# INJC 414

**SURVEY** OF · **RESEARCH REACTORS** 

# **National Academy of Sciences**



SURVEY

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# RESEARCH REACTORS

SUBCOMMITTEE ON RESEARCH REACTORS

COMMITTEE ON NUCLEAR SCIENCE

NATIONAL RESEARCH COUNCIL

National Academy of Sciences Washington, D.C. 1970 This is a report of work under Contract NSF-C310 T.O. 47 between the National Science Foundation and the National Academy of Sciences and Contract AT(30-1)-4184 between the U.S. Atomic Energy Commission and the National Academy of Sciences.

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#### SUMMARY

This report presents the results of a survey of research reactor usage conducted by the Subcommittee on Research Reactors of the Committee on Nuclear Sciences of the National Research Council. The survey was limited to research reactors licensed by the Atomic Energy Commission to operate in the 10 kW-10 MW (thermal power) range. Reactors operated above 10 MW are classed as test reactors and were excluded from the survey. The survey also excluded reactors located at AEC-operated laboratories.

The principal factors of reactor usage surveyed were: power level; hours of operation per quarter year; number of research users; number of samples irradiated; types of facilities used; scientific disciplines of users; and kinds of research supported.

When the survey began, in 1966, there were 71 licensed reactors in operation with 48 of them within the power spread specified by the survey. These 48 reactors represented a combined licensed power of 45 MW. From 32 of these reactors, representing a combined licensed power of 25 MW, data were submitted for the period October 1, 1967, through March 31, 1968, or two quarters. The locations of the 32 reactors were: university, 25; industry, 5; and government installations (extra-AEC), 2.

Significant facts about research reactor usage are:

1. On the average, the higher the power level at which a research reactor operates, the greater its usage. This is true whether based upon hours of use of the reactor or upon the number of people using it. (Exceptions to this general rule are present, however, demonstrating the excellent capabilities of some individuals working with the lower-powered reactors.)

2. For a typical quarter of a year, the 32 research reactors surveyed produced about 12,000 radioactive samples and supported about 285 research users of these samples. In addition, there were about 220 users of beam ports and other experimental facilities. (These figures illustrate very forcibly that a large number of research projects were carried out using research reactors.)

3. In combination, the figures show that 500 different people had their research supported by the 32 research reactors. They indicate that over 700 research people probably were supported by the total of 48 reactors falling within the survey. These reactors represented a combined licensed power of 45 MW. It is clear that no single reactor at 45 MW could support 700 people during a threemonth period. 4. The 16 reactors not included in the survey results were licensed to operate at somewhat greater average power than the 32 reported on here. Applying Fact 1 to these 16 reactors, one can conclude that they would show somewhat greater usage than those that actually reported.

5. The research reactors supported research in many different disciplines. Beam ports, thermal columns, in-core facilities, which often involve complicated pieces of research equipment, are used most by nuclear engineers and physicists. However, the reactors also supported chemists, biologists, medical people, geologists, and others.

6. The existence of research reactors around the country greatly expands the use of radioisotopes. This is particularly true of the short-lived isotopes, which would otherwise be un-available to many scientists.

#### GENERAL

During the past 15 years a substantial number of research reactors were constructed in the United States. They varied considerably in size, power, ownership, and specialized capability. As of the end of 1966, there were 71 such reactors licensed by the Atomic Energy Commission. In addition, about 15 others had been constructed in the national laboratories of the AEC and were operating at that time. Whereas the utilization of the AEC national laboratory research reactors was to some extent known because of AEC reporting requirements, much less was known about the licensed reactors. These were the object of this survey.

The survey was restricted to reactors outside of AEC-operated laboratories. The criterion was that the reactor was subject to the control of the licensing and regulation division of AEC. This meant that the group of reactors surveyed had relatively comparable problems for their operation. The survey was further limited to reactors operating at 10 kW to 10 MW (thermal power). Reactors operating at 10 MW or above are regarded as test reactors rather than research reactors.

#### THE SURVEY

The Subcommittee developed a questionnaire as the means of obtaining the required data. Specifically, it developed a format for the reactor directors to use in submitting data, so that certain statistical analyses could be performed. The following types of information were sought:

General operational data Production of radioisotopes On-site usage of the reactor Kinds and numbers of research people supported Kinds of research supported

A copy of the Utilization Data Form is included as Appendix A. Data were to be submitted on a quarterly basis.

It was the original intention of the Subcommittee to determine the actual utilization and also trends in utilization of the reactors, over a two-year period. However, it took considerable time for the reactor installations to form the habit of collecting the data requested on the Utilization Data Form. For this reason, the Subcommittee amended its aims. In the summer of 1967 it decided to attempt to learn what the reactor utilization might be in a typical quarter and to forgo any attempt at that time to determine possible trends. It was decided that the survey would study two quarters--the fourth quarter of 1967 and the first quarter of 1968--to obtain a typical picture of what was being done with the reactors.

## THE REACTORS

When the survey began, in 1966, there were 71 licensed reactors in the United States, falling into the six categories shown in Table 1. Twenty-three of these reactors operated at less than

Туре	Number
I AGN 201 II TRIGA III Argonaut IV Pool type V Tank type VI Homogeneous	15 14 6 26 5 <u>5</u> 71

TABLE 1 Reactor Types Surveyed

	In Survey	Not in Survey	
University	25	3	
Industrial	5	8	
Government	2	5	

TABLE 2 Reactor Ownership

10 kW (thermal power) and were not included in the study. The balance of 48 reactor directors were invited to collect data in uniform format for the study, and 38 reactor directors indicated that they would supply data if at all possible. Four directors indicated that they could not participate; and six did not respond at all. An alphabetical list of the 48 reactors about which the survey requested data is included as Appendix B.

In the end, data were received about 32 reactors. Not reported on were a number of reactors which were not operated during the specified quarters because they were being modified. Two other reactor directors cited "company proprietary rights" as barring submission of sufficient data to be utilized. The distribution of ownership of the 48 reactors is shown in Table 2.

Table 3 lists the licensed power of each of the 48 reactors, those included in the survey and those not included, as of the end of 1967. It will be noted that the average licensed power of the survey group was slightly lower than that of the group not covered by the survey. Of the 32 reactors in the survey group, 78 percent are operated by universities. Twelve reactors of the survey group had licensed powers of 1 MW or higher. The remaining 20 reactors ranged in power from two at 250 kW to seven at 10 kW.

Although the group surveyed is not a perfect cross section of the entire group of 48, it represents a large fraction of the total group, and the data obtained accordingly permit certain conclusions.

