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November 1977

IAEA NUCLEAR DATA SECTION, KÄRNTNER RING 11, A-1010 VIENNA

Reproduced by the IAEA in Austria November 1977 77-10321

INSTITUTE OF NUCLEAR RESEARCH

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EVALUATION OF THE 58 Ni(n,2n) 57 Ni REACTION CROSS SECTIONS

EWALUACJA PRZEKROJÓW CZYNNYCH REAKCJI ⁵⁸Ni(n,2n)⁵⁷Ni

ОЦЕНКА ЭФФЕКТИВНЫХ СЕЧЕНИЙ РЕАКЦИИ ⁵⁸Ni(n,2n)⁵⁷Ni

Lesław Adamski Michał Herman Andrzej Marcinkowski

Abstract

The excitation curve for the 58 Ni(n,2n) 57 Ni reaction has been evaluated on the basis of 15 experimental data sets or single-energy cross sections. The recommended cross sections are given in 0.1 MeV energy steps, from the threshold energy to 28 MeV. The estimated accuracy is 11.4% below 14 MeV, 8.7% between 14 MeV and 16 MeV, and worse at higher energies.

Streszczenie

Ewaluowano krzywą wzbudzenia reakcji 58 Ni(n,2n) 57 Ni. Uwzględniono wyniki 15-stu prac eksperymentalnych. Zalecane przekroje czynne podano w tabeli co 0.1 MeV, poczynając od progu, aż do 28 MeV. Oszacowane dokładności przekrojów czynnych wynoszą 11.4% dla energii neutronów niższych od 14 MeV i 8.7% dla energii od 14 MeV do 16 MeV. Powyżej 16 MeV dokładność jest gorsza.

Аннотация

Произведено оценку эффективных сечений реакции ${}^{58}{}_{\rm Ni}(n,2n){}^{57}{}_{\rm Ni}$ для энергии нейтронов от порога до 28 МэВ. Точность оцененных сечений составляет 11.4% для энергии нейтронов ниже 14 МэВ и 8.7% для энергии от 14 МэВ до 16 МэВ. При высших энергиях получена точность худшая.

I. INTRODUCTION

The aim of this work is to evaluate the excitation function of the 58 Ni(n, 2n) 57 Ni reaction for neutron energies ranging from the threshold energy equal 12.4154 MeV [Gue 75] up to 28 MeV. The cross sections of the 58 Ni(n, 2n) 57 Ni reaction have been requested in WRENDA 76/77 by A.Michaudon (Ref. No 250) for fission reactor development, as well as by D.Breton (Ref. No 1409), Y.Seki (Ref. No 1410) and G.D.Mc Cracken (Ref. No 1411) for fusion reactor purposes. The 58 Ni(n, 2n) 57 Ni reaction is commonly used for reactor dosimetry applications, mainly for neutron spectra unfolding by multiple-foil activation technique. The energy range of special interest /90% response in a fission neutron spectrum/ is between 13 and 17 MeV.

We have compiled 23 publications (Fig. 2 and 3) concerning the 5^{8} Ni(n,2n) 5^{7} Ni reaction, the last of which was published in 1976. The experimental cross sections are widely scattered at higher neutron energies, ranging between the lowest values measured by Jeronymo et al. and those obtained by Bayhurst et al. in 1975. The discrepancy between the two data sets reaches a factor 2.2 despite of both have been obtained in careful experiments. As a result of evaluation eight data sets or single data have been omitted in the final consideration. All the accepted data sets or single data values have been considered equivalent. This is because we have concluded that the differences in the errors attached by the authors are dim or insignificant.

The recommended data set presented in chapter V has been compared with the results of previous evaluations and with the statistical model estimates accounting for the \tilde{X} -decay of unbound states and precompound emission.

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II. GENERAL DATA ON THE REACTION STUDIED AND THE COM-PETITIVE REACTIONS

The Q values, threshold energies and the cross sections at 14 MeV neutron energy for the 58 Ni(n,2n) 57 Ni reaction and the competing reactions are listed in Table 1.

The data in the Table 1 were taken from the evaluation of Guenther et al. [Gue 75]. Exceptionally the 58 Ni(n,2n) 57 Ni reaction cross section as evaluated in the present work is given.

Reaction	G mb	Q MeV	Threshold MeV
⁵⁸ Ni(n,2n) ⁵⁷ Ni	22.9	-12.203	12.4154
58Ni(n, np+pn) ⁵⁷ Co	550 [×]	- 8.1772	8.319
58Ni(n,p) 58 Co	337	0.3947	
58 Ni(n, ∞) ⁵⁵ Fe	113	2.8902	
58 Ni(n,d) ⁵⁷ Co	17.7	- 5.952	6.056
⁵⁸ Ni(n, 3n) ⁵⁶ Ni		-22.471	22.862

TABLE 1

 $^{\times}$ This value corresponds to 14.5 MeV.

III. METHOD OF MEASUREMENT AND THE RECOMMENDED STANDARDS

All experimental data collected in the present work have been obtained with aid of the activation method. The ⁵⁷Ni produced in the 58 Ni(n,2n)⁵⁷Ni reaction decays with a half-life 36.0 h via β^+ emission and electron capture. Branchings are known to the 1.37 MeV, 1.49 MeV and 1.89 MeV levels of 57 Co. The possible weak transition to the 1.75 MeV level of 57 Co has been neglected in the analysis of the measured activities.



Fig. 1. Decay scheme of ⁵⁷Ni [Led 68] .

Most often β^+ or the 511 keV annihilation δ^- -quanta are counted in order to determine the 57 Ni activity. Some authors use also counting of the 1.37 MeV δ^- -rays. When β^+ or annihilation quanta are counted, the activity belonging to the decay of 57 Ni has to be separated from the activities due to the decay of other reaction products. This is done by chemical separation or decay time analysis.

The following decay data have been adopted as recommended ' standards in the present analysis:

- half-life of 57 Ni ground state $T_{1/2} = 36.0$ h,
- number of emitted β^+ per decay of the ⁵⁷Ni ground state I $\beta^+ = 0.46$,
- number of 511 keV annihilation quanta per decay of the 57 Ni ground state I $\frac{7}{5}$ (511 keV) = 0.934,
- number of emitted 1370 keV \mathcal{F} -rays per decay of the ⁵⁷Ni ground state $I_{\mathcal{F}}^{\mathcal{F}}$ (1370 keV) = 0.86.

Three different methods are commonly applied in the determination of the neutron flux:

- the associated alpha-particle method is based on counting the recoiled alpha particles accompanying each neutron emitted in the $T(d,n)^4$ He reaction. This accurate and reliable method is limited to neutron energies close to 14 MeV only.
- in the proton recoil telescope the protons recoiled from a polyethylene foil bombarded with neutrons are counted in a thin scintillation or semiconductor detector. This method requires the knowledge of the

H(n,n)H reaction cross sections. In the present evaluation these cross sections were taken from ENDF/B-IV (MAT.1269).

- the reference reaction method makes use of the radioactivity pro-

duced in a monitoring reaction of known cross section . The accuracy of the reference reaction method is limited by the error of the monitoring reaction cross section. On the other hand the uncertainties of the attenuation of the neutron flux in the sample and the target-sample geometry can be significantly reduced within this method. Wherever possible we have accepted the ENDF/B recommended cross sections for the monitoring reactions. These data are both new and follow best the mean behaviour of the remaining evaluated curves /UKNDL, KEDAK, [Lap 75] /. The monitoring reactions used in the experiments entering this evaluation and the decay constants accepted as standards are following:

 $\frac{27}{\text{Al}(n, \alpha)^{24} \text{Na}} - \text{the}^{24} \text{Na ground state decays by } \text{S}^{-} \text{ emission with} \\ \text{the half-life } T_{1/2}^{+} = 15.0 \text{ h} \text{ [Led 68]} \text{ to excited states of } ^{24} \text{Mg. The } \text{S}^{-} \\ \text{or the 1369 keV} \quad \text{S}^{-} \text{ rays from the decay of the 1369 keV excited} \\ \text{state in } ^{24} \text{Mg have been used in determining of the activity of } ^{24} \text{Na} \\ \text{ground state. For the purpose of the present work the cross sections} \\ \text{given in ENDF/B-IV (MAT.6193) [You 75]} have been accepted. \\ \text{Taking into account the precise measurement of Vonach et al. [Von 70]} \\ \text{we have estimated the accuracy of these cross sections within 4.2%.} \\ \frac{238}{\text{U}(n, f)} - \text{ for neutron energies surpassing 14 MeV the cross sections} \\ \text{of the } ^{238} \text{U}(n, f) \text{ evaluated by various authors differ by less then 3\%.} \\ \text{We have accepted the ENDF/B-IV (MAT.6262) [Pai 74] data as a recommended standard.} \\ \end{array}$

 $\frac{5^{6} \text{Fe}(n,p)^{56} \text{Mn}}{\text{excited states of}^{56} \text{Fe with a half-life } T_{1/2}^{r} = 2.576 \text{ h} \text{[Led 68]. Among the}}$

5 - rays accompanying the deexcitation of the ⁵⁶Fe levels the 847 keV transition is most prominent and most often measured. Its intensity equals 0.989 per decay of ⁵⁶Mn ground state. The excitation curve for this reaction has been evaluated by Kanda et al. [Kan 72] and by Dudey and Kennerley in ENDF/B-III (MAT.6410) [Dud 75]. These evaluations differ by less then 2% (Fig. 5). We have used the ENDF/B-III data as standard.

