The differences between the total cross sections of JENDL 2 Rev1 and Rev1 are small enough because the cepture cross section is relatively small.

Angular distributions for the $(n,2n, \dots, n,n'n)$ and (n,n'p) reactions were newly added by assuming to be isotropic in the laboratory system.

27. Natural Iron MAT 2260:

The values of AWR's for ⁵⁰Fe and ⁵⁷Fe in MF-2 were corrected. The Q values were adjusted to threshold energies. The total cross section was recalculated as a sum of partial cross sections.

28. Iron 54 MAT 2261

The effective scattering radius of 7.5 fm was replaced with that of 5.6 fm which was adopted for resonance parameters of natural iron.

Threshold energies were recalculated from Q values and AWR. The interpolation was corrected for the elastic scattering cross section. The elastic scattering and capture cross sections at 250 keV the upper boundary of the resonance region were inserted by assuming to be 3.47

barns and 9.6 mb, respectively. The total and total inelastic scattering cross sections were calculated from partial cross sections.

The angular distribution data for elastically scattered neutrons had not been given at 20 MeV in JENDL 2 Revol. The same distribution as that at 15 MeV was assumed. The angular distribution of neutrons from the in 2n reaction was put by assuming isotropic distribution in the laboratory system. All angular distributions of inelastically scattered neutrons to discrete levels were replaced with isotropic distributions in the center of mass system, and those to continuum levels with isotropic distribution in the laboratory system.

29. Iron 56 MAT 2262

In JENDL 2 Rev0, many background data points were given for the elastic scattering and negative data for the capture cross section. For JENDL 2 Rev1, the negative resonance at 3.75 keV was taken away and the effective scattering radius was changed from 5.4 fm to 6.5 fm in order to reproduce the low energy total and capture cross sections and the total cross section around the 24 keV window. As a result of this correction, better fitting to experimental data of the total cross section was obtained even in high energy region. The calculated capture cross section at 0.0253 eV is somewhat greater than recommended value by Mughabghab et al. However no background correction was applied. As to the elastic scattering cross section, the calculated value is smaller on the contrary. The background data in a very simple form was given for this to reproduce 12.46 barns¹⁰ at 0.0253 eV.

Threshold energies were recalculated from Q-values and AWR. The elastic scattering cross section at 250 keV (the upper boundary of the resonance region) was corrected to be 2.99 barns. The total and total inelastic scattering cross sections were calculated from partial cross sections.

The energy range of MF 4. MT 52 was corrected.

30. Iron 57 MAT 2263

The value of AWR was corrected and threshold energies were recaluculated from Q values and the corrected AWR. The cross section of MT-51 was modified around its threshold energy. Then the total and total inelastic scattering cross sections were reconstructed from partial cross sections.

31. Iron 58 MAT 2264

The background data for the capture cross section were not correct between 100 eV and 100 keV. They were modified by inserting the background capture cross section at 100 keV. The total cross section was recalculated.

The transformation matrix of MT 2 in MF 4 was recalculated with CRECTJ5.

32. Cobalt-59 (MAT=2271)

The resonance parameters were replaced with recommended data by Mughabghab et al.⁸⁰, because the parameters of JENDL 2 RevO consisted of only big resonances of which neutron width was given by Mughabghab et al. and some unreasenable capture widths were also found. For JENDL 2 (Rev1), all resonances recommended by Mughabghab et al. were adepted. The resonances whose ℓ value was not given and whose neutron width was relatively large were assumed to be swave resonances. The capture widths of 0.56 eV and 0.7 eV were assumed for swave and p wave resonances, respectively. The effective scattering radius and para meters of the negative resonance at 500 keV were adjusted to reproduce the thermal clastic scattering and capture cross sections recommended by Mughabghab et al. The changes of parameters were very small as follows.

R	-	6.795 fm	from	6.8 fm
neutron v	width	45.0 eV	from	47.75 eV
capture v	width	0.473 eV	from	0.447 eV
			1	

For the resonance formula, the multilevel Breit $W_{\rm H^{\pm}}$, formula was applied

The calculated total cross section is slightly smaller than experimental data about between 100 eV and 2 keV. No background correction was made for this, because the experimental data have some structure which seems to be contributions from other nuclides.