#### RESULTS OF THE SURVEY

Detailed results of the survey are presented in Tables 4 and 5. The reactors are designated by number only, because the Subcommittee had agreed in advance to preserve anonymity of the various facilities. The reactors are listed in descending order of licensed power. University reactors are indicated with a "U." Totals and average values appear at the bottom of each column. The number in parentheses after

1. 5000 kW 1. 5000 kW 2. 5000 2. 5000 3. 2000 3. 3000 4. 2000 4. 2000 5. 2000 5. 1000 6. 1500 6. 1000 7. 1500 7. 1000 8. 1000 9. 1000 10. 1000 10. 250 11. 1000 12. 75 13. 250 13. 50 14. 250 14. 30 15. 200 15. 10 16. 100 16. <u>10</u> 17. 100 12. 1283 kW average power 18. 100 19. 100 20. 100 21. 100 22. 100 23. 100 24. 100 25. 18 26. 10 27. 10 28. 10 29. 10 30. 10 31. 10 32. <u>10</u> 803 kW average power	In	Survey	Not	in Survey	
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3. 2000 4. 2000 5. 2000 5. 2000 6. 1500 7. 1500 7. 1000 8. 1000 9. 1000 9. 1000 10. 1000 11. 1000 12. 1000 12. 75 13. 250 14. 250 15. 10 16. 100 15. 200 15. 10 16. $100$ 17. 100 20. 100 21. 100 22. 100 23. 100 24. 100 25. 18 26. 10 27. 10 28. 10 29. 10 30. 10 31. 10 32. $-10$ 803 kW average power	2.	5000	2.	5000	
4. 2000 5. 2000 6. 1500 6. 1500 7. 1500 7. 1000 8. 1000 9. 1000 10. 250 11. 1000 12. 75 13. 250 14. 250 15. 100 16. 100 17. 100 19. 100 20. 100 21. 100 22. 100 23. 100 24. 100 25. 18 26. 10 27. 10 28. 10 29. 10 30. 10 31. 10 32. $-10$ 803 kW average power	3.	2000	3.	3000	
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30. 10 31. 10 32. <u>10</u> 803 kW average power	29.	10			
31. 10 32. <u>10</u> 803 kW average power	30.	10			
32. <u>10</u> 803 kW average power	31.	10			
803 kW average power	32.	$\frac{10}{10}$			
		803 kW average	power		

TABLE 3 Reactor Powers

TABLE 4 Fourth Quarter of 1967

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	L P (	icansed ower kW)	i Hours of Operation	Hours of Use	Average No. of 40-h Shifts	Integrated Power in MWh	Total No. of Samples	No. of Vsers	Beam Port h	Thermal Column h	Sample Irrad. Facility h	Bulk Irrad. Facility h	In-core Exp. h	No. of Facility Users
1.	U	5000	1,207.9	1,488.8 ( 3)	2.954 (3)	5,847.2 (1)	773 ( 5)	31 (2)	5,244.0 (2)	597.0 ( 2)	1517.0 (1)	352.0 ( 2)	1090.0 (4)	13 ( 4)
2.	U	2000	404.1	744.0 ( 9)	1.4/0 ( 9)	1,019,5 (0)	010 ( /)	e (12)	1,213.5 ( 3)	~	44.0 (10)	14.7 ( 0)	43.3 (12)	14 ( ))
3.		2000	1,8//.5	2,105.0 (1)	4.1/6 (1)	3,710.4 (2)	62 (13) 57/ ( 0)	8 (L3) 22 ( 1)		U V	455.9 (2)	0 660 0 6 11	3332.4 (1)	
- 2.	0	2000	1,012.0	1,932.0 ( 2)	2 821 ( 4)	2 532 0 ( 4)	574 (8)	52 (1)	2 520 0 ( 3)	13260(1)	347.0 (4)	7 0 (10)	2122 0 / 21	39 (1)
6.		1500	1 113 0	1 178 0 ( 5)	2.001 (4)	1 669 5 ( 5)	- 0	0	2,52,50 ( 57 Y	X X X	10.0 (17)	7.0 (10)	11130(2)	1 (28)
7.		1500	217.0	282 0 (19)	0 559 (19)	(0.1	0	0	x	Ŷ	ň	151 0 ( 3)	46 0 (11)	5 (16)
8.	11	1000	446.9	755 6 ( 8)	1 / 90 ( 8)	1 012 8 ( 7)	372 (12)	16 ( 6)	2.8 (18)	ô	332 6 ( 5)	151.0 ( 5)	403 A ( 5)	19 ( 2)
9.	ŭ	1000	535.0	560 D (10)	1 111 (10)	537.0 ( 8)	131 (17)	12 (8)	737.0 ( 6)	1.0 (10)	181.0 ( 7)	4.0 (12)	45514 ( 5)	12 ( 5)
10.	ŭ	1000	429.2	463 0 (11)	0 919 (11)	390.4 ( a)	34 (26)	5 /10)	378.9 ( 8)	Y (10)	386 4 ( 3)	A (12)	õ	A (18)
11.	ម	1000	275.8	390.0 (15)	0.774(15)	14.5 (15)	41 (25)	5 (20)	54.0 (14)	ô	10.0 (18)	õ	ŏ	3 (24)
12.	Ū	1000	225.0	300.0 (18)	0.595 (18)	170.0 (10)	400 (11)	7 (14)	4.0 (17)	ō	8.0 (19)	25.0 (6)	ñ	3 (25)
13.	บ	250	503.8	807.5 ( 6)	1.602 ( 6)	72.5 (11)	182 (16)	11 (10)	422.5 (7)	68.5 (4)	131.5 ( 8)	0	111.5 (9)	9 (10)
14.		250	161.0	226.0 (21)	0.448 (21)	36.2 (13)	1,285 (2)	20 (4)	X	X	0	õ	94.0 (10)	6 (13)
15.	ម	200	285.0	415.0 (14)	0.823 (14)	2,1 (20)	646 (9)	6 (18)	290.0 ( 9)	х	33.0 (12)	Ó	128.0 (7)	_
16.	U	100	762.9	792.0 ( 7)	1.571 (7)	65.6 (12)	218 (14)	21 (3)	2,371.4 (4)	х	0	25.5 ( 5)	a	11 ( 6)
17.	ប	100	349.1	363.3 (16)	0.721 (16)	16.3 (14)	72 (22)	7 (15)	210.5 (10)	78.5 (3)	0	0	0	4 (19)
18.		100	67.8	339.0 (17)	0.673 (17)	7.0 (17)	2,619 (1)	10 (11)	X	x	13.0 (16)	55.0 (4)	0	10 (8)
19.	U	100	153.7	238.0 (20)	0.472 (20)	8.4 (16)	453 (10)	4 (22)	94.2 (13)	39.6 (5)	6.3 (21)	0	٥	6 (14)
20.	U	100	145.0	226.0 (22)	0.448 (22)	1.4 (22)	204 (15)	7 (16)	0	0.5 (12)	0	0	7.0 (16)	3 (26)
21.	U	100	116.8	207.2 (23)	0.411 (23)	6.0 (19)	76 (20)	-	7.0 (16)	8.0 (7)	0	8.0 (9)	127.0 (8)	4 (20)
22.	U	100	50.0	200.0 (24)	0.397 (24)	0.5 (23)	1,050 (4)	3 (24)	X	х	0	0	0	0
23.		100												
24.	U	100	14.0	38.0 (29)	0.075 (29)	0.2 (26)	13 (27)	2 (25)	2.0 (19)	6.0 (8)	0	7.0 (11)	σ	4 (21)
25.		18	338.0	446.5 (12)	0.886 (12)	5.3 (18)	1,171 ( 3)	16 (7)	х	х	322.0 ( 6)	Ó	0	6 (15)
26.	U	10	389.3	430.5 (13)	0.854 (13)	1.6 (21)	515 ( 6)	18 ( 5)	96.0 (12)	6.0 ( 9)	42.0 (11)	1.0 (13)	202.0 ( 6)	10 ( 9)
27.	ប	10	78.0	138.0 (25)	0.274 (25)	0.3 (25)	218 (13)	12 ( 9)	14.0 (15)	1.0 (11)	17.0 (14)	16.0 (7)	11.0 (15)	9 (11)
28.	U	10	52.2	85.5 (26)	0.169 (26)	< 0.1	64 (23)	4 (23)	0	16.1 ( 6)	2.3 (22)	0	32.8 (13)	6 (15)
29.	u	10	55.2	75.1 (27)	0.149 (27)	0.4 (24)	76 (21)	7 (17)	1.9 (20)	х	60.2 (9)	0	16.7 (14)	4 (22)
30.	U	10	36.0	66.0 (28)	0.131 (28)	< 0.1	6 (28)	2 (26)	X	х	16.3 (15)	0	0	2 (27)
31.	U	10	29.5	35.7 (30)	0.071 (30)	0.1 (28)	104 (18)	5 (21)	1.0 (21)	0	28.5 (13)	0	0	4 (23)
32.	U	10	18.0	27.5 (31)	0.055 (31)	0.2 (27)	48 (24)	-	0	0	7.0 (20)	0	0	5 (17)
	TOTAL	: .	13,418.7	16,782.2		21,182.7	12,043	279	25,892.2	2146.2	3973.8	10,325.2	9992.3	226
	AVERA	GE:	433	541		683	401	9.96	1,079	119	128	333	322	7.79