 $\frac{65}{64}$ Cu(n, 2n)⁶⁴Cu - the ⁶⁵Cu decays with a 19% branch via β^+ to the ⁶⁴Ni ground state. A 38% branch of β^- leads to the ⁶⁴Zn ground state. The half-life of this decay equals 12.8 h. We have assumed that the discrepancies between different evaluations, amounting 3%, define the accuracy of the evaluated standard cross sections. Again the ENDF/B4V (MAT.6412) data [Ros 75] have been used in the present work. $\frac{63}{2}$ Cu(n, 2n)⁶²Cu - the ⁶²Cu produced in this reaction decays with 97⁺1% branch via β^+ emission to ⁶²Ni levels. The half-life T_{1/2} equals 9.8 min. Considerable deviations exist between the two available evaluations. We have arbitrarily chosen the UKNDL (DFN 250) data which seem to be more consistent with the analyzed cross sections for the ⁵⁸Ni(n, 2n)⁵⁷Ni reaction than the UKNDL (DFN 681) data.

IV. COMPILATION AND EVALUATION

Out of the 23 compiled papers (Fig. 2 and 3), after thorough examination, 8 works have been rejected. These are the works of: - S.Chojnacki et al. [Cho 65], who obtained an excitation curve rising

- much steeper than all the other data.
- W.G.Cross et al. [Cro 62]. Only the abstract of this work was available

and the 40 $\stackrel{+}{-}$ 5 mb value for the cross section at 14.5 MeV exceeds the average by more than three standard deviations.

- R.W.Fink and Wen-Deh Lu [Fin 70]. Only the abstract of this work . was available and the 38 ⁺ 5 mb value for the cross section exceeds the average by four standard deviations.
- D.J.Hughes and R.B.Schwartz [Hug 58]. The data in [Hug 58] are given in a graphical form only. No information allowing a renormalization was available. The cross section values exceed the average by about five standard deviations:

En	ଁଟ
MeV	mb
13.5	25.4
15.4	60.5
17.1	85.0
18.2	95.5

- J.M.F.Jeronymo et al. [Jer 63]. This excitation curve deviates from the other results in the whole energy range, being about twice lower. No explanation for this fact has been found;

E _n MeV	6 mb
12.55 + 0.2	1 - 1
13.55 + 0.2	8 + 2
14.90 - 0.2	19 - 3
16.50 - 0.2	24 - 3
18.15 - 0.2	34 - 4
19.60 + 0.2	39 + 4
20.60 + 0.2	45 + 4
21.00 + 0.2	38 - 4

- E.B.Paul and R.L.Clarke [Pau 53]. The result 40.6 ⁺ 12 mb at 14.5 MeV exceeds the average by four standard deviations.
- I.L.Preiss and R.W.Fink [Prf 60]. The result 52 ⁺ 5 mb at 14.8 MeV is extremely high.
- K.H.Purser and E.W.Titterton [Pur 59]. The result 38 8 mb at 14.1 MeV is higher than the average by seven standard deviations.

For the 15 accepted data sets the evaluation forms have been prepared. They contain the most important information about the experiment and the standards adopted in the analysis of the measured activities or the counted emission rates. When deviations were met between the standards used by the authors and the recommended standards listed in chapter III an appropriate normalization has been performed. For differing half-lifes approximate linear extrapolation was applied. In some cases it was assumed that the authors used the right standards. The relative errors attached by the authors have been conserved.

AUTHOR: L. A. Rayburn

REFERENCE: Physical Review 122 /1961/ 168, [Ray 61].

LABORATORY: Argonne National Laboratory, Argonne, Illinois.

QUANTITY MEASURED: Cross section for 58 Ni(n, 2n)⁵⁷Ni at neutron energy 14.4 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ± 0.3 MeV, cross section $\pm 1.5\%$. STANDARDS: $T_{1/2}(57Ni) = 43.7$ h, $I_{\chi}(511 \text{ keV}) = 0.938$ per decay of 57Ni, ${}^{63}Cu(n,2n){}^{62}Cu$ $\mathfrak{S} = 503$ mb $\pm 7.3\%$ at 14.4 MeV, $I_{\chi}(511 \text{ keV}) = 1.864$ per decay of ${}^{62}Cu$.

NEUTRON SOURCE: 120 kV Cockcroft-Walton accelerator, $T(d,n)^4$ He reaction on thick Zr-T target.

SAMPLE: Sample in cylindrical Lucite container 1 inch in diameter. FLUX DETERMINATION: Associated particle method.

EXPERIMENTAL ARRANGEMENT: Sample placed 3 cm and 10 cm from target at angle 45° , irradiation not longer than 7 h.

ACTIVITY MEASUREMENT: 511 keV annihilation \mathcal{F} -rays measured with two 1.5 x 1.5 inch Nal(Tl) spectrometers in coincidence, absolute efficiency determined by activity measurement from 63 Cu(n,2n) 62 Cu reaction with known cross section.

CORRECTIONS: For selfabsorption by extrapolating to zero-thickness sample.

ERRORS: All known contributions with the uncertainty of 63 Cu(n,2n) 62 Cu cross section included.

EVALUATORS'COMMENT: No information given on numerical value of $T_{1/2}$ (62 Cu) used.

NORMALIZATION-RESULT: Rayburn's result normalized according to recommended $T_{1/2}^{r}(5^{7}Ni)$, $I_{\chi}^{r}(511 \text{ keV})$ per decay of $5^{7}Ni$, and $^{63}Cu(n,2n)^{62}Cu$ cross section.

Er Mev	<u>ר</u> ק	∆E n [MeV]	ଟ୍ୟ [mb]	62 [mb]	63 [mb]	RB	RT	64 [mb]	∆54 [%]
14.4	4	0.3	34.2	503	525	1.004	0.824	29.5	7.5
5_{4} - cross section for 58 Ni $(n, 2n)^{57}$ Ni given by author, 5_{2} - value of 63 Cu $(n, 2n)^{62}$ Cu cross section used by author,									
бъ	63 - recommended value of 63 Cu(n,2n) 62 Cu cross section,								
R _B R _B	$R_{\rm B} = - \text{normalization factor:}$ $R_{\rm D} = 1 \sqrt{(511 \text{ keV})} / 1 \sqrt{(511 \text{ keV})} = 0.938 / 0.934 = 1.004.$								
R _T	R _T = normalization factor:								
^R T Ծ	$R_{T} = T_{1/2}^{r} ({}^{57}\text{Ni}) / T_{1/2} ({}^{57}\text{Ni}) = 36.0/43.7 = 0.824,$ S ₄ = adopted ${}^{58}\text{Ni}(n, 2n)^{57}\text{Ni}$ cross section								
Δ64	-	cross s	section	error	•		- - ,	٨.	

AUTHORS: R.J. Prestwood, B.P. Bayhurst

REFERENCE: Physical Review 121 /1961/1438, [Pre 61].

LABORATORY: Los Alamos Scientific Laboratory, University of California, Los Alamos, New Mexico.

QUANTITY MEASURED: Cross sections of 58 Ni(n, 2n)⁵⁷Ni reaction for 9 neutron energies from 13.5 to 19.8 MeV.

METHOD: Activation.

ACCURACY: Neutron energy spread $\frac{1}{2}$ 100 to 430 keV, cross section $\frac{1}{2}$ 5 to 8%.

STANDARDS: $T_{1/2} \begin{pmatrix} 5^7 \text{Ni} \end{pmatrix} = 36.0 \text{ h}, \text{ I}_{\beta} = 0.469 \text{ per decay of} \\ \overset{57}{\text{Ni}} [\text{Str 58}], \qquad U(n,f) \text{ cross section } \mathfrak{S} = 1.38 \text{ b} [\text{Bay 61}], \\ \overset{27}{\text{Al}(n, \mathcal{L})}^{24} \text{Na cross sections.}$

NEUTRON SOURCE: Cockcroft-Walton accelerator, $T(d,n)^4$ He reaction on Zr-T target for neutron energies from 13.5 to 14.8 MeV; 6 MeV Van de Graaff accelerator, $T(d,n)^4$ He reaction on gas target for neutron energies from 16.5 to 18 MeV.

SAMPLES: Circular metal foils 9.5 mm in diameter.

FLUX DETERMINATION: In case of solid Zr-T target measured activity of ²⁴Na produced in ²⁷Al(n,∞)²⁴Na reaction and counted associated ∞ - particles; in case of gas target ²³⁸U(n,f) counter checked by radiochemical analysis of fission product ⁹⁹Mo in ²³⁸U foils irradiated at 7 and 8 MeV. EXPERIMENTAL ARRANGEMENT:

- Cockcroft-Walton accelerator: samples positioned on thin plastic hemispherical shell of 5 cm radius at angles from 0° to 165° , aliminium monitor foils placed with each sample, additional aluminium foil placed at 90° and at 10.0^{+} 0.1 cm distance used as primary monitor.
- Van de Graaff accelerator: calibrated fission counter in conjunction with ²³⁸U foil placed behind sample stack on beam axis.

ACTIVITY MEASUREMENT: β^{+} counted in proportional counter. CORRECTIONS: In case of gas target stacks of ⁵⁸Ni foils alternated with ²³⁸U placed on beam axis served for determining of flux attenuation, fission fragment loss accounted for.

ERRORS: Errors account for consistency of experimental data with theoretical excitation function based on statistical model, absolute errors not included.

NORMALIZATION - RESULTS: Authors' cross section referred presumably to cross section given in ref. [Kni 58] and ref. [Bay 61a] have been renormalized according to recommended cross section of reference reaction ${}^{27}\text{Al}(n,\infty){}^{24}\text{Na}$ and ${}^{238}\text{U}(n,f)$, and 1 per decay of ${}^{57}\text{Ni}$ renormalized to recommended value.