33 Natural Nickel (MAT 2280)

Q values were recalculated from threshold energies. The total cross section was reconstructed.

Incorrect order of neutron energies of energy distributions of inelastic scattering to continuum levels was corrected.

34 Nickel 58 MAT 2281

The J value of $168.675~{\rm eV}$ resonance was corrected. The value of AWR was corrected

The n.n.'s cross section at 18.5 MeV was slightly modified to reproduce a smooth curve. The total cross section was recalculated as a sum of partial cross sections.

35. Nickel 60 MAT 2282

Reconstruction of the total cross section was made.

36. Nickel-61 MAT-2283

Reconstruction of the total cross section was made.

37. Nickel 62 MAT 2284

Reconstruction of the total cross section was made.

38. Nickel-64 MAT 2285

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Reconstruction of the total cross section was made.

39 Natural Copper (MAT 2290)

Q values were replaced with values calculated from threshold energies.

The energy range of MI 16 of MF 4 and MF 5 was corrected to be consistent with the (n.2n) cross section data.

40. Copper 63 MAT 2291

The total cross section was recalculated

The angular distributions of neutrons from the n.2n and n.n'o reactions were added by assuming to be isotropic in the laboratory system.

Copper 65 MAT 2292 Å1

The angular distributions of neutrons from the (n,2n) and $(n,n'\alpha)$ reactions were added by assuming to be isotropic in the laboratory system

The energy range of MT 16 of MF 5 was corrected.

Niobium 93 MAT 2411 42.

The total cross section was reconstructed from partial cross sections

The value of AWR of MT 16 and MT 17 in MF 4 was modified. Energy ranges of MT 16 and MT 17 in MF 5 were corrected.

43. Natural Molybdenum MAT 2420

The J value of the 4.622 keV resonance of ⁵⁴Mo was corrected. The effective threshold energies were set at 144.7 keV and 1.0 MeV for the (n,p) and (n,α) cross sections, respectively. The Q value of the (n,3n) cross section was corrected. Other Q values were also corrected from their threshold energies. The total cross section was reconstructed from partial cross sections.

44. Molybdenum 92 (MAT-2421)

The effective threshold energies were set at 1.0 MeV for the (n,p) and n, α cross sections. The total cross section was recalculated.

Molybdenum 94 MAT 2022 45.

The J value of the 4.622-keV resonance was corrected. The cross section of 0.0 was inserted at 1.0 MeV for the (n,α) cross section as an effective threshold energy. The Q value of the (n,3n) cross section was modified. The total cross section was reconstructed from partial cross sections.

46. Molybdenum 95 (MAT 2423)

The cross section of 0.0 was inserted at 1.0 MeV for the (n,α) cross section as an effective threshold energy. The total cross section was reconstructed from partial cross sections.

47. Mollybdenum 96 MAT 2424 :

The cross section of 0.0 was inserted at 1.0 MeV for the (n,α) cross section as an effective threshold energy. The total cross section was reconstructed from partial cross sections.

48. Molybdenum 97 MAT 2425

The cross section of 0.0 was inserted at 1.0 MeV for the (n,α) cross section as an effective threshold energy. The total cross section was reconstructed from partial cross sections.

49. Molybdenum 98 MAT 2426

The cross section of 0.0 was inserted at 1.0 MeV for the (n,α) cross section as an effective threshold energy. The total cross section was reconstructed from partial cross sections.

50. Molybdenum 100 MAT 2427

The cross section of 0.0 was inserted at 1.0 MeV for the (n,α) cross section as an effective threshold energy. The Q value of the n.p. cross section was corrected. The total cross section was reconstructed from partial cross sections.

51. Hafnium 174 (MAT 2721)

The total cross section was recalculated as a sum of partial cross sections.

52. Hafnium 176 (MAT 2722

The total cross section was recalculated as a sum of partial cross sections.

53. Hafnium 177 (MAT 2723)

The total cross section was recalculated as a sum of partial cross

sections.