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U denotes university reactor. Numbers in parentheses represent the rank (1 highest) of that reactor in each category. X indicates that this facility does not exist at that reactor.

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TABLE 5 First Quarter of 1968

		Licenser Power (kW)	i Hours of Operation	Hours of Use	Average No. of 40-h Shifts	Integrated Power in MWh	Total No. of Samples	No. of Users	Beam Port h	Thermal Column h	Sample Irrad, Facility h	Bulk Irrad. Facility h	In-core Exp. h	No. of Facility Users
1.	U	5000	1,257.9	1,531.4 ( 4)	2.991 (4)	6,075.9 (1)	732 ( 6)	40 ( 1)	3,636.0 ( 2)	283.0 ( 2)	1569.0 ( 1)	321.0 ( 2)	800.0 ( 4)	11 ( 6)
2.	U	5000	677.9	984.0 (7)	1.921 (7)	2,365.9 ( 6)	561 (8)	12 (8)	2,033.7 (5)	x	354.3 ( 5)	10.9 (8)	600.4 (5)	16 (2)
3.		2000	1,695.4	2,184.0 ( 1)	4.266 (1)	3,357.6 (2)	44 (24)	11 (9)	500.4 ( /)	0	100.3 (8)		4,579.2 (1)	11 (7)
4.	υ	2000	1,784.0	1,976.0 (2)	3.859 (2)	3,348.0 (3)	521 (10)	29 (2)	11,812.0 (1)		362.0 ( 4)	11,993.0 ( 1)		12 (4)
3.		2000	1,241.0	1,388.0 ( 5)	2.711 ( 5)	2,594.0 (4)	-	-	2,050.0 (4)	1263.0 ( 1)	-	U	4,245.0 ( 2)	0
6.		1500	1,633.0	1,698.0 (3)	3.316 ( 3)	2,449.5 ( 5)	0	0	X	x	Ű	0	1,633.0 ( 3)	1 (27)
7.		1500	51.0	116.0 (26)	2.260 (26)	-	0	0	X	X	0	0	51.0 (12)	6 (14)
8.	U	1000	359.8	499.3 (10)	0.975 (10)	139.3 (10)	520 (9)	0	115.8 (12)	128.3 (3)	460.8 (3)	10 0 0 0	220.7 (7)	33 (1)
9.	U	1000	968.0	995.0 ( 6)	1.943 (6)	956.0 (7)	109 (19)	11 (10)	1,470.0 ( 0)	4.0 ( 9)	600.0 (2)	14.0 ( 6)	U O	11 (8)
10.	U	1000	305.0	365.0 (14)	0.713 (14)	210.7 (9)	54 (23)	. 0 (13)	133.0 ( 9)	л с г ( л)		313.5 ( 3)	U	4 (10)
11.	U	1000	228.1	305.0 (16)	0.596 (16)	47.7 (12)	70 (20)	5 (17)	//.5 (14)	0.5 (0)	13.0 (10)	11 0 ( 7)	0	4 (19)
12.	ប	1000	300.0	390.0 (12)	0.762 (12)	243.0 (8)	390 (11)	4 (21)		U O	17.0 (15)	11.0 ( 7)		4 (20)
13.	U	250	309.0	615.0 ( 9)	1.201 (9)	27.6 (14)	137 (17)	10 (11)	120.2 (10)	<u>v</u>	13322 (1)	0	11/.5 (10)	3 (22)
14.	۰	250	127.0	192.0 (22)	0.375 (22)	28.6 (13)	1,255 (3)	19 ( 5)		X (10)	0	0	64.0 (11)	4 (21)
15.	U	200	178.0	284.3 (19)	0.555 (19)	1.6 (22)	696 ( /)	4 (22)	49.0 (16)	4.0 (10)	9.0 (19)	U	2/2.0 ( 6)	10 (10)
16.	U	100	785.6	819.0 (8)	1.599 (8)	68.7 (11)	1/1 (16)	20 (4)	2,145.2 ( 3)	× · · · · · ·	U O	U	6.0 (15)	5 (15)
17.	U	100	246.0	292.0 (18)	0.570 (18)	10.7 (16)	62 (21)	5 (18)	134.3 (11)	41.1 (4)	1 0 (17)	77 0 ( ()	U	3 (13)
18.		100	51.4	257.0 (20)	0.502 (20)	4.9 (18)	1,357 (1)	10 (12)	X A ARX	X	14.0 (17)	37.0 (4)	0	11 ( 7)
19.	U	100	103.3	238.0 (21)	0.465 (21)	4.9 (19)	893 (5)	9 (14)	54.2 (15)	20.0 ( 6)	30.7 (9)	U	Ų	14 ( 5)
20.		100						F (10)	2 2 (22)	•	•	2 0 (10)	124 8 4 83	7 (75)
21.	U	100	153.7	298.7 (17)	0.583 (17)	4.6 (20)	33 (26)	5 (19)	3.0 (20)	0	0	2.0 (10)	134.0 ( 9)	2 (25)
22.	U	100	30.0	100.0 (27)	0.195 (27)	0.1 (28)	1,000 (4)	2 (25)	X	X	0 5 (10)	0	61 (14)	0 (11)
23.	U	100	63.8	72.8 (29)	0.142 (29)	0.6 (24)	31 (27)	B (10)	3.8 (19)	ů,	9.5 (18)	70(0)	6,1 (14) 0	3 (23)
24.	U	100	288.0	378.0 (13)	0.738 (13)	24.5 (15)	43 (25)	3 (24)	251.0 ( 8)	0	30.0 (10)	7.0 ( 9)	0	0 (17)
25.		18	374.9	469.2 (11)	0.916 (11)	5.9 (1/)	1,316 ( 2)	19 ( 0)		A A / E	337.0 ( 0)	0	1// 0 / 0	7 (12)
26.	U	10	297.4	327.4 (15)	0.639 (15)	0.9 (23)	327 (13)	10 (13)	30.0 (13)	30.0 ( 3)		0	144.0 ( 8)	12 (5)
27.	U 	10	79.0	150.0 (23)	0.293 (23)	0.4 (25)	100 (12)	5 (20)	30.0 (17)		30.0 (11)	0	47.0 (13)	2 (26)
28.	0	10	91.1	136.5 (25)	0.200 (25)	0.2(27)	100 (15)	10 ( 7)	-	10,2 ( /)	10 6 (14)	0	1 4 (16)	- (20)
29.	U 	10	02.0	146.0 (24)	0.285 (24)	1.9 (21)	50 (22)	1 (26)	v	A V	10.0 (14)	10 4 / 51	0	3 (24)
JU.	0	10	95.0	12 0 (30)	0.000 (30)	0 1 /201	19 (22)	4 (23)	1 0 (21)	A 0	6 0 (20)	1310 ( 3)	ñ	5 (16)
31.		10	10.2	12.0 (31)	0.023 (31)	0.2 (26)	126 (18)	-	1.0 (21)	2 0 /11)	2 5 (22)	ň	ñ	5 (17)