En [MeV]	ΔEn [MeV]	ମ୍କ [mb]	62 [mb]	бз [mb]	RB	64 [mb]	∆64 [mb]
13.52	0.15	13.9	130.6	128	1.004	13.7	0.7
13.88	0.10	21.4	128.5	126	1.004	21.1	1.1
14.09	0.10	23.5	128.3	123.5	1.004	26.7	1.7
14.31	0.13	31.1	123.6	121	1.004	30.5	1.6
14.50	0.20	34.3	119.8	119	1.004	34.2	1.7
14.81	0.31	39.3	114.1	115.5	1.004	89.8	2.0
16.50	0.30	53.3	1380	1350	1.004	52.3	4.2
17.95	0.32	67.6	1380	1320	1.004	65.0	3.3
19.76	0.43	77.4	1380	1410	1.004	79.4	4.0

- $\mathbf{5}_{\mathbf{1}}$ cross section given by authors,
- $\mathbf{5}_{2}$ cross section of 27 Al $(n, \mathbf{x})^{24}$ Na and 238 U(n, f) used by authors,
- $\mathbf{5}_{\mathbf{3}}$ recommended cross section of $^{27}\mathrm{Al}(n,\mathbf{x})$ and $^{238}\mathrm{U}(n,f)$,
- R_B normalization factor:
- $R_{B} = I_{\beta} / I_{\beta}^{r} = 0.469/0.467 = 1.004,$
- $\mathbf{5}_{4}$ adopted cross section: $\mathbf{5}_{4} = \mathbf{R}_{B}(\mathbf{5}_{3}/\mathbf{5}_{2})\mathbf{5}_{4}$, $\Delta \mathbf{5}_{4}$ - cross section error: $\Delta \mathbf{5}_{4} = \mathbf{R}_{B}(\mathbf{5}_{3}/\mathbf{5}_{2})\Delta \mathbf{5}_{4}$.

AUTHORS: R.N.Glover and E.Weigold

REFERENCE: Nuclear Physics 29 /1962/ 309, Glo 62.

LABORATORY: Dept. of Nuclear Physics, Australian National University, Canberra, A.C.T.

QUANTITY MEASURED: Cross section for 58 Ni(n, 2n) 57 Ni for 9 energy values from 13.9 to 14.9 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺100 to 300 keV, cross section ⁺8%.

STANDARDS: $T_{1/2}({}^{57}Ni) = 37 h$, $I_{5}(511 \text{ keV}) = 0.928 \text{ per decay}$ of ${}^{57}Ni$, ${}^{65}Cu(n,2n){}^{64}Cu \text{ cross sections.}$

NEUTRON SOURCE: 250 keV Cockcroft-Walton accelerator, $T(d,n)^4$ He reaction on Ti-T target.

SAMPLE: Circular foils of 1 cm^2 area and of various thickness.

FLUX DETERMINATION: Associated ∞ -particle method for flux determination and monitoring, ${}^{65}Cu(n,2n){}^{64}Cu$ reaction used to determine neutron angular distribution.

EXPERIMENTAL ARRANGEMENT: Sample positioned 5 cm from neutron source at angles from 0° to 110° , sample foils sandwiched or interleaved between copper monitor foils.

ACTIVITY MEASUREMENT: Annihilation quanta counted with two 7.6x7.6 cm NaI(Tl) crystals (10 cm apart) in coincidence, samples placed between aluminium absorbers to stop positons.

CORRECTIONS: Decay during irradiation accounted for, flux corrected for neutron scattering from target chamber.

ERRORS: Individual error sources:

- absolute neutron flux determination 6%
- uncertainty in sample position with respect to target 2%
- measurement of detector geometry 3 to 4%
- statistical errors in coincidence counts and/or in counts ratios range from 0.5 to 4%.

EVALUATORS' COMMENT: Work contains complete information about used standards.

NORMALIZATION-RESULTS: Authors' results renormalized according to recommended $T_{1/2}^{r}({}^{57}Ni)$ and $I_{r}^{r}(511 \text{ keV})$ per decay of ${}^{57}Ni$, no renormalization connected with ${}^{65}Cu(n,2n){}^{64}Cu$ cross section was necessary.

E _n [MeV]	∆En [MeV]	61 [mb]	R _T	R _B	62 [mb]	∆ రోజ [%]
13.86	0.10	18.7	0.973	0.994	18.1	8
14,11	0.10	22.9	0.973	0.994	22.1	8
14.24	0.10	27.2	0.973	0.994	26.3	8
14.37	0.15	29.3	0.973	0.994	28.3	8
14.49	0.20	31.7	0.973	0.994	30.7	- 8
14.59	0.20	33.5	0.973	0.994	32.4	8
14.69	0.25	35.9	0.973	0.994	34.7	8
14.77	0.25	36.2	0.973	0.994	35.0	8
14.88	0.30	39.5	0.973	0.994	38.2	8

 $\begin{aligned} \mathbf{G}_{i} &= \text{cross section given by authors,} \\ \mathbf{R}_{T} &= \text{normalization factor:} \\ \mathbf{R}_{T} &= T_{1/2}^{r} ({}^{57}\text{Ni})/T_{1/2} ({}^{57}\text{Ni}) = 36.0/37.0 = 0.973, \\ \mathbf{R}_{B} &= \text{normalization factor:} \\ \mathbf{R}_{B} &= I_{\delta} (511 \text{ keV})/I_{\delta}^{r} (511 \text{ keV}) = 0.928/0.934 = 0.994, \\ \mathbf{G}_{2} &= \text{adopted cross section } \mathbf{G}_{2} &= \mathbf{G}_{i} \times \mathbf{R}_{T} \times \mathbf{R}_{B}, \\ \Delta \mathbf{G}_{2} &= \text{cross section error.} \end{aligned}$

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AUTHORS: N.T.Bramlitt, R.W.Fink.

REFERENCE: Phys. Rev. <u>131</u> /1963/ 2649 [Bra 63]. Some experimental details taken from [Bra 62].

LABORATORY: Department of Chemistry, University of Arkansas, Fayetteville, Arkansas, USA.

QUANTITY: Cross section for 58 Ni $(n, 2n)^{57}$ Ni at 14.7 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺ 200 keV, cross section ⁺ 12.9%.

STANDARDS: 27 Al(n, ∞) 24 Na cross section 6 = 114 mb at 14.7 MeV, 1_{χ} (1370 keV) = 0.75⁺0.06 per decay of 57 Ni.

NEUTRON SOURCE: Arkansas 400 keV Cockcroft-Walton accelerator.

SAMPLES: Nickel foils.

FLUX DETERMINATION: Al monitoring foils placed in front and in back of sample.

ACTIVITY MEASUREMENT: Two-crystal (3" x 3") Nal(Tl) scintillation spectrometer with 200-channel analyzer used for counting 1370 keV gamma-rays.

CORRECTIONS: Neutron output decrease from Ti-T targets observed and taken into account.

ERRORS: Individual sources of error:

- absolute counting correction factor \div 5%

- photopeak area uncertainty + 10%.

NORMALIZATION - DATA:

$$\mathbf{5_{i}}$$
 - cross section given by authors,
 $\mathbf{5_{2}} = {}^{27} \mathrm{Al}(\mathrm{n}, \mathbf{x}) {}^{24} \mathrm{Na}$ cross section used by authors,
 $\mathbf{5_{3}} = {}^{27} \mathrm{Al}(\mathrm{n}, \mathbf{x}) {}^{24} \mathrm{Na}$ recommended cross section,
 $\mathbf{R_{B}} = \mathrm{normalization} \ \mathrm{factor}:$
 $\mathbf{R_{B}} = \mathrm{I}_{\mathbf{x}} (1370 \mathrm{keV}) / \mathrm{I}_{\mathbf{x}}^{\mathbf{r}} (1370 \mathrm{keV}) = 0.75/0.86 = 0.872,$
 $\mathbf{5_{4}} = \mathrm{adopted} \ \mathrm{cross} \ \mathrm{section}: \ \mathbf{5_{4}} = \mathbf{5_{i}} \times \mathbf{R_{B}} \times \mathbf{5_{3}}/\mathbf{5_{2}},$
 $\Delta \mathbf{5_{4}} = \mathrm{cross} \ \mathrm{section} \ \mathrm{error}: \ \Delta \mathbf{5_{4}} = \Delta \mathbf{5_{i}} \times \mathbf{R_{B}} \times \mathbf{5_{3}}/\mathbf{5_{2}}.$

AUTHORS: A.Paulsen and H.Liskien.

REFERENCE: Nucleonik 7 /1965/ 117, [Pau 65].

LABORATORY: Central Bureau for Nuclear Measurements, EURATOM, Geel, Belgium.

QUANTITY: Cross sections for 58 Ni(n, 2n)⁵⁷Ni for 25 energy values from 12.98 MeV to 19.58 MeV.

METHOD OF MEASUREMENTS: Activation.

ACCURACY: Neutron energy spread $\frac{+}{-}$ 170 keV to $\frac{+}{-}$ 470 keV, cross section $\frac{+}{-}$ 7% and $\frac{+}{-}$ 10%.

STANDARDS: $T_{1/2} \begin{pmatrix} 57 & \text{Ni} \end{pmatrix} = 35.8 \text{ h}, I_{\text{S}} (511 \text{ keV}) = 0.938$ photons per decay of 57 & Ni, neutron flux evaluated from formula of Gammel [Gam 63] for total n-p scattering cross section, neutron angular distribution taken from compilation [Pau 64]. NEUTRON SOURCE: 3-MeV Van de Graaff accelerator, Ti-t targets bombarded with 1 MeV and 3 MeV deuterons.

SAMPLES: Metallic discs 20 mm in dia., 5 mm thick.

FLUX DETERMINATION: Proton recoil telescope absolute measurement, flux variations monitored by means of long counter.

EXPERIMENTAL ARRANGEMENT: Samples placed at distance 84.5 mm from the target, deuteron beam limited by aperture 3x8 mm.