54. Hafnium 178 MAT 2724

The clastic scattering cross section in the thermal region did not reproduce the value of 5.0 barns at 0.0253 eV recommended by the evaluator of this muchide. This was corrected by replacing the data with newly evaluated values. The background cross section was also added to the elastic scattering cross section to connect cross sections calculated from resonance parameters to the pointwise data below 0.5 eV. The total cross section was recalculated as a sum of partial cross sections.

55 Hafnium 179 MAT 2725

The total cross section was recalculated as a sum of partial cross sections

56. Hafnium 160 MAT 2726

The total cross section was recalculated as a sum of partial cross sections.

57 Tantatum 181 -MAT 2731

Large inconsistency was found between the total and partial cross sections in the energy range from 7 to 50 keV where is in the resonance region. The total cross section was recalculated as a sum of partial cross sections.

58 Natural Lead MAT 2820

The J values of 527.4 keV and 721.4 keV resonances of ²⁰⁰Pb were incorrect. They were altered from p wave to d wave resonances.

The total cross section was replaced with the eye guided values based on experimental data in the energy range from 500 keV to 15 MeV. The elastic scattering cross section was modified to keep consistency among cross sections.

Inconsistencies of energy ranges among MF-3, MF-4 and MF-5 were corrected.

59. Lead 204 MAT 2821

The interpolation tables of the total, elastic scattering and capture cross sections were corrected. The capture cross section at 50 keV background cross section was modified. After the total cross section was reconstructed from partial cross sections.

The first energy of secondary neutrons in $MF{=}\,5.$ MI=22 was connected.

60. Lead 206 (MAT 2822)

The total cross section was recalculated as a sum of partial cross sections.

The first energies of secondary neutrons of MT 22 and MT 91 in MF 5 were corrected.

61. Lead 207 (MAT 2823)

The interpolation of the elastic scattering cross section was corrected. The elastic scattering cross section was inserted at 500 keV where is an upper boundary of the resonance region. The total cross section was recalculated as a sum of partial cross sections,

The energy range of MT 22 in MF 5 was modified to be the same as that of the ninio cross section.

62 Lead 208 MAT 2824

The J values of 527.4 keV and 721.4 keV resonances were incorrect. They were changed from p wave to d wave resonances.

The total cross section was recalculated as a sum of partial cross sections.

The energy range of MT 22 in MF 5 was modified to be the same as that of the nin's cross section.

63 Thorium 228 (MAT 2901)

The resonance formula was changed to the multilevel Breit-Wigner formula, because negative elastic scattering cross sections were found from the JENDL 2 RevO:.

The cross section shape of the n.2n and (n.3n) cross sections were corrected to be the same curve as recommendation by Ohsawa who was an evaluator of this nuclide. Interpolat in tables were modified for the elastic and capture cross sections. At 2 keV, two values were given for the elastic scattering and capture cross sections in JENDL 2 (Rev0); one was the evaluated value by Ohsawa and another calculated with CASTHY. For JENDL 2 (Rev1), Ohsawa's evaluated value was adopted at 2 keV to give a smooth curve for each cross section. The total and total inelastic scattering cross sections were reconstructed from particle cross sections.

The fission spectrum was added by assuming the Maxwellian spectrum determined from Z^2 A systematics by Smith et $a1^{(r)}$.

64. Thorium 230 (MAT 2902)

The J values of resonance parameters were replaced with J 1 2 for all resonances. The effective scattering radius of 19.25 fm was corrected to be 10.925 fm. The resonance formula was changed to the multilevel Breit Wigner formula to avoid negative elastic scattering cross sections. The background cross section of the capture cross section was recalculated as

 $\sigma_{(\mu,\nu)}(E) = 0.257425 \sqrt{E(eV)}$

to reproduce the recommended 0.0253 eV cross section by Ohsawa. The elastic scattering cross section calculated at 0.0253 eV is 9.77 barns and that recommended by Ohsawa is 9.403 brans. However no background cross section was applied, because no experimental data existed and the difference was not so large.

The interpolation tables were changed to be log log between -664.26 eV and 4 keV. The cross sections at 4 keV were slightly modified so as to join to the values calculated with CASTHY above 4 keV. The curves of the -n.2n and (n.3n) cross sections were corrected. The total cross section was finally recalculated from partial cross sections.