32.	U	10	28.0	//.5 (28)	0.131 (28)	0.3 (20)	120 (10)	-	0.0 (18)	2.0 (11)	2.3 (22)	U	v	5 (27)
	TOTA	uL:	13,879.1	17,331.8		21,974.1	11,300	291	24,765.4	2115.1	4157.5	12,731.0	12,922.1	216
	AVER	AGE:	448	559		732	377	10.0	1,077	111	139	411	417	7.4

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U denotes university reactor. Numbers in parentheses represent the rank (1 highest) of that reactor in each category. X indicates that this facility does not exist at that reactor.

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FIGURE 1 Correlation of reactor power and the hours used. The increased use at higher power is quite dramatic.

a particular item indicates the rank of that reactor (1 being highest) in that column for the category being considered.

#### General Operational Data

Tables 4 and 5 include entries for hours of operation and hours used during the three-month periods covered. By hours of operation is meant the actual hours the reactor was "at power." In many installations, additional time is involved in changing experiments and other activities, and this requires that the reactor "not run." The entries for hours of use include this required "down time." Numbers in parentheses indicate the rank of each reactor in the entire group, from the standpoint of hours used.

Note that there is a strong correlation between reactor power and hours used, as shown in Figure 1, in which the average hours the reactors were used are plotted against their licensed power. All reactors having the same power have been averaged together. The increased use of the higher powered reactors is rather dramatic.

## Number of Work Shifts

It is customary to hire reactor operators for a 40-h week, which they fulfill on an intermittent basis. In the fourth quarter of 1967 the standard 40-h-week shift, adjusted for vacations, equaled 504 h. Under the column in Table 4 marked "shifts," the number of 504-h standard shifts for that quarter is given. It is noted that one reactor indicates over 4.17 standard shifts, which means that it ran continuously without shutdown for the entire quarter. The table shows that 10 out of the 31 reactors reporting hours of use and operation were operated for more than a one-shift period. During the first quarter of 1968 there were 512 h in a standard 40-h-week shift. It will be seen that the reactors reporting averaged slightly more than a one-shift operation during both quarters. Figure 2 indicates the distribution of the reactors in terms of the percentage of one-shift operation.

Although most reactors are run for less than one shift (i.e., less than 8 h per day), the 12 that operated at or above 1 MW ran for an average of 2.2 40-h shifts per week. The eight reactors operating above 1 MW ran for an average of 2.9 40-h shifts per week.

Figures 3 and 4 present data on integrated power (megawatt hours) of the reactors. This is indicative of fuel consumption,



FIGURE 2 Comparison of the usage of the reactors in terms of one-shift operation. The majority of the reactors are operating for one shift or less.

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fuel costs, and operating costs and is correlated with the licensed power levels of the reactors. As would be expected, usage and costs are directly related. The higher-power reactors burn more fuel because they are at higher powers; they also were used the most. The range of energy consumption by these reactors is wide, with an average of 700 MWh. From this figure one can predict an average power of 1.4 MW for a 500-h quarter. This again emphasizes the fact that the higher-powered reactors are used most, since the licensed power of the median reactor in the surveyed group is 100-200 kW.