ACTIVITY MEASUREMENTS: 0.511 MeV gamma rays counted with NaI(TI) crystals 1.75×2.0^{4} and $4.0^{4} \times 3.0^{4}$, counter efficiency calibrated with ²²Na source of β^{+} decay rate known to $\div 5\%$. CORRECTIONS: Left-right assymetry accounted for, photopeak area corrected for ⁵⁸Co activity, 1.9% correction for loss of neutron flux due to scattering and absorption in samples, 6.0% correction for effects observed under photopeak due to gamma rays of higher energies.

ERRORS: Cross sections determined with accuracy 7% except of threshold region where $\leq 10\%$, individual error sources:

- neutron flux determination	÷4%,
- irradiation geometry	⁺ 1.7% to ⁺ 3.8%,
- neutron angular distribution	÷2.5%,
- efficiency of gamma ray detection	\div 1.5% to \div 2.3%,
- counting rate statistics,	
background, dead time, decay	
constants	⁺ -0.6% to ⁺ -2.0%,
- neutron absorption and	
scattering	÷0.5%,
- chemical purity	÷0.3%,
- flux variation in time	÷0.1%,
- sample weight	÷0.1%.

EVALUATORS COMMENT: Work covers broad energy range with densely spaced experimental points giving detailed shape of excitation curve.

NORMALIZATION-RESULTS: Authors results normalized according to recommended $T_{1/2}^{r} ({}^{57}Ni)$ and $I_{\chi}^{r} (511 \text{ keV})$ per decay of ${}^{57}Ni$.

DATA:

En [MeV]	∆E [MeV]	67 [mb]	R _T	RB	62 [mb]	∆62 [mb]
12.98	0.17	3.2	1.0056	1.0043	3.2	0.3
13.10	0.18	4.6	1.0056	1.0043	4.6	0.4
13.38	0.21	10.1	1.0056	1.0043	10.2	0.7
13.54	0.22	11.9	1.0056	1.0043	12.0	0.8
13.88	0.24	19.6	1.0056	1.0043	19.8	1.4
14.05	0.25	22.0	1.0056	1.0043	22.2	1.5
14.42	0.26	29.4	1.0056	1.0043	29.7	2.1
14.61	0.26	33.4	1.0056	1.0043	33.7	2.3
14.99	0.27	38.0	1.0056	1.0043	38.4	2.7
15.18	0.26	39.5	1.0056	1.0043	39.9	2.8
15.55	0.24	43.0	1.0056	1.0043	43.4	3.0
15.71	0.23	44.5	1.0056	1.0043	44.9	3.1
16.03	0.21	46.1	1.0056	1.0043	46.6	3.2
16.24	0.20	47.1	1.0056	1.0043	47.6	3.3
16.59	0.19	50.4	1.0056	1.0043	50.9	3.5
16.93	0.47	49.9	1.0056	1.0043	50.4	3.5
17.27	0.46	51.2	1.0056	1.0043	51.7	3.6
17.59	0.44	55.1	1.0056	1.0043	55.7	3.9
17.90	0.42	55.0	1.0056	1.0043	55.5	3.9
18.19	0.39	56.4	1.0056	1.0043	57.0	4.0
18.47	0.36	58.5	1,0056	1.0043	59.1	4.1
18.71	0.33	60.4	1.0056	1.0043	61.0	4.2
18.94	0.29	60.5	1,0056	1.0043	61.1	4.2
19.29	0.20	63.8	1.0056	1.0043	64.4	4.5
19.58	0.19	66.5	1.0056	1.0043	67.2	4.7

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$$6i - cross section of 58Ni(n,2n)57Ni given by authors,
RT - normalization factor:
RT = T1/2r(57Ni)/T1/2(57Ni) = 36.0/35.8 = 1.0056,
RB - normalization factor:
RB = IX(511 keV)/Ir5(511 keV) = 0.938/0.934 = 1.0043,
62 - adopted cross section for 58Ni(n,2n)57Ni,
Δ62 - cross section error.$$

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AUTHORS: M.Bormann, F.Dreyer, N.Fretwurst, F.Schenka

REFERENCE: EANDC(E) 66U42 [Bor 66], additional references: Nuclear Physics 63 /1965/ 438 [Bor 65], Nuclear Data $\underline{1A}$ /1966/ 103 [Jes 66].

LABORATORY: Institut für Experimentalphysik, Universität Hamburg.

QUANTITY: Cross section for 58 Ni(n, 2n)⁵⁷Ni for 10 energy values from 12.95 to 19.6 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺0.2 to 0.4 MeV, cross section ⁺6.8%.

STANDARDS: No information.

NEUTRON SOURCE: 3 MeV Van de Graaff and 150 keV Cockcroft--Walton accelerators, $T(d,n)^3$ He reaction on Ti-T target Bor 65].

SAMPLE: No information.

FLUX DETERMINATION: Recoiled protons detected in 2.5x2.5 cm stilbene scintillation counter [Bor 65].

EXPERIMENTAL ARRANGEMENT: Samples placed at different angles around neutron source [Bor 65].

ACTIVITY MEASUREMENT: No information.

CORRECTIONS: Self-absorption and self-scattering accounted for by extrapolating to zero thickness [Bor 65].

ERRORS: Absolute flux determination 4.5% [Bor 65], errors do not include decay schemes uncertainties.

EVALUATORS COMMENT: Data taken from CCDN file, report EANDC(E)66U42 unavailable. No normalization because of lack of information.

En MeV]	∆En [MeV]	б [mb]	∆ 6 [mb]
12.95	0.20	4.7	0.3
13.50	0.25	11.9	0.8
14.10	0.25	22.7	1.4
14.90	0.30	34.6	2.4
15.60	0.35	40.6	2.8
16.50	0.35	44.3	3.0
17.25	0.40	48.2	3.3
18.00 .	0.35	51.3	3.5
18.90	0.3	52.1	3.5
19.60	0.25	53.5	3.6

RESULTS:

6' - cross section given by authors,

 $\Delta 6$ - cross section error,

AUTHOR: J.Csikai,

REFERENCE: ATOMKI Közlemények <u>8</u>/1966/79, [Csi 66]. Some details and data taken from other papers by J.Csikai [Csi 65, Csi 67, Csi 68].

LABORATORY: Institute of Nuclear Research of the Hungarian Academy of Sciences, Debrecen, Hungary.

QUANTITY: Cross section of 58 Ni(n, 2n) 57 Ni for 25 energy values from 13.56 to 15.40 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread $\frac{+}{2}$ %, cross section $\frac{+}{2}$ 8 to 15%.

STANDARDS: Cross section 541 mb at 14.6 MeV for the 63 Cu(n,2n) 62 Cu obtained by normalization of measured relative excitation curve to the 488 mb of Letessier [Let 64] at 14.1 MeV.

NEUTRON SOURCE: ATOMKI neutron generator. Ti-T target bombarded with 80 keV deuterons.

SAMPLES: Thin metal wires.

FLUX DETERMINATION: Flux monitored by long counter and plastic scintillator.

EXPERIMENTAL ARRANGEMENT: Wire samples placed radially at various angles around target.

ACTIVITY MEASUREMENT: Nal(T1) crystal connected to 100 or 125-channel analyzer. 511 keV gamma-rays counted. System calibrated with ²⁰³Hg, ⁵⁴Mn, ¹³⁷Cs and ⁸⁸Y gamma-ray sources.

CORRECTIONS: No information.

ERRORS: Energy spread mainly from stopping of deuterons in target equals several keV at 14.0 MeV, more than 100 keV at 13.5 and 14.7 MeV. Main error source is statistics.

EVALUATORS' COMMENT: The aim of this work is an investigation of cross section fluctuations. Relative numerical values obtained on request. Author's normalization equals 37 + 3 mb at 14.6 MeV. No information about errors in [Csi 66].

NORMAEIZATION - DATA:

 $\mathbf{5}_{\mathbf{1}}$ - relative yield given by author,

62 - absolute cross section according to author's normalization,

 $\mathbf{6_3} - {}^{63}\mathrm{Cu}(n,2n)^{62}\mathrm{Cu} \text{ cross section used by author,}$ $\mathbf{6_4} - {}^{63}\mathrm{Cu}(n,2n)^{62}\mathrm{Cu} \text{ recommended cross section,}$ $\mathbf{6_5} - \text{ adopted cross section: } \mathbf{6_5} = \mathbf{6_2} \times \mathbf{6_4}/\mathbf{6_3},$ $\Delta \mathbf{6_5} - \text{ cross section error: } \Delta \mathbf{6_5} = \Delta \mathbf{6_2} \times \mathbf{6_4}/\mathbf{6_3}.$