The fission spectrum was added by assuming the Maxwellian spectrum determined from Z^2 A systematics by Smith et al.

65 Thoritum 232 MAT 2903

The total cross section was recalculated from partial cross sections

66. Thorium 233 MAT 2904

Interpolation tables were modified for the elastic and capture cross sections. The cross sections at 4 keV were modified so as to join to the values calculated with CAGHHY above 4 keV. The curves of the

n.2n and an.3n eross sections were corrected. The total and total inelastic scattering cross section was recalculated from partial cross sections

The fission spectrum was added by assuming the Maxwellian spectrum determined from Z^2 A systematics by Smith et al.

67. Thorsum 234 MAT 2905

Interpolation tables were modified for the elastic and capture cross sections. The cross sections at 4 keV were modified so as to join to the values calculated with CASTHY above 4 keV. The curves of the (n,2n) and (n,3n) cross sections were corrected. The total and total inelastic scattering cross section was recalculated from partial cross sections.

The fission spectrum was added by assuming the Maxwellian spectrum determined from Z^2 A systematics by Smith et al.

68. Protactinium 233 (MAT 2911)

The energy boundary of the resolved and unresolved resonances were inconsistent in JENDL 2 Rev0.. The boundary of two ranges was set to 17.0 eV for JENDL 2 (Rev1).

Inconsistent energy ranges among MF's were corrected. The energy distributions of the n.2n $_{\odot}$ (n.3n) and inelastic scattering to continuum levels were replaced with the results of EVAPSPEC¹⁰). The fission spectrum was added by assuming the Maxwellian spectrum determined from Z^2/A systematics by Smith et al.

69. Uranium 233 (MAT 2921)

The energy of 7.47 MeV in the total number of neutrons and number of prompt neutrons per fission was changed to 7.5 MeV.

The nonelastic scattering cross section given in JENDL 2 (RevO) was deleted. The total and total inelastic scattering cross sections were reconstructed from partial cross sections.

The energy range of angular distributions of neutrons from fiscion was corrected.

70. Uranium 234 - MAT 2922

The resonance formula was altered freat the single level Breit. Wigner to the multilevel Breit Wigner formula. The background cross sections were replaced with zero values.

The Q values of MT 19, 20 and 21 were set to be the same as that of MT 18 $\,$ MF 5 data for MT 19, 20 and 21 were taken from those of MT 18.

Energy ranges of angular distribution data were corrected.

71. Uranium 235 MAT 2923

Interpolation of fission yield data was changed to linear linear.

72. Uranium 236 MAT 2924

No modification was made except correction of anguler distribution data

73 Uranium 238 MAT 2925

Interpolation of fission yield data was changed to linear linear. Negative values were found in the angular distribution of elastically scattered neutrons for the incident neutron energy of 20 MeV. The same distribution as 18 MeV neutrons was used for 20 MeV neutrons to avoid this problem.

74. Neptunium 237 (MAT 2931)

The total cross section was reconstructed from partial cross sections. The energy ranges of energy distributions of neutrons from the (n,2n) and (n,3n) reactions were modified to be the same as their cross section data.

75. Neptunium 239 MAT 2932

The energy of 10^{-5} eV was inserted for the fission cross section. The U value of fission spectrum was changed to -20 MeV. The energy spectra of neutrons from (n,2n), (n,3n) and inelastic scattering to continuum levels were replaced with the results of $EVAPSPEC^{10}$.

76. Plutonium 236 HAT 2941

The Q values of MT 19. 20 and 21 were set to be equal to that of MT 18. The threshold energies of MT 20 and 21 were sodified to be 3 MeV and 11 MeV, respectively, and their cross sections around threshold energies were corrected. The total cross section was recalculated from partial cross sections.

The energy distribution data of MT 19, 20 and 21 were taken from these of MT 18 $\,$

77 Plutonium 238 MAT 2942

The neutron widths of the 2.9 eV and 32.2 eV resonances were modified. The fission widths of the 0.4 eV and 2.9 eV resonances were adjusted to reproduce the thermal fission cross section of 16.5 barns recommended in ENL 325 3rd edition. The resonance formula was changed to the multilevel Prest Wigner formula in order to avoid negative cross sections.