### Production of Radioisotope Samples

The survey determined one aspect of research reactor operation that had not previously been documented--the extent to which reactors assist in research efforts by the generation of radioisotope samples. This aspect of the reactors surveyed here is shown in Tables 4 and 5, which show the number of radioactive samples generated by each reactor. The number in parentheses is the rank number (highest to lowest) of the samples made. Thirty of the reactors surveyed provided records on radioactive samples generated. Twentyeight of them did generate samples during the quarters surveyed, for an average of 400 samples per quarter. The median number was 200 samples. Using the figure of approximately 65 working days in a quarter, all the reactors were supplying on the order of six or more samples each per day of operation. The top reactor supplied 41 radioactive samples per working day during one of the quarters.

All the reactors surveyed seem to be capable of generating radioisotope samples for research users. In Figure 5 the average number of samples made by the various types of reactors is plotted against licensed power levels. One finds no particularly dramatic trend. Figure 6 shows the distribution of reactors relative to number of samples manufactured per quarter. The bar graphs for the two quarters surveyed show relatively similar results.

The survey examined in general the kinds of radioisotope samples being produced by the research reactors studied, via the length of the periods of irradiation. Tables 6 and 7 indicate the irradiation times of the samples generated. The results indicate rather clearly that most of the samples were irradiated for less than 1 h. Although short-lived samples predominated, all types of radioisotopes were apparently generated. The study divided them into three groups: samples involving irradiation of less than 1 h; samples irradiated from 1 to 8 h; and samples irradiated for longer than 8 h. It might be noted that samples irradiated for more than 8 h were undergoing this process for rather long periods of time (i.e., 60 h or 150 h).



FIGURE 5 Production of radioactive samples is correlated with reactor power. Distribution indicates that this is independent of power level.

FIGURE 6 Radioactive sample production in terms of number of reactors and number of samples is given for the two calendar quarters studied.





	Licensed Power (kW)	< 1 h No. of Samples	l-8 h No. of Samples	Hours	> 8 h No. of Samples	Hours	Total No. of Samples	No. of Users
1.	5000	571	165	249	37	1049	773	31
2.	5000	595	19	1072	2	120	616	8
3.	2000	50	25	80	7	3775	82	8
4.	2000	412	106	361	56	9233	574	32
5.	2000	-		-	-	-	-	-
6.	1500	0	0	0	0	0	0	0
7.	1500	0	0	0	0	0	0	0
8.	1000	78	178	595	116	934	372	16
9.	1000	117	7	27	7	101	131	12
10.	1000	19	8	19	7	322	34	· 5
11.	1000	36	5	10	0	0	41	5
12.	T000	200	200	1000	-	_	400	7
13.	250	148	16	58	18	221	182	11
14.	250	1210	75	525	0	0	1285	20
15.	200	341	291	1106	14	112	646	6
10.	100	131	6/	193	20	2119	218	21
1/.	100	62	10	42	_	-	72	7
18.	100	2619	-	-	-	-	2619	10
19.	100	54	399	1826	0	0	453	4
20.	100	203	1	105	0	U	204	/
21. 22	100	1000	48	105 100	-	_	75	-
22.	100	1000	30	100	0	0	T020	3
23.	100	10	-	_	_	_	-	_
24.	18	£10	550	507	U	U	1171	2
26.	10	201	207	1266	- 17	1/9	515	10
27	10	195	237	1300	17	140 0	212	10
28.	10	64	25	45	_	0	210	12
29.	10	30	21	70	6	03	76	4 7
30.	10	0	5	06	0	<u> </u>	70 6	2
31	10	103	1	90	0	0	104	۲ ۲
32	10	23	25	237	0	0	48	-
	TOTAL:	9131	2605	9789	307	18,227	12,043	279
·	AVERAGE:	304	87	326	10	608	401	10

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TAB 5 Fourth Quarter of 1967

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	Licensed Power (kW)	< 1 h No. of Samples	1-8 h No. of Samples	Hours	> 8 h No. of Samples	Hours	Total No. of Samples	No. of Users
1.	5000	620	59	185.0	53	1022.0	732	40
2.	5000	546	3	30.0	12	7546.4	561	12
3.	2000	25	12	49.2	7	5110.4	44	11
4.	2000	362	73	130.0	86	11863.0	521	29
5.	2000	-			-	-	-	-
6.	1500	0	0	0	0	0	0	0
7.	1500	0	0	0	0	0	0	0
8.	1000	250	254	696.4	16	625.9	520	0
9.	1000	78	13	25.0	18	839.0	109	11
10.	1000	8	34	39.0	12	949.0	54	· 6
11.	1000	62	8	25.0	0	0	70	5
12.	1000	240	150	750.0	0	0	390	4
13.	250	66	69	133.3	2	24.3	137	10
14.	250	1220	35	105.0	0	0	1255	19
15.	200	177	518	2200.0	1	8.8	696	4
16.	100	47	94	281.1	30	1454.2	171	20
17.	100	57	4	15.0	1	24.0	62	5
18.	100	1357	0	0	0	0	1357	10
19.	100	407	486	1114.9	0	0	893	9
20.	100	-	_		-	-	-	-
21.	100	0	33	110.0	0	0	33	5
22.	100	1000	0	0	0	0	1000	2
23.	100	31	0	0	0	0	31	6
24.	100	37	6	18.0	0	0	43	3
25.	18	752	564	594.6	0	0	1316	19
26.	10	81	233	815.0	13	112.0	327	22
27.	10	311	16	16.0	6	234.0	333	10
28.	10	153	27	42.0	0	0	180	5
29.	10	250	11	14.6	0	0	261	19
30.	10	57	2	7.5	0	0	59	1
31.	10	17	2	10.0	0	0	19	4
32.	10	107	19	323.0	0	0	126	ò
	TOTAL:	8318	2725	7729.6	257	29,813.0	11,300	291
	AVERAGE:	277	91	257.7	8.6	993.8	377	9.7

.

TABLE 7 First Quarter of 1968

Port Hours Used	Hours Reactor Used	Average Number of Ports Used
12,110.0	1932.0	6.27
5,244.0	1488.8	3.52
2,529.0	1427.0	1.77
2,371.4	792.0	2.99
1,213.5	404.1	3.00
737.0	560.0	1.32
422.5	807.5	0.52
378.9	463.0	0.82
290.0	415.0	0.70
210.5	363.3	0.58
108.5	2105.0	0.05

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TABLE 8 Fourth Quarter of 1967--Port Usage

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On-Site Facility Use

The survey catalogued on-site facility use under five general areas, as indicated in Tables 4 and 5. They are: beam port, thermal column, sample irradiation, bulk irradiation, and in-core activity. For much of this activity it is not possible to determine appropriate averages. In some cases (as indicated by an "X"), the reactor did not have the specified type of facility. In others, the reactors did not keep accurate data as to usage. For reactors that did indicate usage in their reports, Tables 4 and 5 set forth in parentheses the rank of the facilities in order of usage (from highest to lowest).