MeV $[mb]$ $[mb]$ $[mb]$ $[mb]$ $[mb]$ $[mb]$ 13.560.6022.223.113.610.6122.623.513.660.6222.923.913.710.6724.825.813.760.7025.927.013.810.7327.028.113.960.7427.428.513.910.7327.028.113.960.7025.927.014.010.8029.630.814.100.7929.230.414.110.7929.230.414.160.8631.833.114.210.8130.031.214.360.9535.136.614.410.9033.334.714.461.0037.038.514.510.9635.537.014.661.0438.540.014.711.0037.038.514.70.33738.515.400.345488488457.0 §	E, n	Δ _E n G	Ge	63	64	୍ଟ୍ର	Δ <i>σ</i> 5
13.56 0.60 22.2 23.1 13.61 0.61 22.6 23.5 13.66 0.62 22.9 23.9 13.71 0.67 24.8 25.8 13.76 0.70 25.9 27.0 13.81 0.73 27.0 28.1 13.94 0.73 27.0 28.1 13.96 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.80 29.6 30.8 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.95 35.1 36.6 14.41 0.90 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.66 1.04 38.5 40.0 14.7 0.3 37 38.5 14.7 0.3 37 38.5	MeV	MeV	mb	mb	mb	[mb]	[mb]
13.61 0.61 22.6 23.5 13.66 0.62 22.9 23.9 13.71 0.67 24.8 25.8 13.76 0.70 25.9 27.0 13.81 0.73 27.0 28.1 13.86 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.10 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.70 0.3 37 38.5 14.70 0.3 37 38.5 14.70 0.3 45 488 488 45 7.0 37.0 38.5	13.56	0.60	22.2			23.1	
13.66 0.62 22.9 23.9 13.71 0.67 24.8 25.8 13.76 0.70 25.9 27.0 13.81 0.73 27.0 28.1 13.86 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.80 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 14.7 0.3 37 38.5 14.7 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	13.61	0.61	22.6		v	23.5	
13.71 0.67 24.8 25.8 13.76 0.70 25.9 27.0 13.81 0.73 27.0 28.1 13.86 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.89 29.6 30.8 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.66 1.04 38.5 40.0 14.7 0.3 37 38.5 14.7 0.3 37 38.5 14.7 0.3 45 488 488	13.66	0.62	22.9			23.9	
13.76 0.70 25.9 27.0 13.81 0.73 27.0 28.1 13.86 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.80 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.77 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	13.71	0.67	24.8			25.8	
13.81 0.73 27.0 28.1 13.86 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.89 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 14.7 0.3 37 38.5	13.76	0.70	25.9			27.0	
13.86 0.74 27.4 28.5 13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.89 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.09 37.0 38.5 14.51 0.96 35.5 37.0 14.61 1.02 37.7 541 563 39.3 14.66 1.04 14.71 1.00 37.0 38.5 14.7 0.3 37.0 38.5 14.7 0.3 37.0 38.5	13.81	0.73	27.0		•	28.1	
13.91 0.73 27.0 28.1 13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.80 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 14.7 0.3 37.0 38.5	13.86	0.74	27.4			28.5	
13.96 0.70 25.9 27.0 14.01 0.80 29.6 30.8 14.06 0.80 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	13.91	0.73	27.0			28.1	
14.01 0.80 29.6 30.8 14.06 0.80 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	13.96	0.70	25.9			27.0	
14.06 0.80 29.6 30.8 14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 3.0 § 14.7 0.3 45 488 488 45 7.0 §	14.01	0.80	29.6			30.8	
14.11 0.79 29.2 30.4 14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 14.7 0.3 45 488 488 45 7.0 5	14.06	0.80	29.6			30.8	
14.16 0.86 31.8 33.1 14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 14.71 1.00 37.0 14.7 0.3 37 38.5 3.0 § 14.7 0.3 37 38.5 3.0 § 15.40 0.3 45	14.11	0.79	29.2			30.4	
14.21 0.81 30.0 31.2 14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 14.71 1.00 37.0 14.71 0.3 37 38.5 3.0 § 14.7 0.3 45 488 488 45 7.0 §	14.16	0.86	31.8			33.1	
14.26 0.82 30.3 31.6 14.31 0.89 32.9 34.3 14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 14.71 1.00 37.0 14.71 0.3 37 38.5 3.0 § 14.7 0.3 45 488 488 45 7.0 §	14.21	0.81	30.0			31.2	
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14.36 0.95 35.1 36.6 14.41 0.90 33.3 34.7 14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 14.71 1.00 37.0 14.71 1.00 37.0 14.7 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	14.31	0.89	32.9			34.3	
14.410.9033.334.714.461.0037.038.514.510.9635.537.014.561.0037.038.514.611.0237.754156339.314.661.0438.514.711.0037.038.530.814.70.337.038.530.815.400.345488488457.0 §	14.36	0.95	35.1			36.6	
14.46 1.00 37.0 38.5 14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 14.7 0.3 37 38.5 15.40 0.3 45 488 488 45 7.0 §	14.41	0.90	33.3			34.7	
14.51 0.96 35.5 37.0 14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 15.40 0.3 45 488 488	14.46	1.00	37.0			38.5	
14.56 1.00 37.0 38.5 14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37.0 38.5 15.40 0.3 45 488 488	14.51	0.96	35.5			37.0	
14.61 1.02 37.7 541 563 39.3 14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 15.40 0.3 45 488 488	14.56	1.00	37.0	-		38.5	
14.66 1.04 38.5 40.0 14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 15.40 0.3 45 488 488 45 7.0 §	14.61	1.02	37.7	541	563	39.3	
14.71 1.00 37.0 38.5 14.7 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	14.66	1.04	38.5			40.0	
14.7 0.3 37 38.5 3.0 § 15.40 0.3 45 488 488 45 7.0 §	14.71	1.00	37.0			38.5	
15.40 0.3 45 488 488 45 7.0 §	14.7	0.3	37			38.5	3.0 §
	15.40	0.3	45	488	488	45	7.0 §

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§ - from [Csi 68].

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AUTHOR: J.K.Templerley.

REFERENCE: Nucl. Sci. Eng. <u>32</u>/1968/195, [Tem 68].

LABORATORY: U.S.Army Nuclear Defense Lab., Edgewood Arsenal, Maryland, USA.

QUANTITY: Cross section for 58 Ni(n, 2n) 57 Ni at 6 energy values from 13.72 to 14.79 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺ 200 to 320 keV, cross section ⁺ 10 %.

STANDARDS: $T_{1/2}$ (511 keV) = 0.938 per decay of ⁵⁷Ni, from [Way 60].

NEUTRON SOURCE: USANDL Cockcroft-Walton accelerator. Ti-T targets bombarded with 180 and 150 keV deuterons.

SAMPLES: Chemically pure Ni powder in plastic holder with cavity 2 cm in dia. and 4 mm deep.

FLUX DETERMINATION: Absolute measurement, associated ∞ - particle counting at 135°, counting rate monitored continuously and maintained constant within \div 5%.

EXPERIMENTAL ARRANGEMENT: Samples placed at various angles at mean distance of 4.5 cm from neutron source, bombarded 9 hours. Targets changed every 3 hours.

ACTIVITY MEASUREMENT: Absolute 511 keV gamma-ray counting with 3" x 3" Nal(Tl) crystal, calibrated with single gamma-ray

sources of known activity.

CORRECTIONS: Photopeak areas corrected for self-absorption of gamma-rays in samples.

ERRORS: Individual error sources:

- absolute neutron flux + 8%
- absolute gamma-ray activity + 5%.

NORMALIZATION - DATA: Author's results normalized according to recommended $T_{1/2}^{r}(57 \text{ Ni})$ and $I_{\delta}^{r}(511 \text{ keV})$ per decay of 57_{Ni} .

E _n △E _n [MeV] [MeV]	ର୍ୟ [mb]	R _T	R _B	б <u>е</u> [mb]	∆62 [%]
13.72 0.20	19.0	0.973	1.0043	18.6	10
13.95 0.20	24.4	0.973	1.0043	23.8	10 ΄
14.20 0.25	25.5	0.973	1.0043	24.9	10
14.31 0.31	28.1 .	0.973	1.0043	27.5	10
14.53 0.32	27.3	0.973	1.0043	26.7	10
14.79 0.30	29.1	0.973	1.0043	28.4	10

 δ_1 - cross section given by author,

 $\begin{array}{ll} R_{T} & -\operatorname{normalization factor:} \\ R_{T} & = T_{1/2}^{r} (5^{7} \operatorname{Ni}) / T_{1/2} (5^{7} \operatorname{Ni}) = 36.0/37.0 \pm 0.973, \\ R_{B} & -\operatorname{normalization factor:} \\ R_{B} & = I_{\delta} (511 \ \operatorname{keV}) / I_{\delta}^{r} (511 \ \operatorname{keV}) = 0.938/0.934 = 1.0043, \\ \mathbf{6}_{2} & -\operatorname{adopted cross section:} \mathbf{6}_{2} = R_{T} \cdot R_{B} \cdot \mathbf{6}_{4}, \\ \Delta \mathbf{6}_{2} & -\operatorname{cross section error.} \end{array}$

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AUTHORS: R.C.Barrall, M.Silbergeld, D.G.Gardner.

REFERENCE: Nuclear Physics A138 /1969/ 387, [Bar 69].

LABORATORY: Stanford University, California, USA; Lawrence Radiation Laboratory, University of California, Livermore, USA.

QUANTITY: Cross section for 58 Ni $(n, 2n)^{57}$ Ni at 14.8 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread $\frac{1}{2}$ 200 keV, cross section $\frac{1}{2}$ 8.3%.

STANDARDS: $I_{\mathcal{K}}(1370 \text{ keV}) = 0.86 \text{ per decay of } {}^{57}\text{Ni}.$

NEUTRON SOURCE: Insulated Core Transformer accelerator at LRL. Ti-T target bombarded with 200 keV deuterons.

SAMPLES: $7.6 \times 7.6 \times 0.1$ cm metallic sheets covered with 0.75 mm cadmium.

FLUX DETERMINATION: Proton-recoil telescope used for absolute measurement of neutron flux above 9 MeV.

EXPERIMENTAL ARRANGEMENT: Samples placed on large rotating disc, their centres 36.8 cm from target, at angle of 26.5° to deuteron beam. Disc rotated at 5 rpm.

ACTIVITY MEASUREMENT: 7.6 x 7.6 cm NaI(T1) crystal calibrated with US National Bureau of Standards point-source gamma-ray standards. ERRORS: Two sigma error limits about 8%. Major source of error is measurement of neutron flux, at sample position, by means of proton recoils counting.

NORMALIZATION - DATA: No normalization needed.

En	ΔE _n	6	∆ଟ
[MeV]	[MeV]	[mb]	[mb]
14.8	0.2	36	3

AUTHORS: J.Araminowicz and J.Dresler.