The Q values and threshold energies of the (n,2n) and (n,3n) cross sections were corrected. The n,2n cross section at 10 MeV was replaced with correct one. The total cross section was constructed from partial cross sections.

The angular distribution of elastically scattered neutrons by 20 MeV neutrons had negative values. The same distribution as 18 MeV neutrons was assamed at 20 MeV.

78. Plutonium 239 MAT 2943

The interpolation of fission yield data was changed to linear linear. The Q value of the total inelastic scattering cross section was slightly changed from 7.863 keV to 7.86 keV.

79. Plutonium 240 MAT 2944

The Q values and threshold energies of all inelastic scattering cross sections were inconsistent in JENDL 2 (RevO). The threshold energies were recalculated from their Q values. The total and total inelastic scattering cross sections were recalculated from partial cross sections.

The energy range of angular distribution data for the $(r_{1},2n)$ and $n_{1}Sn$ - reactions were corrected,

The energy distribution data were calculated with EVAPSPEC.

80. Plutonium 241 MAI 945

The Q values of the $n_1 \ge n_2 \ge n_3 \ge n_4 \ge n_4 \ge n_4 \ge n_5 = n_5 \ge n_5 = n_5 \ge n_5 \ge n_5 = n_5 = n_5 \ge n_5 \ge n_5 \ge n_5 \ge n_5 \ge n_5 \ge n_5 = n_5 = n_5 = n_5 = n_5 = n_5 \ge n_5 \ge n_5 \ge n_5 \ge n_5 = n_5 = n_5 \ge n_5 = n_$

81. Plutonium 242 MAT 2946

The threshold energies were slightly corrected to satisfy the relation between Q values and threshold energies. The total and total inelastic scattering cross sections were calculated as a sum of partial cross sections.

82 Americium 241 MAT 2051

The figure yield data were renormalized to 2.0

The Q values were replaced with those calculated from threshold energies. The total cross section was reconstructed from partial cross sections

The energy ranges of ME 4, ME 18 and ME 5, ME 16, 17 and 18 were corrected to be consistent with cross section data

The evaporation spectra in MP 5 were recalculated with EVAPSPEC.

83 Americium 242 MAT 2952

The angular distribution of electrically scattered neutrons by 20 MeV neutrons had negative values. The same distribution as 18 MeV neutrons was assumed at 20 MeV.

84 Americaum 242m MAT 2953

The elastic and capture cross scutions around 3.5 eV vere modified. The total cross section was reconstructed from partial cross sections.

The angular distribution of elastically scattered neutrons by FO MeV neutrons had negative values. The same distribution as 18 MeV neutrons was assumed at 20 MeV.

The energy range of MF 5, MP 91 was corrected.

65. Americium 243 MAT 2954

The fission yield data were renormalized to 2.0.

The total cross section was reconstructed from partial cross sections.

The angular distribution of elastically scattered neutrons by 20 MeV neutrons had negative values. The same distribution as 14 MeV neutrons was assumed at 20 MeV. The energy range of MF 4, MT-18 was corrected to be consistent with cross section data.

86. Curium 242 MAT 2961

The angular distribution of elastically scattered neutrons at 20 MeV had negative values. The same distribution as 18 MeV neutrons was assumed at 20 MeV.

87. Curium-243 (MAT-2962)

The capture cross section of JENDL 2 (RevO) was adopted by mistake to be the same as elastic scattering cross section in the energy range from 27 eV to 1 keV. It was replaced with the cross sections correctly estimated with the method described in Ref. 11. The total cross section was reconstructed from partial cross sections.

The angular distribution of elastically scattered neutrons at 20 MeV had negative values. The same distribution as 18 MeV neutrons was assumed at 20 MeV.

88. Curium 244 MAT 2963

The resonance formula was replaced with the multilevel Breit-Wigner formula to avoid negative cross sections. The background cross sections given in JENDL 2 Rev0 were taken away.

The total cross section was recalculated as a sum of partial cross sections.

89. Curium 245 MAT 2964

The Q value of the n.2n was corrected.

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