Because many of the reactors surveyed did have beam ports, and used them during the quarters studied, they were requested to provide data on total port-hours of usage. This total, divided by the total hours of reactor usage, yields the average number of ports used at the reactor, as shown in Table 8. The higher-powered reactors showed better port usage.

The survey indicates that there was a wide variation in the use of reactor facilities. Most reactors showed a rather large use of in-core experiments (see Tables 4 and 5).

## Research Reactor Users

Users of research reactors were divided into two categories: users of radioisotope samples and on-site users of facilities.

Isotope Sample Users The number of research people involved in the use of isotope samples is, of course, an important indicator of reactor use. As shown in Tables 4 and 5, an average of 10 different people used radioisotope samples from each reactor during a typical quarter. The median for the group was 7; the maximum number of users was 32. This indicates that the reactors are supporting research for a rather large number of people at the various laboratories and universities. As can be seen in Tables 4 and 5, the range in the number of users of radioisotope samples is not nearly so great as the range of the number of samples.

Although the number of samples produced by the reactors showed no strong trends related to power levels, it is clear that the number of sample users did increase with increasing power level. This is indicated in Figure 7, a graph of average number of sample users for the group of reactors of the various power levels. (An inconsistency appears with respect to 1.5-MW reactors, probably due to the fact that the two reactors represented are considered as pulse







FIGURE 8 Number of reactors supporting various numbers of radioisotope users.





FIGURE 9 The number of users of reactor experimental facilities is correlated with licensed reactor power level. An increase is noted as the power of the reactor increases.



FIGURE 10 Number of reactor facilities having various numbers of facility users.

Background	Percentage	e of Total
PHYSICAL SCIENCE		
Physics	12	
Chemistry	26	
Geology	6	
555-585	-	44
BIOLOGICAL SCIENCE		
Biology	7.5	
Medicine	5.5	
Health Physics	4.5	
,		17.5
ENGINEERING		
Nuclear	26	
Electrical	2	
Mechanical	1	
		29
MISCELLANEOUS	9.5	
	• • • •	9.5
		100.0

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TABLE 9 Sample Users by Background

TABLE	10	Facility	Users	Ъу	Background	1

Background	Percent	age of Total
PHYSICAL SCIENCE		
Physics	20	
Chemistry	10	
Geology	2	
0.		32
BIOLOGICAL SCIENCE		
Biology	5	
Medicine	5	
Health Physics	5	
		1.5
ENGINEERING		
Nuclear	44	
Electrical	0	
Mechanical	2	
		46
MISCELLANEOUS	7	
		7
		100
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reactors and not normally used for generating radioisotope samples. The data are included for completeness.) Figure 8 shows the distribution of number of users at the various reactors, for the two quarters studied.

On-Site Users of Facilities Hitherto, there has seemed to be little data concerning the number of people using such facilities at reactors as beam ports, thermal columns, and in-core experiments. Tables 4 and 5 indicate the number of users of such facilities at the reactors surveyed; the number in parentheses indicates the rank of each reactor in the particular category of use. It will be seen that the reactors average eight users during a typical quarter, the median being six.

The average number of users of facilities for the various groups of reactors at specified power levels is shown in Figure 9. Here again it is clear that there are markedly more users of facilities at the higher-powered reactors. Figure 10 presents the distribution of on-site users of facilities at the various reactors, for both quarters studied. There is no particular difference between the two quarters. It should be noted that one reactor reports over 30 users in both quarters.

#### Background of Research Reactor Users

The technical background of the users of the research reactors surveyed is shown in Table 9 (sample users) and Table 10 (facility users), using the broad groupings of physical science, biological science, and engineering. Since not all the reactors surveyed submitted data on background of users, these tables indicate percentage of total users rather than actual numbers of users. Approximately two thirds of the users of the survey reactors are identified, however, so the results do have some significance.

Most of the reactor installations are operated by nuclear engineering groups. It is accordingly not surprising to find nuclear engineering as a large user category, on a percentage basis, in Table 9 and an even larger user in Table 10. At the same time, the tables show that use of the reactors is by no means limited to nuclear engineers.

As shown in Table 9, chemists and nuclear engineers were the largest groups of users of radioisotope samples, although physicists and biologists were well represented. Approximately 60 percent of the users of samples were involved in activation analysis, using over 90 percent of all the radioisotope samples reported. The types of activation analyses conducted are listed in Appendix C. On the average, the users of radioisotope samples received about 40 samples per quarter.

The facility users, shown in Table 10, are those who utilize such facilities as the reactor beam ports, the thermal columns, and the in-core facilities. As would be expected, these facilities are used primarily by nuclear engineers and physicists. The research often involves large and complicated items of equipment that must be located at the reactor site. Substantial use of the facilities is also made by research workers from chemistry and the biological sciences.

#### Types of Research Activity

As part of the survey, information was requested about the research programs conducted using the research reactors. A partial list of programs is set forth in Appendix C. Most of the work is identifiable under the following headings: activation analysis, radiation effects, tracer work, radiological chemistry, biomedical engineering, nuclear physics, engineering, radiographic radiation, and materials.

This list indicates the broad multidisciplinary character of the research performed by use of a nuclear reactor. Both isotope users and facility users indicated a large and impressive variety of research.

With further reference to Table 10, it is possible to deduce from the questionnaire responses that the following disciplines were represented among the people using experimental facilities in connection with the research reactors surveyed: radio or nuclear chemistry, nuclear physics, solid-state physics, radiation biology, medicine, veterinary medicine, plant physiology, materials engineering, nuclear engineering, electrical engineering, mechanical engineering, radiological health physics, and geology.

This discipline list is incomplete but represents a wide variety of disciplines involved in the conduct of experiments using reactor radiations. The users of activation analysis data represented the largest range of disciplines and included such groups as police departments and state and local legal organizations.

The broad and important character and ingenuity of the research is readily apparent from a perusal of the experiment lists in Appendix C. For example, in biomedical engineering, research is being performed to enlarge the understanding of certain human and animal body functions necessary to treating disease. Activation analysis is rapidly becoming an indispensable analytical tool used in many branches of science and engineering. Radiation-effects studies are most important to our future space-exploration programs and (to the Department of Defense) in connection with the development of radiation-resistant missile systems.