REFERENCE : Institute of Nuclear Research, report No 1464/1/A [Ara 73], details taken from [Ara 72].

LABORATORY: Institute of Physics, University of Łódź.

QUANTITY: Cross section for 58 Ni(n, 2n)⁵⁷Ni at energy 14.6 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺0.38 MeV [Ara 72], cross section ⁺10%.

STANDARDS: $T_{1/2}({}^{57}Ni) = 42.39 \text{ h}, I_{\mathcal{F}}(511 \text{ keV}) = 0.938 \text{ per}$ decay of ${}^{57}Ni, T_{1/2}({}^{62}Cu) = 10.24 \text{ min} [\text{Ara 72}], {}^{63}Cu(n,2n){}^{62}Cu$ cross section $\mathcal{F} = 533 \text{ mb} [\text{Ara 72}].$

NEUTRON SOURCE: 180 keV Cockcroft-Walton, $T(d,n)^4$ He reaction, deuteron beam diameter 8 mm [Ara 72].

SAMPLES: Metallic powder in plexiglass container \$\overline{4}\$ 25x9 mm [Ara 72].

FLUX DETERMINATION: Measured \mathcal{F} -activity of ⁶²Cu produced in ⁶³Cu(n,2n)⁶²Cu reaction, neutron flux monitored by means of associated \mathcal{K} -particle method.

EXPERIMENTAL ARRANGEMENT: Sample irradiated simultaneously with reference sample at angle 45° [Ara 72].

ACTIVITY MEASUREMENT: Photons counted with two Nal(T1) crystals 38x25 mm, in coincidence, distance between crystals 20 mm, iron shield used, [Ara 72]. CORRECTIONS: Divergence of neutron beam accounted for, corrections made for selfabsorption and for target waste [Ara 72].

ERRORS: Only statistical.

EVALUATORS COMMENT: 63 Cu(n, 2n) 62 Cu cross section $\mathfrak{G} = 538$ mb given in [Ara 73] assumed erroneous. Accepted $\mathfrak{G} = 533$ mb [Ara 72] for normatization.

NORMALIZATION-RESULT: Authors result normalized according to recommended $T_{1/2}^{r}({}^{57}Ni)$, $I_{7}^{r}(511 \text{ keV})$ per decay of ${}^{57}Ni$, and ${}^{63}Cu(n,2n){}^{62}Cu$ cross section,

∆E_n ଟ୍ୟ ଟ୍ ଟୁ R_В**б**4 R_T En $\Delta 64$ [mb] [mb] [mb] mb MeV MeV 14.6 0.38 37.9 533 562 0.849 1.004 34.1 3.4 6 - cross section given by authors, $\mathbf{5}_{2} - {}^{63}$ Cu (n,2n) 62 Cu cross section used by authors, $\mathbf{5}_{\mathbf{5}}$ - recommended $\mathbf{63}^{\mathbf{5}}$ Cu (n, 2n) $\mathbf{62}^{\mathbf{5}}$ Cu cross section, **R**_T - normalization factor: $R_{T} = T_{1/2}^{r} (57 \text{Ni}) / T_{1/2} (57 \text{Ni}) = 36.0/42.39 = 0.849,$ **R**_B - normalization factor: $R_B = I_{\sigma}(511 \text{ keV}) / I_{\sigma}^r (511 \text{ keV}) = 0.938 / 0.934 = 1.004,$ G_4 - adopted cross section $G_4 = G_4 (G_3/G_2) R_T R_B$, $\Delta \vec{6}_4 - \text{cross section error } \Delta \vec{6}_4 = \Delta \vec{6}_4 (\vec{6}_3 / \vec{6}_2) R_T R_B$

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AUTHOR: J.D.Hemingway.

REFERENCE: J. Nuclear Energy 27 /1973/241, [Hem 73].

LABORATORY: Londonderry Lab. for Radiochemistry, University of Durham, England.

QUANTITY: Cross section for ⁵⁸Ni(n,2n)⁵⁷Ni at 14.7 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread $\frac{+}{200}$ keV, cross section $\frac{+}{8\%}$.

STANDARDS: 56 Fe(n,p) 56 Mn cross section at 14.7 MeV equal 97.8 mb [Hem 66]. Isotopic abundances and decay data taken from Nuclear Data Sheets.

NEUTRON SOURCE: Ti-T target bombarded with 150 keV deuterons.

SAMPLES: Nickel oxide mixed with iron granules irradiated in polyethylene container 8 mm x 15 mm in dia.

FLUX DETERMINATION: Variations in flux monitored with proton recoil counter.

EXPERIMENTAL ARRANGEMENT: Sample placed 3 mm away from target (of 10 mm diameter) in forward direction.

ACTIVITY MEASUREMENT: Magnetically separated iron dissolved in standard acid mixture and counted in G-M counter of known efficiency. ⁵⁷Co chemically separated from dissolved NiO, ⁵⁷Co gamma-rays 122 + 136 keV measured using 76 x 76 mm NaI (T1) crystal. System calibrated for 57 Co by $4\pi\beta-\delta$ coincidence counting. Second chemical separation of cobalt from nickel carried out some ten days after irradiation. From two ⁵⁷Co measurements, contribution of 58 Ni(n, 2n) and 58 Ni(n, np etc.) reactions could be separated.

CORRECTIONS: No information.

ERRORS: Individual error sources:

- counting	÷2.7%,
- reference cross section	÷4.5%,
- counter efficiency	below - 2%.

Two latter errors taken as systematic error of $\div 5.5\%$.

Estimated neutron energy spread covers variation of neutron energy with emission angle and energy loss inside target.

NORMALIZATION - DATA: Author's results normalized according to recommended reference cross section.

En	ΔE	бі	62	бз	ୈ	۵ <u>ښ</u>	
MeV	MeV	mb	mb	mb	mb	mþ	
14.7	9.2	32.6	97.8	104.0	34.7	2.9	

 $\mathbf{\tilde{6}_{l}}$ - cross section given by author,

 $\mathbf{5}_{2} - \frac{56}{\text{Fe}(n,p)} \frac{56}{\text{Mn}}$ cross section used by author, $\mathbf{5}_{3} - \frac{56}{\text{Fe}(n,p)} \frac{56}{\text{Mn}}$ recommended cross section,

- $\mathbf{54}$ adopted cross section: $\mathbf{54} = \mathbf{54} \times \mathbf{55}/\mathbf{52}$, $\Delta \vec{6}_4$ - cross section error: $\Delta \vec{6}_4 = \Delta \vec{6}_1 \times \vec{6}_3 / \vec{6}_2$.

AUTHORS: R.Spangler, E.Linn Draper, Jr., T.A.Parish. REFERENCE: Trans. Amer. Nucl. Soc. <u>22</u> /1975/ 818, [Spa 75]. LABORATORY: University of Texas, Austin, USA. QUANTITY: Cross section for 58 Ni(n,2n) 57 Ni at 14.1 MeV. METHOD OF MEASUREMENT: Activation.

ACCURACY: Cross section - 12.5%.

STANDARDS: 27 Al(n, ∞) 24 Na cross section 6 = 115 mb at 14.1 MeV [Bar 69], I γ (511 keV) = 0.92 per decay of 57 Ni.

NEUTRON SOURCE: Texas Nuclear neutron generator.

SAMPLES: Foils of various sizes.

FLUX DETERMINATION: No information.

EXPERIMENTAL ARRANGEMENT: No information.

ACTIVITY MEASUREMENT: 511 keV gamma-rays counted using Nal (T1) crystal and multichannel analyzer.

CORRECTIONS: No information.

ERRORS: No information.

NORMALIZATION - DATA: Authors' results normalized according to recommended $I_{\mathcal{F}}^{r}(511 \text{ keV})$ and recommended reference cross section.

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AUTHORS: S.M.Qaim and N.I.Molla.

REFERENCE: Proceedings of the 9th Symposium on Fusion Technology, Pergamon Press and New York 1976 [Qai 76], and private communication [Qai 77].

LABORATORY: Institut für Chemie der Kernforschungsanlage Jülich.

QUANTITY: Cross section for 58 Ni(n,2n) 57 Ni at 14.7 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺0.3 MeV, cross section ⁺8.6%.

STANDARDS: $T_{1/2}({}^{57}Ni) = 36.0 h$ [Qai 77], $I_{S}(1370 \text{ keV}) = 0.86$ per decay of ${}^{57}Ni$ [Qai 77], ${}^{27}Al(n,S){}^{24}Na$ cross section $\mathfrak{S} = 121 \text{ mb}$, $T_{1/2}({}^{24}Na) = 15.05 h$ [Qai 77].

NEUTRON SOURCE: Tritium target bombarded with deuterons, flux density $(2 + 3) \times 10^9$ cm⁻² sec⁻¹.

SAMPLE: Natural element.

FLUX DETERMINATION: Measured activity of ²⁴Na produced in 27 Al(n, ∞)²⁴Na reaction.

EXPERIMENTAL ARRANGEMENT: No information.

ACTIVITY MEASUREMENT: 1370 keV gamma rays counted with Ge/Li/ spectrometer.

CORRECTIONS: No information.

ERRORS: No information.

NORMALIZATION-RESULT: Authors results normalized according to recommended 27 Al(n, χ) 24 Na cross section.



AUTHORS: B.P.Bayhurst, J.S.Gilmore, R.J.Prestwood, J.B.Wilhelmy, Nelson Jarmie, B.M.Eskkila and R.A.Hardekopf.

REFERENCE: Physical Review C 12 /1975/ 451 [Bay 75].

LABORATORY: University of California, Los Alamos Scientific Laboratory, Los Alamos, New Mexico.