# APPENDIX A

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## RESEARCH REACTOR UTILIZATION DATA

Name of Reactor	( ) Quarter of 1966
Reported by	
Address	Telephone
I. REACTOR OPERATION	
Hours reactor is in operation	hrs
Hours reactor is used (in- clude scheduled down time	
checkouts, etc.)	hrs
Integrated power	megawatt hours

# II. IRRADIATIONS

A. Time Used for Irradiations

TIME	NO. OF SAMPLES	# OF SAMPLES X HOURS
Less than 1 hr		
1 - 8 hrs		
Above 8 hrs		
Grand Total	,	

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B. Users of Irradiations (Please list any publications of the research below an entry if such occurs during period of report.)

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USER IDENTITY		DEPARTMENT	
(Names of Persons,	NO. OF	DIVISION AND/	TYPE OR TITLE
Company, etc.)	SAMPLES	OR DISCIPLINE	OF RESEARCH
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III. EXPERIMENTAL FACILITIES (Includes Ports, Thermal Column, etc.) Under type designate each port or facility by some suitable method.

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A. Time Used

FACILITY	TYPE	TIME (HRS)
1		
2		
3		
4		
5		
6		
7		
8		
9		
10		
11		
12	L	
	Total	

B. Port or Facility Users (Please list any publications of the research below an entry if such occurs during period of report.

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		DEPARTMENT,	
		DIVISION AND/	TYPE OR TITLE
FACILITY #	USER IDENTITY	OR DISCIPLINE	OF RESEARCH

#### IV. REACTOR SYSTEMS

A. Time Used for Reactor Systems

 TIME (HRS)

B. Systems Investigators (Please list any publications . . . etc

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USER IDENTITY (Names of Persons, Company, etc.)

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DEPARTMENT, DIVISION AND/ OR DISCIPLINE

TYPE OR TITLE OF RESEARCH

V. MISCELLANEOUS NON-RESEARCH OPERATION

Type of Activity

<u>Hours</u>

## APPENDIX B

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# RESEARCH REACTORS

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Name	Authorized Power Level (as of Dec. 31, 1967)
Aerojet-General Corp.	250 kW
(Industrial Reactor)	
Armed Force Radiobiology	
Research Institute	100 kW
Babcock & Wilcox Co.	1000 kW
Battelle Memorial Institute	2000 kW
Cornell University	100 kW
General Electric Co.	30 kW
Georgia Institute of Technology	1000 kW
Gulf General Atomic, Inc.	250 kW
Gulf General Atomic, Inc.	1500 kW
IIT Research Institute	75 kW
Industrial Reactor Laboratories, Inc.	5000 kW
Iowa State University	10 kW
Kansas State University	100 kW
Lockheed Aircraft Corp.	3000 kW
Massachusetts Institute of Technology	5000 kW
National Aeronautics and Space	
Administration	100 kW
North Carolina State University	10 kW
Northrop Corporate Laboratories	1000 kW
Ohio State University	10 kW
Pennsylvania State University	1000 kW
Purdue University	10 kW
Rhode Island & Providence	
Planations AEC	1000 kW
Stanford University	10 kW
Texas A&M University	100 kW
Union Carbide Corporation	5000 kW
U.S. Army Materials Research Agency	2000 kW
U.S. Naval Research Laboratory	1000 kW
University of Arizona	100 kW
University of California	100 kW
University of California	1000 kW
University of Florida	100 kW
University of Illinois	250 kW

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University of Kansas	10	k₩
University of Maryland	10	k₩
University of Michigan	2000	kW
University of Missouri	200	kW
University of Missouri	5000	kW
University of Texas	10	kW
University of Virginia	1000	kW
University of Washington	100	kW
University of Wisconsin	1000	kW
V.A. Hospital	18	kW
Virginia Polytechnic Institute	100	kW
Walter Reed Army Institute of Research	50	kW
Washington State University	1000	kW
Western New York Nuclear Research		
Center	2000	kW
Worcester Polytechnic Institute	10	k₩
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APPENDIX C

#### ACTIVATION ANALYSIS

Activation analysis of fallout samples Archaeological investigation by activation analysis Identification of trace impurities by the use of activation analysis Trace-element detection Neutron activation using cobalt-60m Determining uranium content by fission tracks in mica Forensic application Arsenic content in cigarettes <sup>80</sup>Br analysis Activation analysis of archaeological specimens Activation analysis to measure trace quantities of elements in rainwater to evaluate the effectiveness of various cloud-seeding techniques Activation analysis to evaluate the halogen contents of natural and pollution aerosols, atmospheric precipitation, and surface water Age of mica determination by formation of fission tracks Activation analysis for measuring gold in urine of patients who have undergone gold therapy Activation analysis to determine sodium in alumina ceramic Evaluate a technique to determine elemental composition of the top layers of materials Activation analysis to determine trace-element anomalies in drinking water supplies and relating their distributions to the epidemiology of heart disease Activation analysis to determine the age, physiological state, mating, species or strain, and food products of insects Determination of organic chloride in pesticide molecules Neutron activation analysis of trace elements in textile fibers Activation-analysis determination of herbicide residues Activation-analysis determination of DDT residues in cucumbers Production of cobalt and zinc activation-analysis standards Irradiation of redwing blackbird bones and beaver teeth for activationanalysis studies Neutron activation of trace elements in blood serum and tissue Trace-element analysis in seawater Forensic applications of neutron activation analysis Activation analysis source production Neutron activation analysis of organic samples Trace-element studies in rock samples Activation analysis of organic semiconductors

Activation analysis for Au determination in urine, blood, and body tissues Fluoride in treated cloth Fluorine in hydrocarbone Vanadium in oil Oxygen in metals Mercury in foodstuffs Barium and antimony in gunshot residues Uranium in ore samples Bromine in foodstuffs Antimony in bullet lead Silver in rainwater Coding elements for industrial tagging Trace elements in hair Sodium in fingernails Trace elements in blood Rare-earth elements in rocks, minerals, and ores Neutron activation analysis of marine sample Identification of fission products Rare-earth abundances Neutron activation of marine samples Development of techniques for activation analysis and radiochemical experimentation Capture gamma activation analysis Prompt gamma activation analysis

## RADIATION EFFECTS

Radiation effects on organic coolants Radiation effects on crystalline materials Radiation damage of Si Crystals Damage studies in Cu A study of thermal neutron capture recoil damage by observing thermoluminescence spectra, electrical conductivity changes, electron spin resonance characteristics, and Mössbauer spectra Expose thin mica films to fission fragments to produce microporous films to study liquid-phase diffusion of molecules Radiation effects in solids Effects of radiation on electronic components Radiation effects of organic coolants TRACER WORK