QUANTITY MEASURED: Cross sections of 58 Ni(n, 2n) 57 Ni for 10 energy values from 16 to 28 MeV.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread ⁺0.09 to 0.17 MeV, cross section ⁺3.8 to 7.1%.

STANDARDS: $T_{1/2}({}^{57}Ni) = 36.0 h$, ${}^{27}Al(n, \mathcal{X})^{24}Na$ cross sections, $T_{1/2}({}^{24}Na) = 15.0 h$, ${}^{90}Zr(n, 2n)^{89}Zr$ cross sections, $T_{1/2}({}^{89}Zr) = 78.4 h$, H(n, n)H cross sections.

NEUTRON SOURCE: Van de Graaff accelerator with

- D(d,n)³He for 16.2 MeV
- $T(d,n)^4$ He for 17.2 MeV, 18.2 MeV, 20.0 MeV, 22.0 MeV,
- $D(t,n)^4$ He for 21.3 MeV, 23.3 MeV, 24.5 MeV, 26.0 MeV, 28.0 MeV.

SAMPLE: Circular foils 9.5 mm in diameter and 0.06 mm thick.

FLUX DETERMINATION: At 16.2 MeV measured activity of 89 Zr produced in 90 Zr(n,2n) 89 Zr reaction and recoiled protons,

at energies 17.2, 18.2, 20.0, 21.3, 22.0, 23.3, 24.5 MeV measured only activity of ²⁴Na produced in ²⁷Al(n, \mathcal{X}) ²⁴Na reaction, proton recoil telescope used for energies higher than 21.0 MeV, at 28.0 MeV exceptionally measured activity of ⁸⁹Zr from ⁹⁰Zr(n,2n)⁸⁹Zr reaction also.

EXPERIMENTAL ARRANGEMENT: Sample stacks consisting of alternate monitor and sample foils in a steel holder placed at distance 10 - 15 mm from gas target (3 cm long), reaction product separated chemically.

ACTIVITY MEASUREMENT: β^+ spectroscopy with use of proportional counter, decay curves analysed by least-squares method.

CORRECTIONS: Activity induced by the low energy neutrons from (d,n) or (t,n) reactions on gold and other constructional elements accounted for.

ERRORS: Absolute counting efficiency, weighing, decay constants, flux determination

- 5% for 90 Zr(n,2n) 89 Zr cross section - 3 to 5% for 27 Al(n,0C) 24 Na cross section

- 2 to 3% for recoil proton telescope.

EVALUATORS' COMMENT: Only work containing results for energies exceeding 20 MeV. Cross sections obtained at neutron energies between 16 MeV and 20 MeV are considerably higher than other results. No explanation for this discrepancy was found. This excitation curve has been accepted for evaluation in the 20 to 28 MeV energy region.

RESULTS:

E _n MeV	∆E _n MeV	6~ mb	∆6~ mb
	<u>, , , , , , , , , , , , , , , , , , , </u>	- 	
16.21	0.11	63	3
17.23	0.17	70	5
18.24 -	0.09	76	5 ,
19.99	0.11	92	6
21.26	0.18	93	4
22.01	0.13	97	4
23.36	0.11	102	4
24.49	0.12	96	4
26.06	0.12	87	4
28.05	0.14	72	3

 \mathfrak{S} - cross section for 58 Ni $(n, 2n)^{57}$ Ni given by authors.

AUTHORS: L.Adamski, M.Herman, A.Marcinkowski.

REFÉRENCE: Unpublished [Mar 77].

LABORATORY: Institute of Nuclear Research, Warsaw.

QUANTITY MEASURED: Cross sections for 58 Ni(n, 2n) 57 Ni at 4 neutron energies.

METHOD OF MEASUREMENT: Activation.

ACCURACY: Neutron energy spread $\frac{1}{2}$ 0.07 to 0.80 MeV, cross section $\frac{1}{2}$ 8 to 12%.

STANDARDS: $T_{1/2}({}^{57}Ni) = 36.0 \text{ h}, I_{\$}(1370 \text{ keV}) = 0.86 \text{ per}$ decay of ${}^{57}Ni, {}^{27}Al(n,\alpha){}^{24}Na \text{ cross sections}, T_{1/2}({}^{24}Na) = 15.0 \text{ h},$ ${}^{56}Fe(n,p){}^{56}Mn \text{ cross sections}, T_{1/2}({}^{56}Mn) = 2.576 \text{ h}.$

NEUTRON SOURCE: 3 MeV Van de Graaff accelerator, $T(d,n)^4$ He reaction on Ti-T target.

SAMPLE: Foils of Ni and reference foils of Fe and Al rolled together in plexiglas container 4 mm in diameter.

FLUX DETERMINATION: Measured activity of ²⁴Na and ⁵⁶Mn produced in ²⁷Al(n, ∞) ²⁴Na and ⁵⁶Fe(n, p)⁵⁶Mn reactions, respectively.

EXPERIMENTAL ARRANGEMENT: Samples placed 2 cm from target at various angles.

ACTIVITY MEASUREMENT: 1370 keV *S*-rays counted with 40 ccm Ge(Li) spectrometer.

ERRORS: Individual error sources

- statistical errors 5%, - sample weight 0.1%, - flux variation in time 0.3%, - reference cross section 5% for 56 Fe (n,p) 56 Mn, 4.2% for 27 Al(n, ∞) 24 Na, - Ge(Li) relative efficiency 1% (only for 56 Mn \checkmark - rays).

EVALUATIONS' COMMENT: Work undertaken specially for the purpose of this evaluation.

RE	SU	LT	S	:
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En MeV]	∆E _n [MeV]	6] [mb]	62 [mb]	63 [mb]	∆63 [mb]	
14.02	0.80	111.5		18.0	2.1	-
16.42	0.68	70.0		48.3	4.1	
17.42	0.44		73.0	56.8	5.8	
17.42	0.44	64.2		59.0	6.2	
17.85	0.07		66.5	60.0	6.1	
17.85	0.07	60.0		62.5	6.4	
			•			

 $\mathbf{6_1} = {}^{56}\text{Fe}(n,p){}^{56}\text{Mn cross section},$ $\mathbf{6_2} = {}^{27}\text{Al}(n,\infty){}^{24}\text{Na cross section},$ $\mathbf{6_3} = {}^{58}\text{Ni}(n,2n){}^{57}\text{Ni cross section},$ $\mathbf{6_5} = \text{cross section error}.$

V. RECOMMENDED DATA SET

Considering the 58 Ni(n, 2n) 57 Ni experimental cross sections we have distinguished three regions, in which different criteria of evaluation have been assumed. The first region extends from the threshold energy up to about 16 MeV neutron energy. In this region the accepted data are consistent within the experimental errors and concentrated along a smoothly increasing with increasing energy, line. In the second region between 16 MeV and 20 MeV there exist the data of the four excitation curves measured by Prestwood et al. [Pre 61], Paulsen and Liskien [Pau 65], Bormann et al. [Bor 66] and Bayhurst et al. [Bay 75]. These four data sets diverge evidently with increasing neutron energy. The excitation curve of Bayhurst et al. extends into the third region up to 28 MeV and therefore it will be considered independently. In treating the remaining three divergent excitation curves the polynomial

 $P(E) = \sum_{i \leq 6} a_i E^i$ has been fitted separately to each of the data sets

in the energy range from 15.0 MeV to 20.0 MeV supplying P_I , P_{II} and P_{III} . Separate polynomial P_{IV} has been fitted to all the single energy data including the cross sections belonging to the mentioned three data sets, which lie below the 15 MeV limit. The latter fit extends from 13 MeV to 18 MeV. The polynomial P_V has been then fitted to P_I , P_{II} , P_{III} and P_{IV} , assuming a constant energy step. Such a procedure ascribes an equal weight to the three data sets [Pre 61], [Pau 65] and [Bor 66], not depending on the number of points in each excitation curve. This was our intention.

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As a next step the polynomial fit to the Csikai [Csi 66] data, in the energy range 13.6 - 15.4 MeV, as well as an independent polynomial fit to the whole curve measured by Bayhurst et al. [Bay 75], both lying well above the average trend displayed by the remaining data, have been performed. The resulting polynomials were normalized to the P_V , the normalization constants being 0.79 and 0.76, respectively. After normalization the two latter polynomials and the P_I , P_{II} and P_{III} were at last fitted to the $P(E) = \sum_{i=0}^{5} \alpha_i E^i$ giving the recommended data set dis-

played in Fig. 4 and 5 and in Table 2.

The mean square deviation of the data from the recommended data curve /excitation curve treated as single measurement/ together with the average experimental error, assumed to be 7.5% below 14 MeV and 8.0% above 14 MeV, served for determining the errors of the recommended cross section below 16 MeV. The same being true for the lower limit of the errors in whole energy range. The upper limit above 22 MeV equals to the average deviation of the data points given by Bayhurst et al. [Bay 75] from the recommended curve. A linear interpolation in the transition region reflects existence of an overlap between the overestimated data of Bayhurst et al. and the remaining ones.

VI. CONCLUSIONS

A great attention has been paid to the 58 Ni(n,2n) 57 Ni reaction used as a neutron monitor. This resulted in the evaluations of Barrall UKNDL (DFN.236) [Bar 65], Bhat ENDF/B-IV (Mat. 6419), Guenther et al. [Gue 75] and Lapenas [Lap 75] published recently. These evaluations range up to 20 MeV in neutron energy. Their results are compared with the recommended data obtained in the present work in Fig. 6.

The data of Guenther et al. and of Lapenas agree with our evaluation within 3% and 5%, respectively. The other two evaluations deviate from our data for energies lower than 14 MeV and higher than 17 MeV. In the latter energy region the excitation curves of both Barrall and Bhat follow the data obtained by Bormann et al. [Bor 66], which are lower than the average trend. Our recommended cross sections extend above the 20 MeV limit basing on the results of the experiment performed in 1975 by Prestwood et al. [Pre 75].