Age dating Sn migration in teeth <sup>18</sup>F production for clinical studies Tool wear studies Iodine detection Develop a technique to determine traces of phosphate ion in natural waters in the range of 4-200 parts per billion of phosphorous Irradiate tungsten carbide for tool wear determinations Irradiate sand and dust for engine wear studies Provide labeled oil for oil consumption determinations Production of <sup>51</sup>Cr as trace chromium in NiO samples Coding elements for industrial tagging

#### RADIOLOGICAL CHEMISTRY

High-velocity recoil atom reactions
High-energy chemical reactions induced by thermal neutron fission and (n,γ) recoil processes
Rapid and specific chemical techniques for isolation of radioactive products
Development of very rapid (10-60 sec) electrolytic separation techniques for radioactive isotopes with half-lives of the order of 2 to 100 sec
Halogen reactions in cyclohexane
Chemical effects of isomeric transition
Solvent extraction for fused salts

#### BIOMEDICAL ENGINEERING

Sn migration in teeth
Sodium transport across rumen epithelium
K and Na transport in blood
Use of short-lived isotopes for diagnosis
Effect of various drugs on the distribution of radioactive potassium,
 cesium, and rubidium in rat tissues
Potassium determination in patients undergoing open heart surgery
Heart scanning in dogs
Determination of total body potassium in humans
Kidney uptake studies in dogs
Studies of the relationship between potassium levels in cells and
 amino acids

Studies of the relationship between potassium levels and hippocampal seizures Blood-flow studies Activation analysis to determine the age, physiological state, mating, species or strain, and food products of insects 18<sub>F</sub> production Effect of mixed neutron and gamma-ray radiation on grass seed Cancer therapy research Neutron-capture therapy research RBE determination of the medical therapy Drug responsiveness in the preirradiated and postirradiated animal The behavioral decrement and incapacitation response to acute radiation injury The organ and system response to acute radiation injury The effects of ionizing radiation upon lysosomes of mammalian tissues The clinical response of the total animal to acute radiation injury Effects of ionizing radiation on discrimination in the primate Neutron capture in tissue Chromosome variation in chickens following acute radiation Fast neutron biological irradiations NUCLEAR PHYSICS Isomer studies Decay Scheme of <sup>37</sup>A Decay Scheme Studies Beta- and gamma-ray spectroscopy of short-lived fission products Study of de-excitation of fission fragments produced in neutroninduced fission Production of radioactive sources for level scheme determinations Supply sources in conjunction with studies of low-lying nuclearenergy levels Fission product decay (short-lived isotopes) Decay scheme analysis Decay scheme of IIIAg Search for <sup>52</sup>Ti Level structure of <sup>117</sup>Sn and <sup>119</sup>Sn decay schemes of <sup>131</sup>Te Decay of <sup>88</sup>Rb Decay schemes for <sup>135</sup>I and <sup>135</sup>Xe Angular correlation studies on 129Te and 131Te rays Decay of 124Sb and 124I to 124Te

Decay scheme of  $^{69}$ Zn isomers. Yields of Zn fission products Atomic beam studies of  $^{75}$ Ge

Neutron time-of-flight energy system

Spectroscopy of neutron capture gamma rays Neutron decay spectroscopy Ternary fission Preliminary studies for delayed neutron source work Fission physics The study of the photofission of heavy nuclei near the fission threshold

#### ENGINEERING

Coated fuel evaluation Argon activation for monitor calibration Radiographs of fuel rods and plates Radiographs of fuel Radiographs of rods and pins Conduct study of type and extent of radioactivities that might be induced in the ground surrounding reactor Feasibility study for production of <sup>60</sup>Co calibration sources Subcritical pulsing Lattice physics research Void effects in fuel rod clusters Magnetohydrodynamic studies Laser reactor radiation coupling Importance function measurements Void coefficient measurement Development of a pile oscillator and measurement of the reactor transfer function Experimental study of radiation-induced nucleation of bubbles in superheated water Doppler effect in  $^{238}\mathrm{U}$  as a function of surface-to-mass ratio Capture gamma-ray shielding

## RADIOGRAPHY, RADIATION

Ferromagnetic prism refraction of neutrons
Refractive bending of a neutron beam by a magnetic field
Single-slit diffraction of neutrons
Basic mechanisms and controls of the dose in reactor exposure
volumes
Flux and dose studies and modifications related to the LINAC
facility
Development of dosimeters to measure tissue dose and the components contributed by the various types of radiation

Spectrum measurements AmBe neutron source investigations Neutron flux measurements--self-powered detector Spark counter studies Laser reactor radiation coupling A fast neutron spectrometer using polyethylene radiators and solid-state detectors for measuring fast neutron spectra Feasibility study of using small diamond flats as a dosimeter (EPR measurement Development of neutron \_adiographic techniques Determination of neutron spectra in lattices by foil activation Single-crystal neutron spectrometer for reactor spectrum measurement and associated experiments Neutron radiography Resonance fluorescense with recoil-broadened gamma rays Mechanical simulation of Doppler broadening Radiographic technique tests Fission-product production Radiographs of fuel rods and pins Radiographs of fuel rods and plates Radiographs of fuel Isotope production Gamma dosimetry Use of radioactive wires under the skin of mice to monitor daily activities in large natural enclosures Irradiate tantalum wires Irradiation of LiF or  $^{3}\text{He}$  to produce  $^{3}\text{H}$ Fission fragments in glass Autoradiography of a ruby Fission recoil in CsI Production of  $^{72}$ Ge sources for nuclear spectroscopy studies Feasibility study for production of  $^{60}$ Co calibration sources Production of <sup>51</sup>Cr as trace chromium in NiO samples Production of low-level <sup>60</sup>Co sources and development of flux wire method of power calibration Production of 198Au sources Production of 51Cr/58Co scintill-spectrometer calibration source Range of fission fragments in 235U Indium, Kr, Ar,  $CS_2$ , Nd activation  $56_{Mn}$  source Production of short-lived isotopes

MATERIALS

Heat of fission studies High-temperature study of iron magnetic scattering amplitudes Studies of (222) forbidden germanium reflection Studies of coherent paramagnetic scattering by vanadium Study of Kondo effect in a dilute alloy Neutron diffraction A triple-axis crystal spectrometer for neutron inelastic scattering measurements A four-rotor velocity selector for neutron inelastic scattering measurements A polarized-beam crystal spectrometer for neutron elastic scattering measurements on magnetic materials A crystal spectrometer for general neutron elastic scattering measurements Operate a crystal spectrometer for neutron elastic scattering studies Molecular-structure research Neutron diffractometer Radiation damage of Si crystals Expose thin mica films to fission fragments to produce microporous films to study liquid-phase diffusion of molecules Evaluate a technique to determine elemental composition of the top layers of materials