The statistical model [Dec 72] estimates do not facilitate the evaluation, the theoretical cross sections being much higher than the experimental ones. The compound nucleus cross sections should be corrected in order to allow for the precompound emission [Aug 77], however this correction is very small and the prediction of the theory remains too high by about 4 times. This discreapancy is a known fact, which has not found explanation until now. The theorem retical cross sections are compiled in Table 3.

TABLE 3.

En [MeV]	G _{comp} [md]	R	රි theo. [mb]	6 r [mb]
13.0	43.6	1.009	44.0	4.1
15.0	147.9	1.005	148.6	36.0
17.0	201.4	1.096	220.7	52.6

6 - statistical model estimates,

- theoretical predictions,

R

- correction factor accounting for the precompound emission,

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- recommended experimental cross section.

The recommended cross sections are tabelarized in 100 keV energy steps from 12.9 MeV to 28.0 MeV in Table 2. The absolute errors, described in chapter V, are attached. Unfortunately they exceed slightly the 10% error limit requested by Michaudon [Wrenda 76/77], reaching 11.4% from the threshold energy up to 14.0 MeV. In the energy range from 14.0 MeV to 16.0 MeV the accuracy reaches 8.7%. At still higher energies the estimated accuracy is again worse because of the inconsistency of the results of Prestwood et al. with the average trend displayed by the data at lower energies.

TABLE 2

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RECOMMENDED CROSS SECTIONS FOR ${}^{58}N_1(n,2n) {}^{57}N_1$

(The recommended cross sections are described by the 5th order polynomial with the following coefficients:

$$a_0 = -3230.7$$

 $a_1 = 755.26$
 $a_2 = -70.268$
 $a_3 = 3.2762$
 $a_4 = -.075663$
 $a_5 = .00068650$).

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	NEUTRON ENERGY MEV	CROSS SECTION MB	ABSOLUTE ERROR MB	
):≩ i jone ₩	179.2	4791 3	
	12.9	1.8	+ .22	
	13.0	4.1	+ •5 - •5	
	13.1	6.3	+ .77	
	13.2	8.5	+ 1.0 -1.0	
	13.3	10.5	+ 1.2 -1.2	
	13.4	12.5	+ 1.4 -1.4	
	13.5	14.4	+ 1.6 -1.6	
	13.6	16.2	+ 1.8 -1.8	
	13.7	18.0	+ 2.0 -2.0	
	13.8	19.7	+ 2.2 -2.2	
	13,9	21.3	+ 2.4 -2.4	
	14.0	22.9	+ 2.0 -2.0	
	14.1	24.4	+ 2.1 -2.1	
•	14.2	25.9	+ 2.3 -2.3	
	14.3	27.3	+ 2.4 -2.4	
	14.4	28.7	+ 2.5 -2.5	•
	14.5	30.0	+ 2.6 -2.6	
	14.6	31.3	+ 2.7 -2.7	
	14.7	32.5	+ 2.8 -2.8	
	14.8	33.7	+ 2.9 -2.9	
	14.9	34.9	+ 3.0 -3.0	
	15.0	36.0	+ 3.1 -3.1	
	15.1	37.1	+ 3.2 -3.2	
	15.2	38.1	+ 3.3 -3.3	
	15.3	39.1	+ 3.4 -3.4	
	15.4	40.1	+ 3.5 -3.5	
	15.5	41.1	+ 3.6 -3.6	
	15.0	42.0	+ 3.7 -3.7	
	10.1	42.9	+ 3.1 -3.1	
	10.0	4.3 • [+ 3.8 -3.8	
	12.0	44•D		
	10.0	40.4		
	16.1	40.0	$+ 4_{0}3 - 4_{0}0$	
	16.3	47.7	+ 4.7 = 4.2	
	16.4	48.5	+ 4.0 -4.2	
	16.5	49.2	+ 5.1 -4.3	
	16.6	49.9	+ 5,3 -4.3	
	16.7	50.6	+ 5.5 -4.4	
	16.8	51.3	+ 5.7 -4.5	
	16.9	51.9	+ 5.9 -4.5	
	17.0	52.6	+ 6.2 -4.6	
	17.1	53.2	+ 6.4 -4.6	
	17.2	53.8	+ 6.6 -4.7	
	17.3	54.4	+ 6.8 -4.7	
	17.4	55.0	+ 7.1 -4.8	•
	17,5	55.6	+ 7.3 -4.8	
	17.6	56.2	+ 7.5 -4.9	
· ·	17.7	56.7	+ 7.7 -4.9	
	17.8	57.3	+ 8.0 -5.0	
	17.9	57.8	+ 8.2 -5.0	

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	NEUTRON Energy S Mev	CROSS ECTION MB	ABSOL ERR MB	UTE ROR	·	
	18.0 18.1 18.2 18.3 18.4 18.5 18.6	58.4 58.9 59.4 59.9 60.4 60.9 61.4	+ 8.5 + 8.7 + 9.2 + 9.4 + 9.7 + 9.9	-5.1 -5.1 -5.2 -5.2 -5.3 -5.3 -5.3		
• •	18.7 18.8 18.9 19.0 19.1 19.2 19.3 19.4	62.4 62.9 63.3 63.8 64.3 64.7 65.1	+10.2 +10.4 +10.7 +10.9 +11.2 +11.4 +11.7 +11.9	-5.4 -5.5 -5.5 -5.6 -5.6 +5.6		
	19.5 19.6 19.7 19.8 19.9 20.0 20.1	65.6 66.0 66.4 66.8 67.2 67.5 68.0	+12.2 +12.5 +12.7 +13.0 +13.3 +13.5 +13.8	-5.7 -5.7 -5.8 -5.8 -5.8 -5.9 -5.9		
	20.2 20.3 20.4 20.5 20.6 20.7 20.8 20.9	68.4 68.8 69.2 69.5 69.9 70.2 70.5 70.5	+14.1 +14.3 +14.6 +14.9 +15.1 +15.4 +15.7	-6.0 -6.0 -6.0 -6.1 -6.1 -6.1 -6.2		
	20.9 21.0 21.1 21.2 21.3 21.4 21.5 21.6	71.2 71.5 71.5 71.8 72.1 72.3 72.6 72.9	+15.9 +16.2 +16.5 +16.7 +17.0 +17.3 +17.5 +17.8	-6.2 -6.2 -6.2 -6.3 -6.3 -6.3 -6.3		
	21.7 21.8 21.9 22.0 22.1 22.2 22.3	73.1 73.3 73.5 73.7 73.9 74.1 74.3	+18.0 +18.3 +18.5 +18.8 +18.8 +18.9 +18.9 +18.9	-6.4 -6.4 -6.4 -6.4 -6.4 -6.4 -6.5		
	22.5 22.6 22.7 22.8 22.9 23.0	74.5 74.6 74.7 74.8 74.9 74.9	+19.0 +19.0 +19.0 +19.1 +19.1 +19.1 +19.1	-6.5 -6.5 -6.5 -6.5 -6.5		

NEUTRON ENERGY MEV	CROSS SECTION MB	ABSO ER M	LUTE ROR B
23.1 23.2 23.3 23.4 23.5 23.6 23.7 23.8 23.9 24.0 24.1 24.2 24.3 24.4	75.0 75.0 75.0 74.9 74.9 74.9 74.8 74.7 74.6 74.5 74.5 74.3 74.2 74.0 73.8	+19.1 +19.1 +19.1 +19.1 +19.1 +19.1 +19.1 +19.1 +19.0 +19.0 +19.0 +18.9 +18.9 +18.8	-6.5 -6.5 -6.5 -6.5 -6.5 -6.5 -6.5 -6.5
24.5 24.6 24.7 24.8 24.9 25.0 25.1 25.2 25.3 25.4 25.5 25.6	73.5 73.3 73.0 72.7 72.4 72.1 71.7 71.4 71.0 70.6 70.1 69.7	+18.8 +18.7 +18.6 +18.5 +18.5 +18.5 +18.4 +18.3 +18.2 +18.1 +18.0 +17.9 +17.8	-6.4 -6.4 -6.3 -6.3 -6.3 -6.2 -6.2 -6.2 -6.2 -6.1 -6.1
25.7 25.8 25.9 26.0 26.1 26.2 26.3 26.4 26.5 26.6 26.7 26.8	69.2 68.7 68.2 67.7 67.1 66.5 66.0 65.3 64.7 64.1 63.4 62.7	+17.6 +17.5 +17.4 +17.3 +17.1 +17.0 +16.8 +16.7 +16.5 +16.3 +16.2 +16.0	-6.0 -6.0 -5.9 -5.9 -5.8 -5.8 -5.7 -5.7 -5.7 -5.6 -5.6 -5.5 -5.5
26.9 27.0 27.1 27.2 27.3 27.4 27.5 27.6 27.7 27.8 27.9 28.0 28.1	62.1 61.4 60.6 59.9 59.2 58.4 57.6 56.8 56.0 55.2 54.4 53.6 52.8	+15.8 +15.6 +15.5 +15.3 +15.1 +14.9 +14.7 +14.7 +14.5 +14.3 +14.1 +13.9 +13.7 +13.5	-5.4 -5.3 -5.2 -5.1 -5.1 -5.1 -5.0 -4.9 -4.9 -4.8 -4.7 -4.7 -4.6

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Fig. 2. Compilation of data for 58 Ni(n,2n) 57 Ni reaction cross section

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recommended cross sections and the error limits.

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/fragment/.



Fig. 6. Comparison of evaluations for 58 Ni(n,2n) 57 Ni cross sections. The possible trend assumed by Vlasov [Vla 76] is also shown.

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